

July 5, 1988

Docket Nos.: 50-369  
and 50-370

Mr. H. B. Tucker, Vice President  
Nuclear Production Department  
Duke Power Company  
422 South Church Street  
Charlotte, North Carolina 28242

Dear Mr. Tucker:

SUBJECT: ISSUANCE OF AMENDMENT NO. 88 TO FACILITY OPERATING LICENSE NPF-9 AND  
AMENDMENT NO. 69 TO FACILITY OPERATING LICENSE NPF-17 - MCGUIRE  
NUCLEAR STATION, UNITS 1 AND 2 (TACS 64744/64745)

The Nuclear Regulatory Commission has issued the enclosed Amendment No. 88 to Facility Operating License NPF-9 and Amendment No. 69 to Facility Operating License NPF-17 for the McGuire Nuclear Station, Units 1 and 2. These amendments consist of changes to the Technical Specifications (TS) in response to your application dated February 17, 1987, as revised June 3, 1988. Other changes requested in the February 17, 1987, letter will be addressed in future correspondence.

The amendments change the Technical Specifications by updating the index and making other purely administrator changes. The amendments are effective as of their date of issuance.

A copy of the related safety evaluation supporting Amendment No. 88 to Facility Operating License NPF-9 and Amendment No. 69 to Facility Operating License NPF-17 is enclosed.

Notice of issuance of amendments will be included in the Commission's next bi-weekly Federal Register notice.

Sincerely,

Original signed by:

Darl Hood, Project Manager  
Project Directorate II-3  
Division of Reactor Projects I/II

Enclosures:

1. Amendment No. 88 to NPF-9
2. Amendment No. 69 to NPF-17
3. Safety Evaluation

cc w/enclosures:  
See next page

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McGuire Nuclear Station

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DATED: July 5, 1988

AMENDMENT NO. 88 TO FACILITY OPERATING LICENSE NPF-9 - McGuire Nuclear Station, Unit 1  
AMENDMENT NO. 69 TO FACILITY OPERATING LICENSE NPF-17 - McGuire Nuclear Station, Unit 2

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B. Grimes 9A-2

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GPA/PA 17F-2

ARM/LFMB AR-2015

E. Butcher 11D-3

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

DUKE POWER COMPANY

DOCKET NO. 50-369

McGUIRE NUCLEAR STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 88  
License No. NPF-9

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment to the McGuire Nuclear Station, Unit 1 (the facility) Facility Operating License No. NPF-9 filed by the Duke Power Company (the licensee) dated February 17, 1987, as revised June 3, 1988, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations as set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, as amended, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is hereby amended by page changes to the Technical Specifications as indicated in the attachments to this license amendment, and Paragraph 2.C.(2) of Facility Operating License No. NPF-9 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 88, are hereby incorporated into the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Original signed by:

David B. Matthews, Director  
Project Directorate II-3  
Division of Reactor Projects-I/II

Attachment:  
Technical Specification  
Changes

Date of Issuance: July 5, 1988

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6/29/88



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

DUKE POWER COMPANY

DOCKET NO. 50-370

McGUIRE NUCLEAR STATION, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 69  
License No. NPF-17

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment to the McGuire Nuclear Station, Unit 2 (the facility) Facility Operating License No. NPF-17 filed by the Duke Power Company (the licensee) dated February 17, 1987, as revised June 3, 1988, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations as set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, as amended, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is hereby amended by page changes to the Technical Specifications as indicated in the attachments to this license amendment, and Paragraph 2.C.(2) of Facility Operating License No. NPF-17 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 69, are hereby incorporated into the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Original signed by:

David B. Matthews, Director  
Project Directorate II-3  
Division of Reactor Projects-I/II

Attachment:  
Technical Specification  
Changes

Date of Issuance: July 5, 1988

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6/21/88

D:PDII-3  
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6/29/88

ATTACHMENT TO LICENSE AMENDMENT NO.

FACILITY OPERATING LICENSE NO. NPF-9

DOCKET NO. 50-369

AND

TO LICENSE AMENDMENT NO.

FACILITY OPERATING LICENSE NO. NPF-17

DOCKET NO. 50-370

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the areas of change. The corresponding overleaf pages are also provided to maintain document completeness.

