

Docket Nos.: 50-369
and 50-370

Mr. H. B. Tucker, Vice President
Nuclear Production Department
Duke Power Company
422 South Church Street
Charlotte, North Carolina 28242

Dear Mr. Tucker:

Subject: Issuance of Amendment No. to Facility Operating License NPF-9
and Amendment No. to Facility Operating License NPF-17 - McGuire
Nuclear Station, Units 1 and 2 (TACS 65906/65907)

The Nuclear Regulatory Commission has issued the enclosed Amendment No. to Facility Operating License NPF-9 and Amendment No. to Facility Operating License NPF-17 for the McGuire Nuclear Station, Units 1 and 2. These amendments consist of changes to the Technical Specifications in response to your application dated July 31, 1987.

The amendments change Technical Specification 3/4.7.4 "Nuclear Service Water System" and its Bases to reflect that portions of the system are shared between the two McGuire units, but that the system is not shared in its entirety. Minor relabeling of valves on page 3/4 7-11a was also done in accordance with my telephone discussion with Mr. Jerry Day of your staff. The amendments are effective as of their date of issuance.

A copy of the related safety evaluation supporting Amendment No. to Facility Operating License NPF-9 and Amendment No. to Facility Operating License NPF-17 is enclosed.

Notice of issuance of amendments will be included in the Commission's next bi-weekly Federal Register notice.

Sincerely,

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Darl Hood, Project Manager
Project Directorate II-3
Division of Reactor Projects I/II

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Enclosures:

- 1. Amendment No. to NPF-9
- 2. Amendment No. to NPF-17
- 3. Safety Evaluation

cc w/enclosures: See next page

DISTRIBUTION:
See attached page

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McGuire Nuclear Station

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

DUKE POWER COMPANY

DOCKET NO. 50-369

McGUIRE NUCLEAR STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 78
License No. NPF-9

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment to the McGuire Nuclear Station, Unit 1 (the facility) Facility Operating License No. NPF-9 filed by the Duke Power Company (the licensee) dated July 31, 1987, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations as set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, as amended, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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2. Accordingly, the license is hereby amended by page changes to the Technical Specifications as indicated in the attachments to this license amendment, and Paragraph 2.C.(2) of Facility Operating License No. NPF-9 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 78, are hereby incorporated into the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

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Darl S. Hood, Acting Director
Project Directorate II-3
Division of Reactor Projects-I/II

Attachment:
Technical Specification
Changes

Date of Issuance: January 4, 1988

PD#II-3/DRP-I/II
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12/17/87

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PD#II-3/DRP-I/II
DHood
12/17/87

OGC-Bethesda
[Signature]
12/26/87

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PD#II-3/DRP-I/II
DHood, Acting PD
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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

DUKE POWER COMPANY

DOCKET NO. 50-370

McGUIRE NUCLEAR STATION, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 59
License No. NPF-17

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment to the McGuire Nuclear Station, Unit 2 (the facility) Facility Operating License No. NPF-17 filed by the Duke Power Company (the licensee) dated July 31, 1987, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations as set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, as amended, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is hereby amended by page changes to the Technical Specifications as indicated in the attachments to this license amendment, and Paragraph 2.C.(2) of Facility Operating License No. NPF-9 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 59, are hereby incorporated into the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

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Darl S. Hood, Acting Director
Project Directorate II-3
Division of Reactor Projects-I/II

Attachment:
Technical Specification
Changes

Date of Issuance: January 4, 1988

PD#II-3/DRP-I/II
MRood/mac
12/17/87

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PD#II-3/DRP-I/II
DHood
12/17/87

OGC-Bethesda
[Signature]
12/26/87

^{DSH}
PD#II-3/DRP-I/II
DHood, Acting PD
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1/4/88

ATTACHMENT TO LICENSE AMENDMENT NO. 78

FACILITY OPERATING LICENSE NO. NPF-9

DOCKET NO. 50-369

AND

TO LICENSE AMENDMENT NO. 59

FACILITY OPERATING LICENSE NO. NPF-17

DOCKET NO. 50-370

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the areas of change. The corresponding overleaf pages are also provided to maintain document completeness.

