



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

January 9, 1986

Docket Nos: 50-369
and 50-370

Mr. H. B. Tucker, Vice President
Nuclear Production Department
Duke Power Company
422 South Church Street
Charlotte, North Carolina 28242

Dear Mr. Tucker:

Subject: Issuance of Amendment No. 50 to Facility Operating License
NPF-9 and Amendment No. 31 to Facility Operating License
NPF-17 - McGuire Nuclear Station, Units 1 and 2

The Nuclear Regulatory Commission has issued the enclosed Amendment No. 50 to Facility Operating License NPF-9 and Amendment No. 31 to Facility Operating License NPF-17 for the McGuire Nuclear Station, Units 1 and 2. These amendments are issued in response to your application dated April 2, 1985.

The amendments revise a surveillance requirement associated with Technical Specification 3/4.1.3.3, Rod Position Indication System. A portion of your application regarding the applicability footnote is not included in these amendments and is the subject of separate correspondence. The amendments are effective as of their date of issuance.

A copy of the related safety evaluation supporting Amendment No. 50 to Facility Operating License No. NPF-9 and Amendment No. 31 to Facility Operating License NPF-17 is enclosed.

Notice of issuance will be included in the Commission's next bi-weekly Federal Register notice.

Sincerely,

B. J. Youngblood, Director
Project Directorate No. 4
Division of PWR Licensing-A

Enclosures:

1. Amendment No. 50 to NPF-9
2. Amendment No. 31 to NPF-17
3. Safety Evaluation

cc w/enclosures:
See next page

Certified By

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McGuire Nuclear Station

cc:

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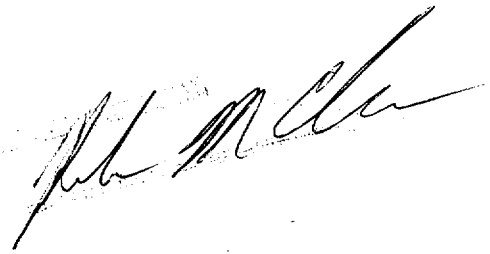
January 9, 1986

AMENDMENT NO. 50 TO FACILITY OPERATING LICENSE NPF-9 - MCGUIRE NUCLEAR STATION, UNIT 1
AMENDMENT NO. 31 TO FACILITY OPERATING LICENSE NPF-17 - MCGUIRE NUCLEAR STATION, UNIT 2

DISTRIBUTION:

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Submitted by

A handwritten signature in dark ink, appearing to be "John M. Chen", is written over a faint, circular official stamp.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

DUKE POWER COMPANY

DOCKET NO. 50-369

McGUIRE NUCLEAR STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 50
License No. NPF-9

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment to the McGuire Nuclear Station, Unit 1 (the facility) Facility Operating License No. NPF-9 filed by the Duke Power Company (the licensee) dated April 2, 1985, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application as amended, the provisions of the Act, and the regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is hereby amended by page changes to the Technical Specifications as indicated in the attachments to this license amendment and paragraph 2.C.(2) of Facility Operating License No. NPF-9 is hereby amended to read as follows:

(2) Technical Specifications

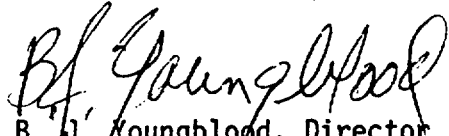
The Technical Specifications contained in Appendix A, as revised through Amendment No. 50, are hereby incorporated into this license.

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The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION


B. J. Youngblood, Director
Project Directorate No. 4
Division of PWR Licensing-A

Attachment:
Technical Specification Changes

Date of Issuance: January 9, 1986



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

DUKE POWER COMPANY

DOCKET NO. 50-370

McGUIRE NUCLEAR STATION, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 31
License No. NPF-17

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment to the McGuire Nuclear Station, Unit 2 (the facility) Facility Operating License No. NPF-17 filed by the Duke Power Company (the licensee) dated April 2, 1985, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application as amended, the provisions of the Act, and the regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is hereby amended by page changes to the Technical Specifications as indicated in the attachments to this license amendment and paragraph 2.C.(2) of Facility Operating License No. NPF-17 is hereby amended to read as follows:

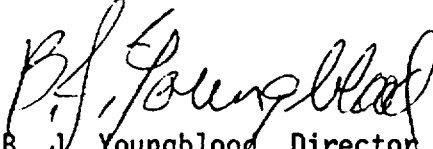
(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 31, are hereby incorporated into this license.

The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION


B. J. Youngblood, Director
Project Directorate No. 4
Division of PWR Licensing-A

Attachment:
Technical Specification Changes

Date of Issuance: January 9, 1986

ATTACHMENT TO LICENSE AMENDMENT NO. 50

FACILITY OPERATING LICENSE NO. NPF-9

DOCKET NO. 50-369

AND

TO LICENSE AMENDMENT NO. 31

FACILITY OPERATING LICENSE NO. NPF-17

DOCKET NO. 50-370

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised page is identified by Amendment number and contains vertical lines indicating the areas of change. The corresponding overleaf page is provided to maintain document completeness.

