Docket Nos.: 50-369

50-370

Mr. H. B. Tucker, Vice President Nuclear Production Department Duke Power Company 422 South Church Street Charlotte. North Carolina 28242

Dear Mr. Tucker:

SUBJECT: LICENSE AMENDMENT CORRECTIONS

My letter dated May 6, 1987, forwarded Amendments 71 and 52 to Facility Operating Licenses NPF-9 and NPF-17 for the McGuire Nuclear Station, Units 1 and 2, respectively. Enclosed with the amendments were Technical Specifications page changes. One of those pages contained a typographical error. Please replace page 3/4 8-8 of the May 6 transmittal with the enclosed revised page.

My letter dated July 5, 1988, forwarded Amendments 88 and 69 to the McGuire operating licenses. One of the overleaf pages forwarded with the amendments contained an error. Please replace page 3/4 3-22 of the July 5 transmittal with the enclosed revised page.

Sincerely,

Original Signed By:

Darl Hood, Project Manager Project Directorate II-3 Division of Reactor Projects - I/II

Enclosure: As stated

GPA/PA

ARM/LFMB

MCGUIRE PLANT FILE

cc w/encl:
See next page

DISTRIBUTION Docket File		NRC PDR		Local PDR	
PDII-3 Reading		SVarga	14-E-4	GLainas	14-H-3
DMatthews	MRood	DHood	OGC 15-B-18	DHagan	MNBB-3302
EJordan	MNBB-3302	BGrimes	9-A-2	TBarnhart	(4)P1-137
Wanda Jones	P-130A				
E. Butcher	11-F-23				
ACRS (10)	H-1016				

8809150342 880902 PDR ADDCK 05000369

17-F-2

AR-2015

	•	116
·	2517	2517m
PDIJ-3	PDII-3	PDII-3
PDIJI-3	DHood:sw	DMatthews
7///88	9/1/88	9/2/88

Drol

Mr. H. B. Tucker Duke Power Company

cc: Mr. A.V. Carr, Esq. Duke Power Company P. O. Box 33189 422 South Church Street Charlotte, North Carolina 28242

County Manager of Mecklenburg County 720 East Fourth Street Charlotte, North Carolina 28202

Mr. Robert Gill
Duke Power Company
Nuclear Production Department
P. O. Box 33189
Charlotte, North Carolina 28242

J. Michael McGarry, III, Esq. Bishop, Liberman, Cook, Purcell and Reynolds 1200 Seventeenth Street, N.W. Washington, D. C. 20036

Senior Resident Inspector c/o U.S. Nuclear Regulatory Commission Route 4, Box 529 Hunterville, North Carolina 28078

Regional Administrator, Region II U.S. Nuclear Regulatory Commission 101 Marietta Street, N.W., Suite 2900 Atlanta, Georgia 30323

S. S. Kilborn
Area Manager, Mid-South Area
ESSD Projects
Westinghouse Electric Corporation
MNC West Tower - Bay 239
P. O. Box 355
Pittsburgh, Pennsylvania 15230

McGuire Nuclear Station

Dr. John M. Barry Department of Environmental Health Mecklenburg County 1200 Blythe Boulevard Charlotte, North Carolina 28203

Mr. Dayne H. Brown, Chief Radiation Protection Branch Division of Facility Services Department of Human Resources 701 Barbour Drive Raleigh, North Carolina 27603-2008

TABLE 4.8-1 DIESEL GENERATOR TEST SCHEDULE

NUMBER OF FAILURES IN LAST 20 VALID TESTS*	NUMBER OF FAILURES IN LAST 100 VALIDTESTS*	TEST FREQUENCY
≤ 1	<u><</u> 4	Once per 31 days
<u>></u> 2**	<u>≥</u> 5	Once per 7 days

^{*}Criteria for determining number of failures and number of valid tests shall be in accordance with Regulatory Position C.2.e of Regulatory Guide 1.108, but determined on a per diesel generator basis.

For the purposes of determining the required test frequency, the previous test failure count may be reduced to zero if a complete diesel overhaul to like-new conditions is completed, provided that the overhaul including appropriate post-maintenance operation and testing, is specifically approved by the manufacturer and if acceptable reliability has been demonstrated. The reliability criterion shall be the successful completion of 14 consecutive tests in a single series. Ten of these tests shall be in accordance with the routine surveillance requirements of Specifications 4.8.1.1.2.a.4 and 4.8.1.1.2.a.5; the remaining four tests in accordance with the 184-day requirements specified in the footnote to Specification 4.8.1.1.2.a.4 and Specification 4.8.1.1.2.a.5. If this criterion is not satisfied during the first series of tests, any alternate criterion to be used to transvalue the failure count to zero requires NRC approval.

^{**}The associated test frequency shall be maintained until seven consecutive failure free demands have been performed <u>and</u> the number of failures in the last 20 valid demands has been reduced to one.