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## REACTIVITY CONTROL SYSTEMS

### CONTROL ROD INSERTION LIMITS

#### LIMITING CONDITION FOR OPERATION

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3.1.3.6 The control banks shall be limited in physical insertion as shown in Figure 3.1-1.

APPLICABILITY: MODES 1\* and 2\*#.

ACTION:

With the control banks inserted beyond the above insertion limits, except for surveillance testing pursuant to Specification 4.1.3.1.2:

- a. Restore the control banks to within the limits within 2 hours, or
- b. Reduce THERMAL POWER within 2 hours to less than or equal to that fraction of RATED THERMAL POWER which is allowed by the bank position using the above figures, or
- c. Be in at least HOT STANDBY within 6 hours.

#### SURVEILLANCE REQUIREMENTS

---

4.1.3.6 The position of each control bank shall be determined to be within the insertion limits at least once per 12 hours except during time intervals when the Rod Insertion Limit Monitor is inoperable, then verify the individual rod positions at least once per 4 hours.

\*See Special Test Exceptions 3.10.2 and 3.10.3.

#With  $K_{eff}$  greater than or equal to 1.0.

TABLE 4.3-1 (Continued)

TABLE NOTATION

- \* - With the Reactor Trip System breakers closed and the Control Rod Drive System capable of rod withdrawal.
- ## - Below P-6 (Intermediate Range Neutron Flux Interlock) Setpoint.
- ### - Below P-10 (Low Setpoint Power Range Neutron Flux Interlock) Setpoint.
- (1) - If not performed in previous 7 days.
- (2) - Comparison of calorimetric to excore power indication above 15% of RATED THERMAL POWER. Adjust excore channel gains consistent with calorimetric power if absolute difference is greater than 2%. The provisions of Specification 4.0.4 are not applicable for entry into MODE 2 or 1.
- (3) - Single point comparison of incore to excore axial flux difference above 15% of RATED THERMAL POWER. Recalibrate if the absolute difference is greater than or equal to 3%. The provisions of Specification 4.0.4 are not applicable for entry into MODE 2 or 1.
- (4) - Neutron detectors may be excluded from CHANNEL CALIBRATION.
- (5) - Detector plateau curves shall be obtained, evaluated, and compared to manufacturer's data. For the Intermediate Range and Power Range Neutron Flux channels the provisions of Specification 4.0.4 are not applicable for entry into MODE 2 or 1.
- (6) - Incore - Excore Calibration, above 75% of RATED THERMAL POWER. The provisions of Specification 4.0.4 are not applicable for entry into MODE 2 or 1.
- (7) - Each train shall be tested at least every 62 days on a STAGGERED TEST BASIS.
- (8) - With power greater than or equal to the interlock Setpoint the required operational test shall consist of verifying that the interlock is in the required state by observing the permissive annunciator window.
- (9) - Monthly surveillance in MODES 3\*, 4\* and 5\* shall also include verification that permissives P-6 and P-10 are in their required state for existing plant conditions by observation of the permissive annunciator window. Monthly surveillance shall include verification of the High Flux at Shutdown Alarm Setpoint of less than or equal to five times background.
- (10) - Setpoint verification is not required.

TABLE 3.3-3 (Continued)

ENGINEERED SAFETY FEATURES ACTUATION SYSTEM INSTRUMENTATION

<u>FUNCTIONAL UNIT</u>	<u>TOTAL NO. OF CHANNELS</u>	<u>CHANNELS TO TRIP</u>	<u>MINIMUM CHANNELS OPERABLE</u>	<u>APPLICABLE MODES</u>	<u>ACTION</u>
7. Auxiliary Feedwater					
a. Manual Initiation	2	1	2	1, 2, 3	22
b. Automatic Actuation Logic and Actuation Relays	2	1	2	1, 2, 3	21
c. Stm. Gen. Water Level- Low-Low					
1) Start Motor- Driven Pumps	4/stm. gen.	2/stm. gen. in any opera- ting stm gen.	3/stm. gen. in each operating stm. gen.	1, 2, 3	19
2) Start Turbine- Driven Pump	4/stm. gen.	2/stm. gen. in any 2 operating stm. gen.	3/stm. gen in each operating stm. gen	1, 2, 3	19
d. Auxiliary Feedwater Suction Pressure - Low (Suction Supply Automatic Realignment)	2/pump	2/pump	2/pump	1, 2, 3	24
e. Safety Injection Start Motor-Driven Pumps					
See Item 1. above for all Safety Injection initiating functions and requirements					

