

Docket

April 10, 1987

Docket Nos.: 50-369
and 50-370

Mr. H. B. Tucker, Vice President
Nuclear Production Department
Duke Power Company
422 South Church Street
Charlotte, North Carolina 28242

Dear Mr. Tucker:

Subject: Issuance of Amendment No.70 to Facility Operating License NPF-9 and
Amendment No.51 to Facility Operating License NPF-17 - McGuire
Nuclear Station, Units 1 and 2

The Nuclear Regulatory Commission has issued the enclosed Amendment No.70 to Facility Operating License NPF-9 and Amendment No.51 to Facility Operating License NPF-17 for the McGuire Nuclear Station, Units 1 and 2. These amendments consist of changes to the Technical Specifications in response to some of the requests in your letters dated May 14, July 14, and November 21, 1986 and March 12, 1987. Other changes requested in these letters have been or will be addressed separately.

The amendments change Technical Specifications 3.6-1 and 3.6-2 to reflect a change in service [from Upper Head Injection (UHI) test line to Post Accident Liquid Sampling (PALS) return line] for containment penetration M348. The amendments are effective as of their date of issuance.

A copy of the related safety evaluation supporting Amendment No.70 to Facility Operating License NPF-9 and Amendment No.51 to Facility Operating License NPF-17 is enclosed.

Notice of issuance of amendments will be included in the Commission's next bi-weekly Federal Register notice.

Sincerely,

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Darl Hood, Project Manager
PWR Project Directorate #4
Division of PWR Licensing-A

Enclosures:

1. Amendment No.70 to NPF-9
2. Amendment No.51 to NPF-17
3. Safety Evaluation

cc w/enclosures: See next page

Distribution:

See attached page

DSH
PWR#4/DPWR-A
DHood/mac
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PWR#4/DPWR-A
MDuncan
08/1/87

PWR#4/DPWR-A
BJYoungblood
08/1/87

DATED: April 10, 1987

AMENDMENT NO.70 TO FACILITY OPERATING LICENSE NPF-9 - McGuire Nuclear Station, Unit 1
AMENDMENT NO.51 TO FACILITY OPERATING LICENSE NPF-17 - McGuire Nuclear Station, Unit 2

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

DUKE POWER COMPANY

DOCKET NO. 50-369.

McGUIRE NUCLEAR STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 70
License No. NPF-9

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment to the McGuire Nuclear Station, Unit 1 (the facility) Facility Operating License No. NPF-9 filed by the Duke Power Company (the licensee) dated May 14, 1986, as revised or supplemented July 14, and November 21, 1986, and March 12, 1987, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations as set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, as amended, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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2. Accordingly, the license is hereby amended by page changes to the Technical Specifications as indicated in the attachments to this license amendment, and Paragraph 2.C.(2) of Facility Operating License No. NPF-9 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 70, are hereby incorporated into the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

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Darl Hood, Project Manager
PWR Project Directorate #4
Division of PWR Licensing-A

Attachment:
Technical Specification
Changes

Date of Issuance: April 10, 1987

PWR#4/DPWR-A
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PWR#4/DPWR-A
BJYoungblood
04/2/87



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

DUKE POWER COMPANY

DOCKET NO. 50-370

McGUIRE NUCLEAR STATION, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 51
License No. NPF-17

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment to the McGuire Nuclear Station, Unit 2 (the facility) Facility Operating License No. NPF-17 filed by the Duke Power Company (the licensee) dated May 14, 1986, as revised or supplemented July 14, and November 21, 1986, and March 12, 1987, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations as set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, as amended, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is hereby amended by page changes to the Technical Specifications as indicated in the attachments to this license amendment, and Paragraph 2.C.(2) of Facility Operating License No. NPF-17 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No.51, are hereby incorporated into the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

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Darl Hood, Project Manager
PWR Project Directorate #4
Division of PWR Licensing-A