<u>Amended</u> <u>Page</u>	<u>Overleaf</u> <u>Page</u>
3/4 7-11	
3/4 7-11a (new)	3/4 7-12
B 3/4 7-3	
B 3/4 7-3a	
B 3/4 7-3b (new)	B 3/4 7-4

PLANT SYSTEMS

3/4.7.4 NUCLEAR SERVICE WATER SYSTEM

LIMITING CONDITION FOR OPERATION

3.7.4 At least two independent nuclear service water loops shall be OPERABLE.

APPLICABILITY: MODES 1, 2, 3, and 4.

ACTION:

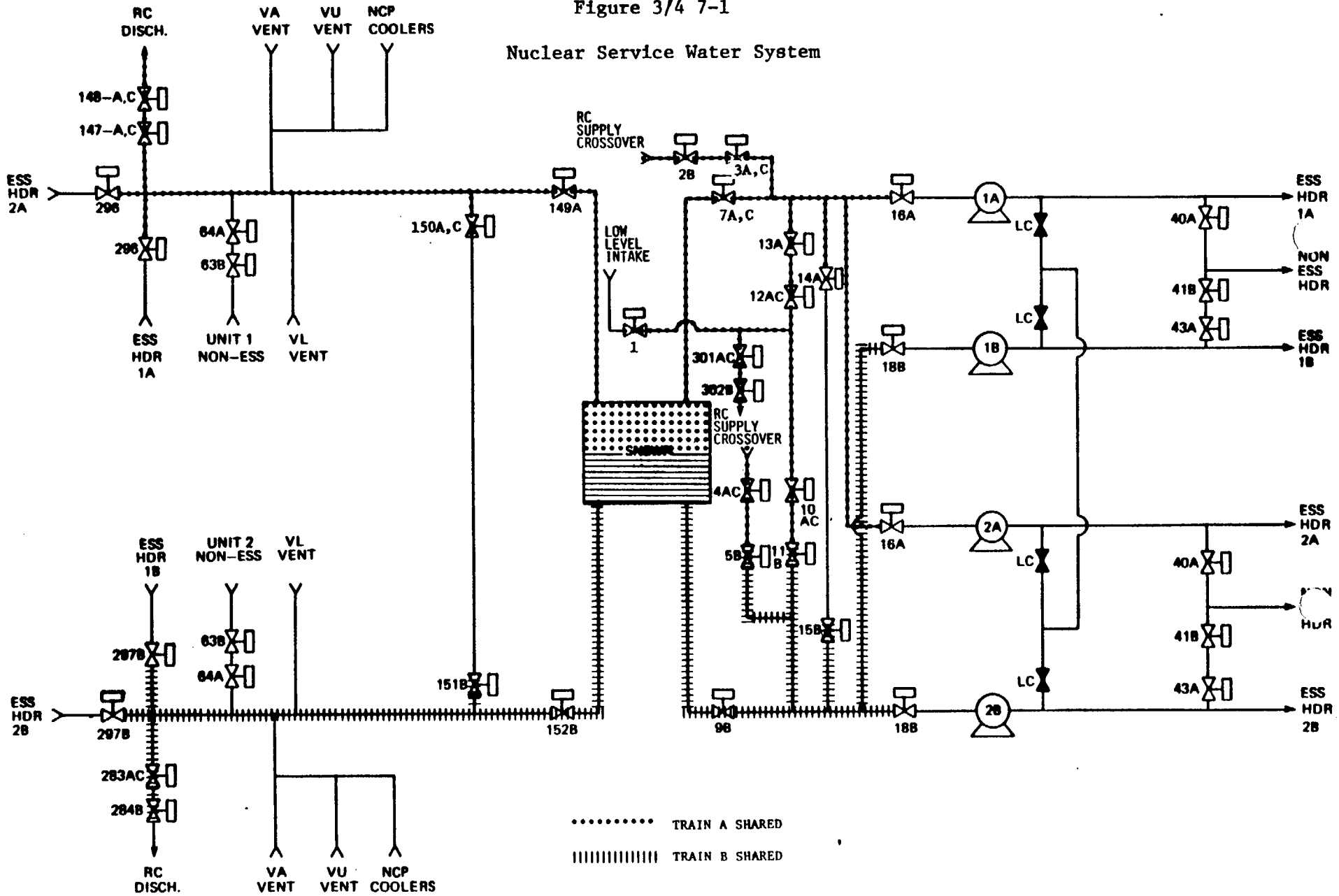
- a. With the unit specific portion of only one nuclear service water loop per unit OPERABLE, restore both unit specific loops to OPERABLE status within 72 hours or place the affected unit at least in HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.
- b. With only one of the shared portions as defined by Figure 3/4 7-1 of the Unit 1 and Unit 2 nuclear service water loops OPERABLE, restore the shared portion of the loops to OPERABLE status within 72 hours or place both units in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