McGUIRE - UNITS 1 AND 2

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Amendment No. 88 (Unit 1)  
Amendment No. 69 (Unit 2)

TABLE 3.3-3 (Continued)

ENGINEERED SAFETY FEATURES ACTUATION SYSTEM INSTRUMENTATION

<u>FUNCTIONAL UNIT</u>	<u>TOTAL NO. OF CHANNELS</u>	<u>CHANNELS TO TRIP</u>	<u>MINIMUM CHANNELS OPERABLE</u>	<u>APPLICABLE MODES</u>	<u>ACTION</u>
7. Auxiliary Feedwater (continued)					
f. Station Blackout Start Motor-Driven Pumps and Turbine-Driven Pump	6-3/Bus	2/Bus Either Bus	2/Bus	1, 2, 3	19*
g. Trip of All Main Feedwater Pumps Start Motor- Driven Pumps	2/pump	1/pump	1/pump	1, 2 <sup>#</sup>	14
8. Automatic Switchover to Recirculation					
RWST Level	3	2	2	1, 2, 3	15*
9. Loss of Power					
4 kV Emergency Bus Undervoltage-Grid Degraded Voltage	3/Bus	2/Bus	2/Bus	1, 2, 3, 4	15*
10. Engineered Safety Features Actuation System Interlocks					
a. Pressurizer Pressure, P-11	3	2	2	1, 2, 3	20
b. Low-Low T <sub>avg</sub> , P-12	4	2	3	1, 2, 3	20
c. Reactor Trip, P-4	2	2	2	1, 2, 3	22
d. Steam Generator Level, P-14	3/stm gen.	2/stm gen. in any operating stm gen.	2/stm gen. in each operating stm gen.	1, 2, 3	20

TABLE 4.3-2 (Continued)

ENGINEERED SAFETY FEATURES ACTUATION SYSTEM INSTRUMENTATION  
SURVEILLANCE REQUIREMENTS

<u>FUNCTIONAL UNIT</u>	<u>CHANNEL CHECK</u>	<u>CHANNEL CALIBRATION</u>	<u>ANALOG CHANNEL OPERATIONAL TEST</u>	<u>TRIP ACTUATING DEVICE OPERATIONAL TEST</u>	<u>ACTUATION LOGIC TEST</u>	<u>MASTER RELAY TEST</u>	<u>SLAVE RELAY TEST</u>	<u>MODES FOR WHICH SURVEILLANCE IS REQUIRED</u>
10. Engineered Safety Features Actuation System Interlocks								
a. Pressurizer Pressure, P-11	N.A.	R	M	N.A.	N.A.	N.A.	N.A.	1, 2, 3
b. Low, Low $T_{avg}$ , P-12	N.A.	N.A.	N.A.	R	N.A.	N.A.	N.A.	1, 2, 3
c. Reactor Trip, P-4	N.A.	R	M	N.A.	N.A.	N.A.	N.A.	1, 2, 3
d. Steam Generator Level, P-14	S	R	M	N.A.	M(1)	M(1)	Q	1, 2, 3

MCGUIRE - UNITS 1 AND 2

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Amendment No. 88 (Unit 1)  
Amendment No. 69 (Unit 2)