Attachment:
Technical Specification
Changes

Date of Issuance: April 10, 1987

PWR#4/DPWR-A
MDuncan:mac
04/1/87

DSIT
PWR#4/DPWR-A
DHood
04/1/87

*After amendment letter
check STATE & SEC
before issuance
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OGC/BETH
MYoung
04/3/87
PWR#4/DPWR-A
BJYoungblood
04/9/87

ATTACHMENT TO LICENSE AMENDMENT NO.70

FACILITY OPERATING LICENSE NO. NPF-9

DOCKET NO. 50-369

AND

TO LICENSE AMENDMENT NO.51

FACILITY OPERATING LICENSE NO. NPF-17

DOCKET NO. 50-370

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the areas of change.

Amended
Page

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3/4 6-9a
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3/4 6-30

TABLE 3.6-1
SECONDARY CONTAINMENT BYPASS LEAKAGE PATHS

<u>PENETRATION NUMBER</u>	<u>SERVICE</u>	<u>RELEASE LOCATION</u>	<u>TEST TYPE</u>
M216	Pressurizer Relief Tank Makeup	Auxiliary Building	Type C
M212	Nitrogen to Pressurizer Relief Tank	Auxiliary Building	Type C
M259	Reactor Makeup Water Tank to NV System	Auxiliary Building	Type C
M373	Ice Condenser Glycol In	Auxiliary Building	Type C
M372	Ice Condenser Glycol Out	Auxiliary Building	Type C
M330	Nitrogen to Accumulators	Auxiliary Building	Type C
M321	Safety Injection Test Line	Auxiliary Building	Type C
M348	Upper Head Injection Test Line# Post Accident Liquid Sample Discharge#	Auxiliary Building	Type C
M374	Containment Floor Sump Incore Instrument Sump Discharge	Auxiliary Building	Type C
M360	Reactor Coolant Drain Tank Gas Space to Waste Gas System	Auxiliary Building	Type C
M375	Reactor Coolant Drain Tank Heat Exchanger Discharge	Auxiliary Building	Type C
M356	Equipment Decontamination	Auxiliary Building	Type C
M235	Pressurizer Sample	Auxiliary Building	Type C
M309	Reactor Coolant Hot Leg Sample	Auxiliary Building	Type C
M322	Component Cooling to Component Cooling Drain Tank	Auxiliary Building	Type C

McGUIRE - UNITS 1 and 2

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Amendment No. 70 (Unit 1)
Amendment No. 51 (Unit 2)

TABLE 3.6-1 (Continued)

SECONDARY CONTAINMENT BYPASS LEAKAGE PATHS

<u>PENETRATION NUMBER</u>	<u>SERVICE</u>	<u>RELEASE LOCATION</u>	<u>TEST TYPE</u>
---	Cont. Press. Monitor Narrow Range	Auxiliary Building	Type C
M354	Fuel Transfer Tube	Auxiliary Building	Type B

#Upon the deactivation of the Upper Head Injection System by removal of related components and piping and modifications to the Cold Leg Accumulators, this penetration is utilized for Post Accident Liquid Sample discharge.

McGUIRE - UNITS 1 and 2

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Amendment No. 51 (Unit 2)
Amendment No. 70 (Unit 1)