TABLE 3.3-3 (Continued)

ENGINEERED SAFETY FEATURES ACTUATION SYSTEM INSTRUMENTATION

			IABLE	3.3 3 (Concinu	<u>-u/</u>			
	•	ENGINEERE	D SAFETY FEATUR	ES ACTUATION S	YSTEM INSTRUM	ENTATION		
FUNC	r I ONA!	L UNIT	TOTAL NO. OF CHANNELS	CHANNELS TO TRIP	MINIMUM CHANNELS OPERABLE	APPLICABLE MODES	ACTION	
7.	Auxi	liary Feedwater (continue	d)					
		Start Motor-Driven Pumps	6-3/Bus	2/Bus Either Bus	2/Bus	1, 2, 3	19	l
		Feedwater Pumps Start Motor-	2/pump	1/pump	1/pump	1, 2#	14	
8.		rculation	3	2	2	1, 2, 3	15	١
		RWST Level	. 3	-				
9.	Loss	of Power 4 kV Emergency Bus Undervoltage-Grid Degraded Voltage	3/Bus	2/Bus	2/Bus	1, 2, 3, 4	15	1
10.	10. Engineered Safety Featur Actuation System Interlo		_	2	2	1. 2. 3	20	
	a.		3	2	-	· •		
	b.	•	4	2	3			
	c.	avy Reactor Trip, P-4	2	2		-		
	d.	Steam Generator Level, P-14	3/stm gen.	2/stm gen. in any operating stm gen.	2/stm gen in each operating stm gen.	. 1, 2, 3	20	
	7. 8. 9.	7. Auxi f. g. 8. Auto Reci 9. Loss 10. Eng Act a. b. c.	FUNCTIONAL UNIT 7. Auxiliary Feedwater (continue f. Station Blackout Start Motor-Driven Pumps and Turbine-Driven Pump g. Trip of All Main Feedwater Pumps Start Motor-Driven Pumps 8. Automatic Switchover to Recirculation RWST Level 9. Loss of Power 4 kV Emergency Bus Undervoltage-Grid Degraded Voltage 10. Engineered Safety Features Actuation System Interlocks a. Pressurizer Pressure, P-11 b. Low-Low Tavg, P-12 c. Reactor Trip, P-4 d. Steam Generator	FUNCTIONAL UNIT 7. Auxiliary Feedwater (continued) f. Station Blackout Start Motor-Driven Pumps and Turbine-Driven Pump G-3/Bus g. Trip of All Main Feedwater Pumps Start Motor- Driven Pumps Start Motor- Driven Pumps 8. Automatic Switchover to Recirculation RWST Level 9. Loss of Power 4 kV Emergency Bus Undervoltage-Grid Degraded Voltage 10. Engineered Safety Features Actuation System Interlocks a. Pressurizer Pressure, P-11 b. Low-Low Tavg, c. Reactor Trip, P-4 d. Steam Generator 3 TOTAL NO. 0F CHANNELS 103/Bus 3/Bus 3/Bus	FUNCTIONAL UNIT TOTAL NO. CHANNELS TO TRIP 7. Auxiliary Feedwater (continued) f. Station Blackout Start Motor-Driven Pumps and Turbine-Driven Pump G-3/Bus G. Trip of All Main Feedwater Pumps Start Motor-Driven Pumps Start Motor-Driven Pumps Start Motor-Driven Pumps Start Motor-Driven Pumps 8. Automatic Switchover to Recirculation RWST Level 9. Loss of Power 4 kV Emergency Bus Undervoltage-Grid Degraded Voltage 10. Engineered Safety Features Actuation System Interlocks a. Pressurizer Pressure, P-11 b. Low-Low Tavg, P-12 c. Reactor Trip, P-4 d. Steam Generator Level, P-14 TOTAL NO. CHANNELS TO TRIP 2/Bus 3/Bus 2/Bus 2/Bus 2/Bus 3/Bus 2/Bus 2/Bus 3/Bus 2/Bus 2/Bus 4 4 2/Bus 4 4 2/Bus 4 4 2/Bus 4 4 4 2/Bus 4 4 4 4 4 4 4 4 4 4 4 4 4	FUNCTIONAL UNIT TOTAL NO. CHANNELS TO TRIP TOTAL NO. CHANNELS OPERABLE TOTAL NO. CHANNELS OPERABLE TOTAL NO. CHANNELS TO TRIP TOTAL NO. CHANNELS OPERABLE TO TRIP TOTAL NO. CHANNELS OPERABLE MINIMUM CHANNELS OPERABLE TO TRIP TOTAL NO. CHANNELS TO TRIP TOTAL NO. CHANNELS TO TRIP TOTAL NO. CHANNELS TO TRIP MINIMUM CHANNELS OPERABLE Auxiliary Feedwater (continued) F. Station Blackout Start Motor-Driven Pumps	FUNCTIONAL UNIT TOTAL NO. OF CHANNELS TO TRIP Auxiliary Feedwater (continued) f. Station Blackout Start Motor-Driven Pumps and Turbine-Driven Pumps Start Motor-Driven Pumps 2/pump 1/pump 1/pump 1, 2# 8. Automatic Switchover to Recirculation RWST Level 3 2 2 1, 2, 3 9. Loss of Power 4 kV Emergency Bus Undervoltage-Grid Degraded Voltage 10. Engineered Safety Features Actuation System Interlocks a. Pressurizer Pressure, P-11 b. Low-Low Tayg, P-12 4 2 3 1, 2, 3 1, 2, 3 2 2 1, 2, 3 2 2 1, 2, 3 3 3 3 3 3 3 3 3 3 3 3 3 3 3 3 3 3	FUNCTIONAL UNIT TOTAL NO. CHANNELS MINIMUM CHANNELS ACTUATION FUNCTIONAL UNIT TOTAL NO. OF CHANNELS TO TRIP Auxiliary Feedwater (continued) f. Station Blackout Start Motor-Driven Pumps and Turbine-Driven Pumps Start Motor-Driven Pumps Start