TABLE 3.3-10  
ACCIDENT MONITORING INSTRUMENTATION

<u>INSTRUMENT</u>	<u>REQUIRED NO. OF CHANNELS</u>	<u>MINIMUM CHANNELS OPERABLE</u>
1. Containment Pressure	2	1
2. Reactor Coolant Temperature - T <sub>HOT</sub> and T <sub>COLD</sub> (Wide Range)	2	1
3. Reactor Coolant Pressure - Wide Range	2	1
4. Pressurizer Water Level	2	1
5. Steam Line Pressure	2/steam generator	1/steam generator
6. Steam Generator Water Level - Narrow Range	2/steam generator	1/steam generator
7. Refueling Water Storage Tank Water Level	2	1
8. Auxiliary Feeder Flow Rate	2/steam generator	1/steam generator
9. Reactor Coolant System Subcooling Margin Monitor	2***	1
10. PORV Position Indicator*	2/valve	1/valve
11. PORV Block Valve Position Indicator**	1/valve	1/valve
12. Safety Valve Position Indicator	2/valve	1/valve
13. Containment Water Level (Wide Range)	2	1
14. In Core Thermocouples	4/core quadrant	2/core quadrant
15. Unit Vent - High Range Noble Gas Monitor (High-High Range - EMF-36)	1	1
16. Steam Relief - High Range Monitor (Unit 1 - EMF-24, 25, 26, 27) (Unit 2 - EMF-10, 11, 12, 13)	1/steam line	1/steam line
17. Containment Atmosphere - High Range Monitor (11 F-51a or 51b)	1	1
18. Reactor Vessel Level Instrumentation****		
a. Dynamic Head (D/P) Range	2	1
b. Lower Range	2	1

\*Not applicable if the associated block valve is in the closed position.

\*\*Not applicable if the associated block valve is in the closed position and power is removed.

\*\*\*Only one channel per unit is required until the end of the first refueling outage following 1/86 for each unit.

\*\*\*\*Not applicable until the beginning of Cycle 4 for Unit 1 and Cycle 3 for Unit 2.

**TABLE 4.3-7**  
**ACCIDENT MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS**

	INSTRUMENT	CHANNEL	CHANNEL
		CHECK	CALIBRATION
McGUIRE - UNITS 1 AND 2  3/4 3-57  Amendment No. 88 (Unit 1) Amendment No. 69 (Unit 2)	1. Containment Pressure	M	R
	2. Reactor Coolant Temperature - T <sub>HOT</sub> and T <sub>COLD</sub> (Wide Range)	M	R
	3. Reactor Coolant Pressure - Wide Range	M	R
	4. Pressurizer Water Level	M	R
	5. Steam Line Pressure	M	R
	6. Steam Generator Water Level - Narrow Range	M	R
	7. Refueling Water Storage Tank Water Level	M	R
	8. Auxiliary Feedwater Flow Rate	M	R
	9. Reactor Coolant System Subcooling Margin Monitor	M	R
	10. PORV Position Indicator	M	R
	11. PORV Block Valve Position Indicator	M	R
	12. Safety Valve Position Indicator	M	R
	13. Containment Water Level (Wide Range)	M	R
	14. In Core Thermocouples	M	R
	15. Unit Vent - High Range Noble Gas Monitor (High-High Range - EMF-36)	M	R
	16. Steam Relief - High Range Monitor (Unit 1 - EMF-24, 25, 26, 27) (Unit 2 - EMF-10, 11, 12, 13)	M	R
	17. Containment Atmosphere - High Range Monitor (EMF-51a or 51b)	M	R
	18. Reactor Vessel Level Instrumentation		
	a. Dynamic Head (D/P) Range	M	R
	b. Lower Range	M	R