SURVEILLANCE REQUIREMENTS

4.7.4 At least two nuclear service water loops per unit shall be demonstrated OPERABLE:

- a. At least once per 31 days by verifying that each valve (manual, power-operated, or automatic) servicing safety-related equipment that is not locked, sealed, or otherwise secured in position is in its correct position; and
- b. At least once per 18 months during shutdown, by verifying that:
 - 1) Each automatic valve servicing safety-related equipment actuates to its correct position on a Safety Injection test signal, and
 - 2) Each nuclear service water pump starts automatically on a Safety Injection and Station Blackout test signal.

Figure 3/4 7-1
Nuclear Service Water System



PLANT SYSTEMS

3/4.7.5 STANDBY NUCLEAR SERVICE WATER POND

LIMITING CONDITION FOR OPERATION

- 3.7.5 The standby nuclear service water pond shall be OPERABLE with:
- a. A minimum water level at or above elevation 739.5 feet Mean Sea Level, USGS datum, and
 - b. An average water temperature of less than or equal to 78°F at elevation 700 feet in the intake structure.

APPLICABILITY: MODES 1, 2, 3, and 4.

ACTION: (Units 1 and 2)

With the requirements of the above specification not satisfied, be in at least HOT STANDBY within 6 hours and in COLD SHUTDOWN within the following 30 hours.

SURVEILLANCE REQUIRMENTS

- 4.7.5 The standby nuclear service water pond shall be determined OPERABLE:
- a. At least once per 24 hours by verifying the water level to be within its limit,
 - b. At least once per 24 hours during the months of July, August and September by verifying the water temperature to be within its limit, and
 - c. At least once per 12 months by visually inspecting the dam and verifying no abnormal degradation, erosion, or excessive seepage.

PLANT SYSTEMS

BASES

3/4.7.1.4 MAIN STEAM LINE ISOLATION VALVES

The OPERABILITY of the main steam line isolation valves ensures that no more than one steam generator will blow down in the event of a steam line rupture. This restriction is required to: (1) minimize the positive reactivity effects of the Reactor Coolant System cooldown associated with the blowdown, and (2) limit the pressure rise within containment in the event the steam line rupture occurs within containment. The OPERABILITY of the main steam isolation valves within the closure times of the Surveillance Requirements are consistent with the assumptions used in the accident analyses.

3/4.7.2 STEAM GENERATOR PRESSURE/TEMPERATURE LIMITATION

The limitation on steam generator pressure and temperature ensures that the pressure-induced stresses in the steam generators do not exceed the maximum allowable fracture toughness stress limits. The limitations of 70°F and 200 psig are based on a steam generator RT_{NDT} of 10°F and are sufficient to prevent brittle fracture.

3/4.7.3 COMPONENT COOLING WATER SYSTEM

The OPERABILITY of the Component Cooling Water System ensures that sufficient cooling capacity is available for continued operation of safety-related equipment during normal and accident conditions. The redundant cooling capacity of this system, assuming a single failure, is consistent with the assumptions used in the accident analyses.

