

December 19, 2001

Mr. J. A. Price  
Vice President - Nuclear Technical Services - Millstone  
Dominion Nuclear Connecticut, Inc.  
c/o Mr. David A. Smith  
Rope Ferry Road  
Waterford, CT 06385

SUBJECT: MILLSTONE NUCLEAR POWER STATION, UNIT NO. 2 - ISSUANCE OF  
AMENDMENT RE: NUMBER REVISION TO LIST OF DOCUMENTS IN  
TECHNICAL SPECIFICATION 6.9.1.8b (TAC NO. MA1780)

Dear Mr. Price:

The Commission has issued the enclosed Amendment No. 260 to Facility Operating License No. DPR-65 for the Millstone Nuclear Power Station, Unit No. 2 (MP2), in response to your application dated April 11, 2001, as supplemented June 14, 2001.

The amendment revises the list of documents that describe the analytical methods used to determine the core operating limits specified in Technical Specification 6.9.1.8b. The revision consists of updating the list of documents to include the latest NRC-approved methodologies, along with deleting the revision numbers and dates of all the documents in the list. The information needed to identify a specific report, i.e, revision number or date, will be included in the MP2 Core Operating Limits Report, which is part of the MP2 Technical Requirements Manual.

A copy of the related Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

**/RA/**

John T. Harrison, Project Manager, Section 2  
Project Directorate I  
Division of Licensing Project Management  
Office of Nuclear Reactor Regulation

Docket No. 50-336

Enclosures: 1. Amendment No. 260 to DPR-65  
2. Safety Evaluation

cc w/encls: See next page

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DOMINION NUCLEAR CONNECTICUT, INC.

DOCKET NO. 50-336

MILLSTONE NUCLEAR POWER STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 260  
License No. DPR-65

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by the applicant dated April 11, 2001, as supplemented June 14, 2001, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-65 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 260 are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of issuance, and shall be implemented prior to restart from refueling outage 14 which is currently scheduled in early February of 2002.

FOR THE NUCLEAR REGULATORY COMMISSION

**/RA/**

James W. Clifford, Chief, Section 2  
Project Directorate I  
Division of Licensing Project Management  
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical  
Specifications

Date of Issuance: December 19, 2001

ATTACHMENT TO LICENSE AMENDMENT NO. 260

FACILITY OPERATING LICENSE NO. DPR-65

DOCKET NO. 50-336

Replace the following pages of the Appendix A, Technical Specifications, with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

Remove

6-18a  
6-19

Insert

6-18a  
6-19

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
RELATED TO AMENDMENT NO. 260  
TO FACILITY OPERATING LICENSE NO. DPR-65  
DOMINION NUCLEAR CONNECTICUT, INC.  
MILLSTONE NUCLEAR POWER STATION, UNIT NO. 2  
DOCKET NO. 50-336

## 1.0 INTRODUCTION

By letter dated April 11, 2001, as supplemented June 14, 2001, the Dominion Nuclear Connecticut, Inc., (the licensee), submitted a request for changes to the Millstone Nuclear Power Station, Unit No. 2 (MP2) Technical Specifications (TSs). The requested changes would update the list of documents describing the analytical methods used to determine the core operating limits specified in TS 6.9.1.8b. Specifically, these changes would update the documents describing the analytical methods used in the current Small Break Loss of Coolant Accident analysis (SBLOCA), setpoint methodology, and non-LOCA methodology. In addition, the revision number and the date of documents listed in TS 6.9.1.8b would be deleted, in a manner consistent with that approved by the Nuclear Regulatory Commission (NRC) in Standard Technical Specification Change Traveler TSTF-363.

## 2.0 BACKGROUND

In an effort to avoid TS changes for every fuel reload cycle that results in the changes of the cycle-specific parameter limits, licensees have relocated the cycle-specific core operating parameters from the TSs to the Core Operating Limits Report (COLR). This is done in accordance with the guidelines of NRC Generic Letter (GL) 88-16 (Reference 1). GL 88-16 also requires the licensees to identify, in the TSs "Reporting Requirements," the previously approved analytical methods used to determine the core operating limits by identifying the topical report number, title, and date (or identify the staff's safety evaluation report for a plant-specific methodology by NRC letter and date).

