September 2, 1988

Docket Nos.: 50-369
50-370

Mr. H. B. Tucker, Vice President
Nuclear Production Department
Duke Power Company
422 South Church Street
Charlotte, North Carolina 28242

Dear Mr. Tucker:

SUBJECT: LICENSE AMENDMENT CORRECTIONS

My letter dated May 6, 1987, forwarded Amendments 71 and 52 to Facility Operating Licenses NPF-9 and NPF-17 for the McGuire Nuclear Station, Units 1 and 2, respectively. Enclosed with the amendments were Technical Specifications page changes. One of those pages contained a typographical error. Please replace page 3/4 8-8 of the May 6 transmittal with the enclosed revised page.

My letter dated July 5, 1988, forwarded Amendments 88 and 69 to the McGuire operating licenses. One of the overleaf pages forwarded with the amendments contained an error. Please replace page 3/4 3-22 of the July 5 transmittal with the enclosed revised page.

Sincerely,

Original Signed By:

Darl Hood, Project Manager
Project Directorate II-3
Division of Reactor Projects - I/II

Enclosure:
As stated

cc w/encl:
See next page

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MCGUIRE PLANT FILE					

PDII-3	PDII-3	PDII-3
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Mr. H. B. Tucker
Duke Power Company

McGuire Nuclear Station

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TABLE 4.8-1

DIESEL GENERATOR TEST SCHEDULE

NUMBER OF FAILURES IN LAST 20 VALID TESTS*	NUMBER OF FAILURES IN LAST 100 VALID TESTS*	TEST FREQUENCY
≤ 1	≤ 4	Once per 31 days
≥ 2**	≥ 5	Once per 7 days

*Criteria for determining number of failures and number of valid tests shall be in accordance with Regulatory Position C.2.e of Regulatory Guide 1.108, but determined on a per diesel generator basis.

For the purposes of determining the required test frequency, the previous test failure count may be reduced to zero if a complete diesel overhaul to like-new conditions is completed, provided that the overhaul including appropriate post-maintenance operation and testing, is specifically approved by the manufacturer and if acceptable reliability has been demonstrated. The reliability criterion shall be the successful completion of 14 consecutive tests in a single series. Ten of these tests shall be in accordance with the routine surveillance requirements of Specifications 4.8.1.1.2.a.4 and 4.8.1.1.2.a.5; the remaining four tests in accordance with the 184-day requirements specified in the footnote to Specification 4.8.1.1.2.a.4 and Specification 4.8.1.1.2.a.5. If this criterion is not satisfied during the first series of tests, any alternate criterion to be used to transvalue the failure count to zero requires NRC approval.

**The associated test frequency shall be maintained until seven consecutive failure free demands have been performed and the number of failures in the last 20 valid demands has been reduced to one.

TABLE 3.3-3 (Continued)

ENGINEERED SAFETY FEATURES ACTUATION SYSTEM INSTRUMENTATION

FUNCTIONAL UNIT	TOTAL NO. OF CHANNELS	CHANNELS TO TRIP	MINIMUM CHANNELS OPERABLE	APPLICABLE MODES	ACTION
7. Auxiliary Feedwater (continued)					
f. Station Blackout Start Motor-Driven Pumps and Turbine-Driven Pump	6-3/Bus	2/Bus Either Bus	2/Bus	1, 2, 3	19
g. Trip of All Main Feedwater Pumps Start Motor-Driven Pumps	2/pump	1/pump	1/pump	1, 2#	14
8. Automatic Switchover to Recirculation RWST Level	3	2	2	1, 2, 3	15
9. Loss of Power 4 kV Emergency Bus Undervoltage-Grid Degraded Voltage	3/Bus	2/Bus	2/Bus	1, 2, 3, 4	15
10. Engineered Safety Features Actuation System Interlocks					
a. Pressurizer Pressure, P-11	3	2	2	1, 2, 3	20
b. Low-Low T_{avg}, P-12	4	2	3	1, 2, 3	20
c. Reactor Trip, P-4	2	2	2	1, 2, 3	22
d. Steam Generator Level, P-14	3/stm gen.	2/stm gen. in any operating stm gen.	2/stm gen. in each operating stm gen.	1, 2, 3	20