TABLE 3.6-2

CONTAINMENT ISOLATION VALVES

		<u>TABLE 3.0 2</u>		
		<u>CONTAINMENT ISOLATION VALVES</u>		
	<u>VALVE NUMBER</u>	<u>FUNCTION</u>	<u>MAXIMUM ISOLATION TIME (SEC)</u>	
McGUIRE - UNITS 1 AND 2	1. Phase "A" Isolation			
	BB-1B#	Steam Generator A Blowdown Containment Outside Isolation	<10	
	BB-2B#	Steam Generator B Blowdown Containment Outside Isolation	<10	
	BB-3B#	Steam Generator C Blowdown Containment Outside Isolation	<10	
	BB-4B#	Steam Generator D Blowdown Containment Outside Isolation	<10	
	BB-5A#	Steam Generator A Blowdown Containment Inside Isolation	<10	
	BB-6A#	Steam Generator B Blowdown Containment Inside Isolation	<10	
	BB-7A#	Steam Generator C Blowdown Containment Inside Isolation	<10	
	BB-8A#	Steam Generator D Blowdown Containment Inside Isolation	<10	
	3/4 6-24	CF-26AB#	Steam Generator D Feedwater Containment Isolation	<5
CF-28AB#		Steam Generator C Feedwater Containment Isolation	<5	
CF-30AB#		Steam Generator B Feedwater Containment Isolation	<5	
CF-35AB#		Steam Generator A Feedwater Containment Isolation	<5	
CF-126B		Steam Generator A Main Feedwater to Auxiliary Feedwater Nozzle Isolation	<10	
CF-127B		Steam Generator B Main Feedwater to Auxiliary Feedwater Nozzle Isolation	<10	
CF-128B		Steam Generator C Main Feedwater to Auxiliary Feedwater Nozzle Isolation	<10	
CF-129B		Steam Generator D Main Feedwater to Auxiliary Feedwater Nozzle Isolation	<10	
CF-134A		Steam Generator A Feedwater Containment Isolation Bypass	<10	
CF-135A		Steam Generator B Feedwater Containment Isolation Bypass	<10	
Amendment No. 88	CF-136A	Steam Generator C Feedwater Containment Isolation Bypass	<10	
	CF-137A	Steam Generator D Feedwater Containment Isolation Bypass	<10	
	CF-151B	Auxiliary Nozzle Temper SG A	<10	

McGUIRE - UNITS 1 AND 2

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**Amendment No. 88 (Unit 1)**  
**Amendment No. 69 (Unit 2)**

<u>VALVE NUMBER</u>	<u>FUNCTION</u>	<u>MAXIMUM ISOLATION TIME (SEC)</u>
1. Phase "A" Isolation (continued)		
CF-153B	Auxiliary Nozzle Temper SG B	<10
CF-155B	Auxiliary Nozzle Temper SG C	<10
CF-157B	Auxiliary Nozzle Temper SG D	<10
KC-305B#	Excess Letdown Hx Supply Pent. Isolation Outside	<30
KC-315B#	Excess Letdown Hx Ret. Hdr. Pent. Isolation Outside	<30
KC-320A	NCDT Hx Supply Hdr. Pent. Isolation Outside	<15
KC-332B	NCDT Hx Supply Hdr. Pent. Isolation Inside	<15
KC-333A	NCDT Hx Return Hdr. Pent. Isolation Outside	<15
KC-429B	RB Drain Header Inside Containment Isolation	<15
KC-430A	RB Drain Header Outside Containment Isolation	<15
NB-260B	Reactor Makeup Water Tank to NV System	<15
NC-53B	Nitrogen to Pressurizer Relief Tank Containment Isolation Outside	<10
NC-54A	Nitrogen to Pressurizer Relief Tank Containment Isolation Inside	<10
NC-56B	PRT Makeup	<10
NC-195B	NC Pump Motor Oil Containment Isolation Outside	<15
NC-196A	NC Pump Motor Oil Containment Isolation Inside	<15
NF-228A	Air Handling Units Glycol Supply Containment Isolation Outside	<15
NF-233B	Air Handling Units Glycol Supply Containment Isolation Inside	<15
NF-234A	Air Handling Units Glycol Supply Containment Isolation Outside	<15
NI-47A	Accumulator Nitrogen Supply Outside Containment Isolation	<15
NI-95A	Test HDR Inside Containment Isolation	<10



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
RELATED TO AMENDMENT NO. 88 TO FACILITY OPERATING LICENSE NPF-9  
AND AMENDMENT NO. 69 TO FACILITY OPERATING LICENSE NPF-17

DUKE POWER COMPANY

DOCKET NOS. 50-369 AND 50-370

MCGUIRE NUCLEAR STATION, UNITS 1 AND 2

INTRODUCTION

By letter dated February 17, 1987, and revised June 3, 1988, Duke Power Company (the licensee) proposed amendments to make several changes to the McGuire Technical Specifications (TS). The changes, in part, are:

