

Docket Nos.: 50-369  
and 50-370

September 9, 1987

Mr. H. B. Tucker, Vice President  
Nuclear Production Department  
Duke Power Company  
422 South Church Street  
Charlotte, North Carolina 28242

Dear Mr. Tucker:

Subject: Issuance of Amendment No. 75 to Facility Operating License NPF-9  
and Amendment No. 56 to Facility Operating License NPF-17 - McGuire  
Nuclear Station, Units 1 and 2 (TACs 65437/65438)

The Nuclear Regulatory Commission has issued the enclosed Amendment No. 75 to Facility Operating License NPF-9 and Amendment No. 56 to Facility Operating License NPF-17 for the McGuire Nuclear Station, Units 1 and 2. These amendments consist of changes to the Technical Specifications in response to your application dated July 2, 1987, which utilizes information in your letter of May 4, 1987 and which is supplemented August 26, 1987.

The amendments change the Technical Specifications to incorporate the ventilation system of the Equipment Staging Building as a new gaseous effluent release point, to specify the limiting conditions for operation and surveillance requirements for this ventilation system and its monitoring instrumentation, and to add associated requirements to the gaseous waste sampling and analysis program. The amendments are effective as of their date of issuance.

A copy of the related safety evaluation supporting Amendment No. 75 to Facility Operating License NPF-9 and Amendment No. 56 to Facility Operating License NPF-17 is enclosed.

Notice of issuance of amendments will be included in the Commission's next bi-weekly Federal Register notice.

Sincerely,

*151*  
Darl Hood, Project Manager  
Project Directorate II-3  
Division of Reactor Projects-I/II

Enclosures:

1. Amendment No. 75 to NPF-9
2. Amendment No. 56 to NPF-17
3. Safety Evaluation

cc w/enclosures: See next page

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*mlb*  
PD#II-3/DRP-I/II  
LCrocker, Acting Director  
09/9/87

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McGuire Nuclear Station

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DATED: September 9, 1987

AMENDMENT NO. 75 TO FACILITY OPERATING LICENSE NPF-9 - McGuire Nuclear Station, Unit 1  
AMENDMENT NO. 56 TO FACILITY OPERATING LICENSE NPF-17 - McGuire Nuclear Station, Unit 2

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

DUKE POWER COMPANY

DOCKET NO. 50-369

McGUIRE NUCLEAR STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 75  
License No. NPF-9

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment to the McGuire Nuclear Station, Unit 1 (the facility) Facility Operating License No. NPF-9 filed by the Duke Power Company (the licensee) dated July 2, 1987, and supplemented August 26, 1987, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations as set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, as amended, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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2. Accordingly, the license is hereby amended by page changes to the Technical Specifications as indicated in the attachments to this license amendment, and Paragraph 2.C.(2) of Facility Operating License No. NPF-9 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 75, are hereby incorporated into the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

151

Lawrence P. Crocker, Acting Director  
Project Directorate II-3  
Division of Reactor Projects-I/II

Attachment:  
Technical Specification  
Changes

Date of Issuance: September 9, 1987

PD#II-3/DRP-I/II  
MDuncan/mac  
09/3/87

DSH  
PD#II-3/DRP-I/II  
DHood  
09/3/87

OGC-Bethesda  
SH Lewis  
09/4/87  
with suggested  
change to  
notice

PD#II-3/DRP-I/II  
LCrocker, Acting Director  
09/9/87

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

DUKE POWER COMPANY

DOCKET NO. 50-370

McGUIRE NUCLEAR STATION, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 56  
License No. NPF-17

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment to the McGuire Nuclear Station, Unit 2 (the facility) Facility Operating License No. NPF-17 filed by the Duke Power Company (the licensee) dated July 2, 1987, and supplemented August 26, 1987, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations as set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, as amended, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is hereby amended by page changes to the Technical Specifications as indicated in the attachments to this license amendment, and Paragraph 2.C.(2) of Facility Operating License No. NPF-17 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 56, are hereby incorporated into the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

151

Lawrence P. Crocker, Acting Director  
Project Directorate II-3  
Division of Reactor Projects-I/II

Attachment:  
Technical Specification  
Changes

Date of Issuance: September 9, 1987

PD#II-3/DRP-I/II  
MDuncan/mac  
09/3/87

DSH  
PD#II-3/DRP-I/II  
DHood  
09/3/87

LHZ  
OGC-Bethesda  
SH Lewis  
09/4/87  
with suggestion  
on notices

MG  
PD#II-3/DRP-I/II  
LCrocker, Acting Director  
09/11/87

ATTACHMENT TO LICENSE AMENDMENT NO. 75

FACILITY OPERATING LICENSE NO. NPF-9

DOCKET NO. 50-369

AND

TO LICENSE AMENDMENT NO. 56

FACILITY OPERATING LICENSE NO. NPF-17

DOCKET NO. 50-370

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the areas of change. The corresponding overleaf pages are also provided to maintain document completeness.