3/4.7.4 NUCLEAR SERVICE WATER SYSTEM

The OPERABILITY of the Nuclear Service Water System ensures that sufficient cooling capacity is available for continued operation of safety-related equipment during normal and accident conditions. The redundant cooling capacity of this system, assuming a single failure, is consistent with the assumptions used in the accident conditions within acceptable limits. Periodic flow balance tests, delta-P tests, and heat balance tests are performed as required to assure adequate flow to all essential heat exchangers for which flow instrumentation is provided.

Portions of the Nuclear Service Water System are common to both units. These shared portions of the system are indicated on Figure 3/4 7-1 and common valves are listed in Table B 3/4 7-1 and include common suction piping and cross-connect piping as indicated on the figure.

With the exception of ORN-1 all shared valves receive emergency power from two essential motor control centers (1EMXH, 2EMXH). ORN-1 is normally open with the power removed. Motor Control Center (MCC) 1EMXH can be powered by either Unit 1 or Unit 2 A Train Emergency D/G's via their associated switchgear. Motor Control Center 2EMXH can be powered by either Unit 1 or 2 B Train Emergency D/G's via their associated switchgear.

PLANT SYSTEMS

BASES

3/4.7.4 NUCLEAR SERVICE WATER SYSTEM (Continued)

The four loops (two per unit) ensure redundancy and the availability of cooling to both units, even if a single failure were to render two loops inoperable. (Such a failure would involve train A of both units or train B of both units, not both trains of the same unit). The Action statements are separated to clarify that portions of the systems are shared though the majority of each of the four loops is independent.

In the event of a safety injection or blackout signal on either unit, train A of both units will align to Lake Norman and train B of both units will align to the SNSWP. Additionally, all train A to train B cross-connects will close on both units as will non-safety to safety related cross-connects. These actuations assure independence of the loops and the required redundancy under design basis conditions.

3/4.7.5 STANDBY NUCLEAR SERVICE WATER POND

The limitations on the standby nuclear service water pond level and temperature ensure that sufficient cooling capacity is available either to: (1) provide normal cooldown of the facility, or (2) mitigate the effects of accident conditions within acceptable limits.

TABLE B 3/4 7-1

UNITS 1 AND 2

NUCLEAR SERVICE WATER SYSTEM SHARED VALVES

ORN-1	LOW LEVEL INTAKE SUP TO RN
ORN-2B	TRAIN A RC SUPPLY
ORN-3A,C	TRAIN A RC SUPPLY
ORN-4A,C	TRAIN B RC SUPPLY
ORN-5B	TRAIN B RC SUPPLY
ORN-7A,C	TRAIN A SNSWP SUPPLY
ORN-9B	TRAIN B SNSWP SUPPLY
ORN-10A,C	TRAIN B LLI SUPPLY
ORN-11B	TRAIN B LLI SUPPLY
ORN-12A,C	TRAIN A LLI SUPPLY
ORN-13A	TRAIN A LLI SUPPLY
ORN-14A	TRAIN A SUCT X-CONNECT
ORN-15B	TRAIN B SUCT X-CONNECT
ORN-147A,C	TRAIN A DISCH TO RC
ORN-148A,C	TRAIN A DISCH TO RC
ORN-149A	TRAIN A DISCH TO RC
ORN-150A,C	TRAIN A DISCH X-CONNECT
ORN-151B	TRAIN B DISCH X-CONNECT
ORN-152B	TRAIN B DISCH TO SNSWP
ORN-283A,C	TRAIN B DISCH TO RC
ORN-284B	TRAIN B DISCH TO RC
ORN-301A,C	RV SUPPLY FROM LLI
ORN-302B	RV SUPPLY FROM LLI

STANDBY NUCLEAR SERVICE WATER POND (Continued)

The limitations on minimum water level and maximum temperature are based on providing a 30-day cooling water supply to safety-related equipment without exceeding their design basis temperature and is consistent with the recommendations of Regulatory Guide 1.27, "Ultimate Heat Sink for Nuclear Plants," March 1974. The Surveillance Requirements specified for the dam inspection will conform to the recommendations of Regulatory Guide 1.127, Revision 1, March 1978