TABLE 3.6-2 (Continued)
CONTAINMENT ISOLATION VALVES

<u>VALVE NUMBER</u>	<u>FUNCTION</u>	<u>MAXIMUM ISOLATION TIME (SEC)</u>
1. Phase "A" Isolation (continued)		
WL-1B	NCDT Pumps Discharge Outside Containment Isolation	<10
WL-2A	NCDT Pumps Discharge Inside Containment Isolation	<10
WL-39A	NCDT Vent Inside Containment Isolation	<10
WL-41B	NCDT Vent Outside Containment Isolation	<10
WL-64A	RB Sump Pump Discharge Inside Containment Isolation	<15
WL-65B	RB Sump Pump Discharge Outside Containment Isolation	<15
WL-321A	Containment Vent Unit Drains Inside Containment Isolation	<15
WL-322B	Containment Vent Unit Drains Outside Containment Isolation	<15
WL-1301B##	PALS Discharge Outside Containment Isolation	<15
WL-1302A##	PALS Discharge Inside Containment Isolation	<15
YM-115B	Demin. Water Containment Outside Isolation	<15
2. Phase "B" Isolation		
KC-338B	NC Pump Supply Header Pent. Isolation (outside)	<40
KC-424B	NC Pumps Return Hdr. Pent Inside Isolation	<40
KC-425A	NC Pumps Return Hdr. Outside Isolation	<40
RN-252B	Nonessential Supply to PB Penetration Outside Isolation	<30
RN-253A	Nonessential Supply to RB Penetration Inside Isolation	<30
RN-276A	Nonessential Return to RB Penetration Inside Isolation	<30
RN-277B	Nonessential Return to RB Penetration Outside Isolation	<30
RV-32A	Lower Containment Vent. Unit Supply Containment Isolation (outside)	<60
RV-33B	Lower Containment Vent. Unit Supply Containment Isolation (inside)	<60
RV-76A	Lower Containment Vent. Unit Discharge Containment Isolation (inside)	<60
RV-77B	Lower Containment Vent. Unit Discharge Containment Isolation (outside)	<60
VI-129B	"A" Header Containment Outside Isolation	<15
VI-150B	Instrument Air Lower Containment Outside Isolation	<15
VI-160B	"B" Header Containment Outside Isolation	<15

MCGUIRE - UNITS 1 and 2

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Amendment No. 51 (Unit 2)
Amendment No. 70 (Unit 1)

TABLE 3.6-2 (Continued)
CONTAINMENT ISOLATION VALVES

<u>VALVE NUMBER</u>	<u>FUNCTION</u>	<u>MAXIMUM ISOLATION TIME (SEC)</u>
2. Phase "B" Isolation (continued)		
RV-79A	Upper Containment Vent. Unit Supply Containment Isolation Outside	<30
RV-80B	Upper Containment Vent. Unit Supply Containment Isolation Inside	<30
RV-101A	Upper Containment Vent. Unit Discharge Containment Isolation Inside	<30
RV-102B	Upper Containment Vent. Unit Discharge Containment Isolation Outside	<30
3. Main Steam Isolation		
SM-1AB#	Main Steam D Isolation	<5
SM-3AB#	Main Steam C Isolation	<5
SM-5AB#	Main Steam B Isolation	<5
SM-7AB#	Main Steam A Isolation	<5
SM-9AB#	Main Steam D Isolation Bypass Ctrl.	<5
SM-10AB#	Main Steam C Isolation Bypass Ctrl.	<5
SM-11AB#	Main Steam B Isolation Bypass Ctrl.	<5
SM-12AB#	Main Steam A Isolation Bypass Ctrl.	<5
4. Manual		
NC141*	NC Pump Motor Oil Drain	N.A.
NC142*	NC Pump Motor Oil Drain	N.A.
WE13*	Equipment Decontamination	N.A.
WE23*	Equipment Decontamination	N.A.
VX34*	Containment H ₂ Sample	N.A.
VX40*	Containment H ₂ Sample	N.A.
FW11*	Refueling Water	N.A.
FW13*	Refueling Water	N.A.
FW4*	Refueling Water	N.A.

*May be opened on an intermittent basis under administrative control.

**Valve also receives a High Radiation (H) isolation signal.

Not subject to Type C leakage tests.

Upon the deactivation of the Upper Head Injection System by removal of related components and piping and modifications to the Cold Leg Accumulators, this valve is utilized to isolate Post Accident Liquid Sample discharge.