- (1) The TS Index is updated to achieve consistency with changes authorized by the Commission by prior amendments.
- (2) Reference to Figure 3.1-2, which had been left blank pending NRC approval of three-loop operation, is deleted from TS 3.1.3.6 for consistency with TS 3.4.1.1 which prohibits part-loop operation, and because (as noted during prior Amendments 65 and 46) no NRC approval of three-loop operation is pending.
- (3) In the Table Notation for TS Table 4.3.1, Item (9), the term "Boron Dilution Alarm" is changed to "High Flux at Shutdown Alarm." This is a change in nomenclature only, to provide for consistency with plant terminology.
- (4) In Table 3.3-3, Item 7.e, the left-hand margin of the statement "See Item 1 above for all safety injection initiating functions and requirements" is shifted to the right in order to clarify that the statement applies only to Item 7.e and not to the entire page on which it appears. This change therefore corrects an error in the location of the statement.
- (5) The positions of Items 10.b and 10.c in TS Surveillance Table 4.3-2 are exchanged for consistency with their order in the corresponding LCO Tables 3.3-3 and 3.3-4.
- (6) Under Item 18 "Reactor Vessel Level Instrumentation" of both TS Tables 3.3-10 and 4.3-7, the term "Wide Range" is renamed "Dynamic Head (D/P) Range," and the term "Narrow Range" is renamed "Lower Range." These are changes in nomenclature only and provide for consistency with plant terminology. Also, two obsolete footnotes for Table 3.3-10 (which applied only until the end of the first refueling outage following 1/86 for each unit, or until the beginning of Cycle 4 for Unit 1 and Cycle 3 for Unit 2) are deleted.

(7) Erroneous valve train designations in TS Table 3.6-2 resulting from typographical error (i.e., designations for valve CF-153 rather than Valve CF-135) associated with previous Amendments 63 and 44 are corrected.

Because the June 3, 1988, submittal clarified certain aspects of the original request, the substance of the changes noticed in the Federal Register and the proposed no significant hazards determination were not affected.

Other changes in the licensee's submittal will be handled separately.

#### EVALUATION

The staff has reviewed the licensee's proposed changes, noted above, and finds that they are purely administrative. Each of these changes is made to achieve consistency, correct an error or change nomenclature. These changes have no adverse impact upon safety and are, therefore, acceptable.

#### ENVIRONMENTAL CONSIDERATION

These amendments relate to changes in recordkeeping, reporting or administrative procedures or requirements. Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(10). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of these amendments.

#### CONCLUSION

The Commission made a proposed determination that the amendments involve no significant hazards consideration which was published in the Federal Register (52 FR 9566) on March 25, 1987. The Commission consulted with the state of North Carolina. No public comments were received, and the state of North Carolina did not have any comments.

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: D. Hood, PD#II-3/DRP-I/II

Dated:



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

July 5, 1988

Docket No. 50-369  
50-370

MEMORANDUM FOR: Sholly Coordinator

FROM: Darl S. Hood, Project Manager  
Project Directorate II-3  
Division of Reactor Projects-I/II

SUBJECT: REQUEST FOR PUBLICATION IN BIWEEKLY FR NOTICE - NOTICE  
OF ISSUANCE OF AMENDMENT TO FACILITY OPERATING LICENSE  
(TACs 64744/64745)

Duke Power Company, Docket Nos. 50-369 and 50-370, McGuire Nuclear Station,  
Units 1 and 2, Mecklenburg County, North Carolina

Date of application for amendments: February 17, 1987, as revised June 3,  
1988

Brief description of amendments: The amendments changed the Technical  
Specifications by updating the index and making other purely administrative  
changes.

Date of issuance: July 5, 1988

Effective date: July 5, 1988

Amendment Nos.: 88 and 69

Facility Operating License Nos. NPF-9 and NPF-17: Amendments revised the  
Technical Specifications.

Date of initial notice in FEDERAL REGISTER: March 25, 1987 (52 FR 9566) The  
supplemental letter did not change the initial proposed no significant hazards  
consideration determination.

The Commission's related evaluation of the amendments is contained in a  
Safety Evaluation dated July 5, 1988.

No significant hazards consideration comments received: No.

Sholly Coordinator

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Original signed by:

Darl S. Hood, Project Manager  
Project Directorate II-3  
Division of Reactor Projects-I/II

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