3/4.7.6 CONTROL AREA VENTILATION SYSTEM

The OPERABILITY of the Control Area Ventilation System ensures that: (1) the ambient air temperature does not exceed the allowable temperature for continuous duty rating for the equipment and instrumentation cooled by this system, and (2) the control room will remain habitable for operations personnel during and following all credible accident conditions. Cumulative operation of the system with the heaters on for 10 hours over a 31-day period is sufficient to reduce the buildup of moisture on the adsorbers and HEPA filters. The OPERABILITY of this system in conjunction with control room design provisions is based on limiting the radiation exposure to personnel occupying the control room to 5 rem or less whole body, or its equivalent. This limitation is consistent with the requirements of General Design Criterion 19 of Appendix A, 10 CFR 50. ANSI N510-1975 will be used as a procedural guide for surveillance testing.

3/4.7.7 AUXILIARY BUILDING FILTERED VENTILATION EXHAUST SYSTEM

The OPERABILITY of the Auxiliary Building Filtered Ventilation Exhaust System ensures that radioactive materials leaking from the ECCS equipment within the auxiliary building following a LOCA are filtered prior to reaching the environment. The operation of this system and the resultant effect on offsite dosage calculations were assumed in the accident analyses. ANSI N510-1975 will be used as a procedural guide for surveillance testing. The methyl iodide penetration test criteria for the carbon samples have been made more restrictive than required for the assumed iodine removal in the accident analysis because the humidity may be greater than 70% under normal operating conditions.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO.78 TO FACILITY OPERATING LICENSE NPF-9
AND AMENDMENT NO. 59 TO FACILITY OPERATING LICENSE NPF-17

DUKE POWER COMPANY

DOCKET NOS. 50-369 AND 50-370

McGUIRE NUCLEAR STATION, UNITS 1 AND 2

I. INTRODUCTION

In NRC Inspection Reports Nos. 50-369/85-38 and 50-370/85-39, dated June 27, 1986, the staff identified concerns involving the McGuire Units 1 and 2 nuclear service water (RN) system. One of these concerns related to the Technical Specifications (TS) for the RN system, TS 3.4.7.4, which did not indicate the shared aspects of the McGuire RN system. A Notice of Violation and Proposed Imposition of Civil Penalty (Notice) was subsequently transmitted (Enforcement Action 87-07) to Duke Power Company (the licensee) on March 6, 1987. Item B of that Notice concerned the fact that the licensee, contrary to TS 3/4.7.4, cross-connected the 1A and 2A trains of the McGuire RN system based on an incorrect 10 CFR 50.59 evaluation. By letter dated April 3, 1987, the licensee responded to Enforcement Action 87-07 and committed to revise the RN system TS to reflect the shared aspects.

By letter dated July 31, 1987, the licensee proposed license amendments to revise the RN system TS accordingly. The proposed license amendments revise the action statement of the RN system TS to reflect that some portions of the RN system are shared between the two units, but the system is not shared in its entirety. The shared and unshared portions of the RN system would be specifically identified in the TS by adding Figure 3/4 7-1 to the specifications and adding new text to the Bases Section in addition to Table B 3/4 7-1 which lists the Unit 1 and 2 shared valves. The proposed amendments would also clarify the related surveillance requirement which is intended to be applied on a "per unit" basis.

II. EVALUATION

The revision to the ACTION Section of TS 3/4.7.4 includes separate statements to clarify the actions and allowable outage times for the situations in which an inoperable component affects a single unit or both units. The previous action statement required that with only one RN loop OPERABLE, two loops were to be OPERABLE within 72 hours or the plant was to be in at least hot standby within the next 6 hours, and in cold shutdown within the following 30 hours. The first action statement, item (a) of the revised specification, is basically the same as the previous except that it requires the ACTION to be taken when the unit specific portion of only one RN loop is OPERABLE and requires that both unit specific loops be returned to OPERABLE status. The action statement,

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therefore, applies to only one unit. The second action statement, item (b) of the revised specification, addresses the shared portions of the RN system as defined by the new Figure 3/4 7-1. It requires that with only one of the shared portions of the loop OPERABLE, both portions are to be returned to an OPERABLE status within 72 hours or both units are to be in hot standby within the next 6 hours and in cold shutdown within the following 30 hours.