NOTE: Times are for valve operation only, and do not include any sensor response or circuit delay times. See Specification 3/4.3.2 for system actuation response times.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 70 TO FACILITY OPERATING LICENSE NPF-9
AND AMENDMENT NO. 51 TO FACILITY OPERATING LICENSE NPF-17

DUKE POWER COMPANY

DOCKET NOS. 50-369 AND 50-370

McGUIRE NUCLEAR STATION, UNITS 1 AND 2

INTRODUCTION

By letters dated May 14 and July 14, 1986, Duke Power Company (the licensee) proposed amendments to change the service designation of penetration number M348 in Technical Specification (TS) Table 3.6.1 "Secondary Containment Bypass Leakage Paths" and an associated footnote (Note 1) to indicate that this penetration, currently designated as the Upper Head Injection (UHI) Test Line, will be utilized for the post accident liquid sample (PALS) discharge line following removal of the UHI system. Similarly, the licensee requested that associated valve numbers WL 1301B and WL 1302A on the PALS discharge line, which will receive a Phase A containment isolation signal for automatic closure within at least 15 seconds, be added to TS Table 3.6-2 "Containment Isolation Valves" and noted to be effective upon removal of the UHI system.

By letter dated November 21, 1986, the licensee rephrased the proposed change as recommended by the NRC staff to provide clarity without changing the meaning of the original text. Supplemental information in support of the proposed change and a minor correction were provided by the licensee on March 12, 1987.

EVALUATION

By previous License Amendments No. 57 (Unit 1) and 38 (Unit 2), the NRC authorized changes to the McGuire TSs associated with the physical removal of the UHI systems, including changes to reflect deletion of UHI related containment penetrations and containment isolation valves when the UHI system is removed. By separate action (NUREG-0737, Item II.B.3), the staff provided a position on design of post accident sampling capability, which states that residues of sample collection should be returned to containment or to a closed system. The TS change implemented by these amendments re-designates penetration M348 as a post accident liquid sample (PALS) discharge line. The modified penetration will reuse the previous isolation valves, NI-267A and NI-264B, redesignated as WL-1302A and WL-1301B, respectively. By letter dated March 12, 1987, the licensee corrected the May 14, 1986 letter to note that although valves WL-1301B and WL-1302A had both been designated to be located outside containment, valve WL-1302A should have been designated to be located inside containment. The corrected arrangement, with these valves receiving a Phase "A" Containment Isolation Signal to close within 15 seconds satisfies General Design Criterion (GDC) 56, (part 4) as the penetration will have one automatic isolation valve

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inside containment and one automatic valve outside containment. These valves also are, and will continue to be, subject to the Type C requirements of 10 CFR 50, Appendix J, "Primary Reactor Containment Leakage Testing for Water Cooled Power Reactors."

Accordingly, the staff finds that the PALS discharge line isolation and leak testing provisions conform to GDC 56 and Appendix J requirements respectively. The proposed change is, therefore, acceptable.

ENVIRONMENTAL CONSIDERATION

These amendments involve changes to the installation or use of facility components located within the restricted area as defined in 10 CFR Part 20. The staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite and that there is no significant increase in individual or cumulative occupational exposure. The NRC staff has made a determination that the amendments involve no significant hazards consideration, and there has been no public comment on such finding. Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of these amendments.

CONCLUSION

The Commission made a proposed determination that the amendments involve no significant hazards consideration which was published in the Federal Register (51 FR 30571 and 41852) on August 27 and November 19, 1986. The licensee's subsequent submittals dated November 21, 1986, and March 12, 1987, provide clarity, a minor correction, or supporting information and do not substantively alter the proposed change as described in the Federal Register, and do not affect the Commission's proposed determination of no significant hazards consideration. The Commission consulted with the state of North Carolina. No public comments were received, and the state of North Carolina did not have any comments.

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributors: C. Li, PAPS
Darl S. Hood, PWR#4

Dated: April 10, 1987