Amended
Page

3/4 1-18

Overleaf
Page

3/4 1-17

REACTIVITY CONTROL SYSTEMS

POSITION INDICATION SYSTEMS-OPERATING

LIMITING CONDITION FOR OPERATION

3.1.3.2 The Shutdown and Control Rod Position Indication System and the Demand Position Indication System shall be OPERABLE and capable of determining the control rod positions within ± 12 steps.

APPLICABILITY: MODES 1 and 2.

ACTION:

- a. With a maximum of one rod position indicator per bank inoperable either:
 1. Determine the position of the nonindicating rod(s) indirectly by the movable incore detectors at least once per 8 hours and immediately after any motion of the nonindicating rod which exceeds 24 steps in one direction since the last determination of the rod's position, or
 2. Reduce THERMAL POWER to less than 50% of RATED THERMAL POWER within 8 hours.
- b. With a maximum of one demand position indicator per bank inoperable either:
 1. Verify that all rod position indicators for the affected bank are OPERABLE and that the most withdrawn rod and the least withdrawn rod of the bank are within a maximum of 12 steps of each other at least once per 8 hours, or
 2. Reduce THERMAL POWER to less than 50% of RATED THERMAL POWER within 8 hours.

SURVEILLANCE REQUIREMENTS

4.1.3.2 Each rod position indicator shall be determined to be OPERABLE by verifying that the Demand Position Indication System and the Rod Position Indication System agree within 12 steps at least once per 12 hours except during time intervals when the Rod Position Deviation Monitor is inoperable, then compare the Demand Position Indication System and the Rod Position Indication System at least once per 4 hours.

REACTIVITY CONTROL SYSTEMS

POSITION INDICATION SYSTEM-SHUTDOWN

LIMITING CONDITION FOR OPERATION

3.1.3.3 One rod position indicator (excluding demand position indication) shall be OPERABLE and capable of determining the control rod position within ± 12 steps for each shutdown or control rod not fully inserted.

APPLICABILITY: MODES 3*#, 4*# and 5*#.

ACTION:

With less than the above required position indicator(s) OPERABLE, immediately open the Reactor Trip System breakers.

SURVEILLANCE REQUIREMENTS

4.1.3.3 Each of the above required rod position indicator(s) shall be determined to be OPERABLE by performance of an ANALOG CHANNEL OPERATIONAL TEST at least once per 18 months. The Reactor Trip System Breakers can be closed in order to perform this surveillance.

*With the Reactor Trip System breakers in the closed position.
#See Special Test Exception 3.10.5.



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SAFETY EVALUATION REPORT

RELATED TO AMENDMENT NO. 50 TO FACILITY OPERATING LICENSE NPF-9

AND TO AMENDMENT NO. 31 TO FACILITY OPERATING LICENSE NPF-17

DUKE POWER COMPANY

DOCKET NOS. 50-369 AND 50-370

MCGUIRE NUCLEAR STATION, UNITS 1 AND 2

INTRODUCTION

By letter dated April 2, 1985, Duke Power Company (the licensee) requested a revision to the Technical Specification Surveillance Requirements (3/4.1.3.3) for the rod position indication system in Modes 3, 4 and 5 (Hot standby, Hot shutdown, and Cold shutdown). The proposed change is to add a statement to allow the closing of the Reactor Trip System (RTS) breakers in order to perform the required surveillance of Specification 4.1.3.3. In the event of an inoperable indicator, the RTS breakers are required to be opened which would, in preventing rod motion, increase the time and radiation exposure associated with the surveillance. Action on that part of the proposed amendments which would have added "Control Rod Drive System capable of rod withdrawal" has been deferred pending receipt of further information.

EVALUATION

Surveillance Specification 4.1.3.3 requires rod position indicator(s) to periodically be determined to be operable by performance of an analog channel operational test. If a rod position indicator has been declared inoperable, the required Action is to immediately open the RTS breakers. In this situation, the rods cannot be moved. To perform the surveillance without moving the rods requires six workers in the vessel head area for three days to make all required connections and perform the test. This change allows the RTS breakers to be closed, but only during the surveillance, to verify rod position indicator operability by rod motion. If the surveillance results should indicate an inoperable rod position indicator, the required Action (immediately open the RTS breakers) remains unchanged. After completion of repairs, the surveillance would be repeated to determine operability of the indicator(s). Again, the proposed change allows the RTS breakers to be closed during this surveillance. This process would be repeated until the indicator(s) is declared operable.

In support of the proposed change, the licensee provided the justification and safety analysis. Because the proposed change would not affect the required shutdown margin that must be maintained in accordance with Technical Specification 3/4.1.1.1 and 3/4.1.1.2, it would have no impact upon plant safety and is, therefore, acceptable.

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ENVIRONMENTAL CONSIDERATION

These amendments involve changes to the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and a surveillance requirement. We have determined that the amendments involve no significant increase in the amounts and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative radiation exposure. The NRC staff has made a proposed determination that the amendments involve no significant hazards consideration, and there has been no public comment on such finding. Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of these amendments.

CONCLUSION

The Commission made a proposed determination that the amendments involved no significant hazards consideration which was published in the Federal Register (50 FR 46212) on November 6, 1985 and consulted with the state of North Carolina. No public comments were received, and the State of North Carolina did not have any comments.

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributors: D. Hood, PAD 4, Division of PWR Licensing-A
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Dated: January 9, 1986