In addition to the new action statements, the amendments also revise the surveillance requirements to state that at least two RN loops "per unit" shall be demonstrated OPERABLE. The previous surveillance requirements did not specify "per unit." The BASES Section of the Technical Specifications is also revised to reflect the shared aspects of the system.

The revised action statements include the same time requirements (6 hours for hot standby and 30 hours for cold shutdown) as the previous TS. The revision is to assure that whenever a shared portion of the RN system is inoperable, both units are affected by the action statement. The design of the RN system meets the requirements of General Design Criterion (GDC) 5, "Sharing of Structures, Systems and Components" in that no single failure will prevent the RN system from performing its safety function, including, in the event of an accident in one unit, an orderly shutdown and cooldown of the other unit. However, the previous TS 3/4.7.4 could have been interpreted in a manner which would indicate that GDC 5 could be violated in the event of certain equipment failures if the action statement was not applied to both units. The revised TS 3/4.7.4 ensures that GDC 5 will be met by specifically identifying which portions of the RN system are shared and under what conditions the action statement should be applied to both units.

During its review of the licensee's submittal, the staff noted a labeling discrepancy between Table B 3/4 7-1 and Figure 3/4 7-1. In the table, the valve in the low level intake supply lines to the RN system was labeled ORN-1 while the valve of interest on the drawing did not have an identifying number. Also in the table the valves identified as ORN-3A, C, ORN-7A, C and ORN-150A, C did not have the "C" designation in Figure 3/4 7-1. Because these letters "A" and "C" have power supply connotations and since the figure is to be part of the TS, the staff concluded it should be corrected to agree with Table B 3/4 7-1. In a subsequent discussion with the licensee, the licensee agreed that the proposed TS Figure 3/4 7-1 should be revised such that the valves numbered 3A, 7A and 150A become 3A, C; 7A, C and 150A, C while the single unnumbered valve between the low level intake and the RN system should be designated 1, to agree with Table B 3/4 7-1 where it is identified as ORN-1. Therefore, these administrative changes have been made.

Accordingly, the staff has reviewed the revision to TS 3/4.7.4 and its bases B 3/4.7.4 along with the licensee's justification and safety analysis related to the revision. Based on its review, the staff concludes that the revision, which is primarily a clarification, properly reflects the shared and unshared aspects of the nuclear service water system and adequately specifies when the action statement for the limiting conditions for operation should apply to both units. The revision, thereby, assures that the McGuire nuclear service water system will be operated in accordance with GDC 5, as it relates to sharing of safety related equipment. The staff, therefore, concludes that the revision is acceptable.

III. ENVIRONMENTAL CONSIDERATION

These amendments involve changes to the installation or use of facility components located within the restricted area as defined in 10 CFR Part 20 and clarification of a surveillance requirement. The staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite and that there is no significant increase in individual or cumulative occupational exposure. The NRC staff has made a determination that the amendments involve no significant hazards consideration, and there has been no public comment on such finding. Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of these amendments.

IV. CONCLUSION

The Commission made a proposed determination that the amendments involve no significant hazards consideration which was published in the Federal Register (52 FR 34003) on September 9, 1987. The administrative change subsequently made by the NRC staff regarding the labeling of certain valves in TS Figure 3/4 7-1 does not change this determination or its basis. The Commission consulted with the state of North Carolina. No public comments were received, and the state of North Carolina did not have any comments.

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributors: Darl S. Hood, PDII-3/DRPI/II
William LeFave, Plant Systems Branch

Dated: January 4, 1988

DATED: January 4, 1988

AMENDMENT NO. 78 TO FACILITY OPERATING LICENSE NPF-9 - McGuire Nuclear Station, Unit 1
AMENDMENT NO. 59 TO FACILITY OPERATING LICENSE NPF-17 - McGuire Nuclear Station, Unit 2

DISTRIBUTION:

Docket File 50-369/370

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