In December 1999 (Reference 2), the NRC staff accepted a method proposed by Siemens Power Corporation of referencing approved topical reports. The proposed method would allow licensees to use current topical reports to support limits in the COLR without having to submit an amendment request for the facility operating license each time a revision to the topical report is approved by the NRC. This method would allow the references to approved topical reports in the TSs to be cited using the report number and title. The citation in the COLR would include specific information for each of the TS references to topical reports used to prepare the COLR



(i.e., report number, title, revision, date, and any supplements). This method of referencing was subsequently accepted generically by TSTF-363 (Reference 3).

### 3.0 EVALUATION

The licensee proposed the following four changes to TS 6.9.1.8b, Core Operating Limits Report:

- Revise references of all topical reports in TS 6.9.1.8b to identify the latest version by listing the topical report number and title. The specific information identifying each topical report, such as revision number, date, and supplements would be listed in the cycle-specific COLR.
- Revise TS 6.9.1.8b Item 6 to reference most recent small break loss of coolant accident analysis. The original topical reports, listed in Items 6 and 7 (XN-NF-82-49(P)(A) and XN-NF-82-49(P)(A) Supplement 1) are deleted and replaced by Topical Report EMF-2328(P)(A), "PWR Small Break LOCA Evaluation Model S-RELAP5 Based." Items 8 through 15 are renumbered.
- Revise TS 6.9.1.8b Item 15 (renumbered Item 14) to reference most recent setpoint methodology for Combustion Engineering type reactors. The original topical report, XF-NF-507(P)(A) Supplements 1 and 2, is replaced by Topical Report EMF-1961(P)(A), "Statistical Setpoint/Transient Methodology for Combustion Engineering Type Reactors."
- Revise TS 6.9.1.8b by adding Item 15, Topical Report EMF-2310(P)(A), "SRP Chapter 15 Non-LOCA Methodology for Pressurized Water Reactors." This new document was approved by the NRC by letter dated May 11, 2001 (Reference 4).

Since every topical report identified in TS 6.9.1.8b is approved by the NRC, as indicated by the "(A)" in the report number, there is an assurance that only the approved versions of the referenced topical reports will be used for the determination of the core operating limits. As previously stated, the COLR, which is part of the MP2 Technical Requirements Manual, will provide specific information identifying the particular approved topical reports used to determine the core operating limits for the particular cycle.

In addition, the use of the revised methodologies (References 5, 6, and 7) constitutes an improvement over the previous methodologies and is in accordance with the guidance contained in GL 88-16 and the NRC's Safety Evaluation Report for TSTF-363. Consequently, the proposed changes will have no adverse effect on plant safety; and, therefore, the staff finds the proposed changes to TS 6.9.1.8b acceptable.

### 4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Connecticut State official was notified of the proposed issuance of the amendment. The State official had no comments.

## 5.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (66 FR 38760). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

## 6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

## 7.0 REFERENCES

1. U.S. Nuclear Regulatory Commission Generic Communication, "Removal of Cycle-Specific Parameter Limits From Technical Specifications (Generic Letter 88-16)," October 4, 1988.
2. Letter from S. A. Richards (USNRC) to J. F. Mallay, Siemens Power Corporation, "Acceptance for Siemens References to Approved Topical Reports in Technical Specifications (TAC No. MA6492)," December 15, 1999.
3. Industry/TSTF Standard Technical Specification Change Traveler TSTF-363, "Revise Topical Report References in ITS 5.6.5, COLR," April 2000.
4. Letter from S. A. Richards, USNRC, to J. F. Mallay, Siemens Power Corporation, "Acceptance For Referencing of Licensing Topical Report EMF-2310(P), Revision 0, 'SRP Chapter 15 Non-LOCA Methodology For Pressurized Water Reactors' (TAC NO. MA7192)," May 11, 2001.
5. EMF-2328(P)(A), "PWR Small Break LOCA Evaluation Model S-RELAP5 Based," Framatome ANP, March 2001.
6. EMF-1961(P)(A), Revision 0, "Statistical Setpoint/Transient Methodology for Combustion Engineering Type Reactors," Siemens Power Corporation, July 2000.

7. EMF-2310(P)(A), Revision 0, "SRP Chapter 15 Non-LOCA Methodology for Pressurized Water Reactors," Framatome ANP, May 2001.

Principal Contributors: K. Kavanagh  
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Date: December 19, 2001