Amended  
Page

Overleaf  
Page

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5-4

3/4 3-75  
5-3



TABLE 3.3-13 (Continued)

RADIOACTIVE GASEOUS EFFLUENT MONITORING INSTRUMENTATION

	<u>INSTRUMENT</u>	<u>MINIMUM CHANNELS OPERABLE</u>	<u>APPLICABILITY</u>	<u>ACTION</u>
5.	Containment Purge System			
	Noble Gas Activity Monitor - Providing Alarm and Automatic Termination of Release (Low Range - EMF-39)	1	*	38
6.	Auxiliary Building Ventilation System			
	Noble Gas Activity Monitor (EMF-41 or EMF-36)	1	*	37
7.	Fuel Storage Area Ventilation System			
	Noble Gas Activity Monitor (EMF-42 or EMF-36)	1	*	37
8.	Contaminated Parts Warehouse Ventilation System			
a.	Noble Gas Activity Monitor (EMF-53)	1 per station	***	37
b.	Flow Rate Monitor	1 per station	***	36
c.	Sampler Minimum Flow Device	1 per station	***	36
9.	Radwaste Facility Ventilation System			
a.	Noble Gas Activity Monitor (EMF-52)	1 per station	***	37
b.	Flow Rate Monitor	1 per station	***	36
c.	Sampler Minimum Flow Rate	1 per station	***	36
10.	Equipment Staging Building Ventilation System			
a.	Noble Gas Activity Monitor (EMF-59)	1	***	37
b.	Flow Rate Monitor	1	***	36
c.	Sampler Minimum Flow Device	1	***	36

McGUIRE - UNITS 1 and 2

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Amendment No. 75 (Unit 1)  
Amendment No. 56 (Unit 2)

MCUIRE - UNITS 1 and 2

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TABLE 4.3-9

RADIOACTIVE GASEOUS EFFLUENT MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS

<u>INSTRUMENT</u>	<u>CHANNEL CHECK</u>	<u>SOURCE CHECK</u>	<u>CHANNEL CALIBRATION</u>	<u>ANALOG CHANNEL OPERATIONAL TEST</u>	<u>MODES IN WHICH SURVEILLANCE REQUIRED</u>
1. WASTE GAS HOLDUP SYSTEM					
a. Noble Gas Activity Monitor - Providing Alarm and Automatic Termination of Release (Low Range - EMF-50 or 1EMF-36)	P	P	R(3)	Q(1)	*
b. Effluent System Flow Rate Measuring Device	D	N.A.	R	Q	*
2. WASTE GAS HOLDUP SYSTEM Explosive Gas Monitoring System					
a. Hydrogen Monitor	D	N.A.	Q(4)	M	**
b. Oxygen Monitor	D	N.A.	Q(5)	M	**
c. Oxygen Monitor (alternate)	D	N.A.	Q(5)	M	**
3. Condenser Evacuation System					
Noble Gas Activity Monitor (EMF-33)	D	M	R(3)	Q(2)	*
4. Vent System					
a. Noble Gas Activity Monitor (Low Range - EMF-36)	D	M	R(3)	Q(2)	*
b. Iodine Sampler	W	N.A.	N.A.	N.A.	*
c. Particulate Sampler	W	N.A.	N.A.	N.A.	*
d. Flow Rate Monitor	D	N.A.	R	Q	*
e. Sampler Minimum Flow Device	D	N.A.	R	Q	*

TABLE 4.3-9

RADIOACTIVE GASEOUS EFFLUENT MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS

<u>INSTRUMENT</u>	<u>CHANNEL CHECK</u>	<u>SOURCE CHECK</u>	<u>CHANNEL CALIBRATION</u>	<u>ANALOG CHANNEL OPERATIONAL TEST</u>	<u>MODES IN WHICH SURVEILLANCE REQUIRED</u>
5. Containment Purge System					
Noble Gas Activity Monitor - Providing Alarm and Automatic Termination of Release Low (Range - EMF-39)	D	P	R(3)	Q(1)	*
6. Auxiliary Building Ventilation System					
Noble Gas Activity Monitor (EMF-41 or EMF-36)	D	M	R(3)	Q(2)	*
7. Fuel Storage Area Ventilation System					
Noble Gas Activity Monitor (EMF-42 or EMF-36)	D	M	R(3)	Q(2)	*
8. Contaminated Parts Warehouse Ventilation System					
a. Noble Gas Activity Monitor (EMF-53)	D	M	R(3)	Q(2)	*
b. Flow Rate Monitor	D	N.A.	R	Q	*
c. Sampler Minimum Flow Device	D	N.A.	R	Q	*
9. Radwaste Facility Ventilation System					
a. Noble Gas Activity Monitor (EMF-52)	D	M	R(3)	Q(2)	*
b. Flow Rate Monitor	D	N.A.	R	Q	*
c. Sampler Minimum Flow Device	D	N.A.	R	Q	*
10. Equipment Staging Building Ventilation System					
a. Noble Gas Activity Monitor (EMF-59)	D	M	R(3)	Q(2)	*
b. Flow Rate Monitor	D	N.A.	R	Q	*
c. Sampler Minimum Flow Device	D	N.A.	R	Q	*

McGUIRE - UNITS 1 and 2

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Amendment No. 75 (Unit 1)  
Amendment No. 56 (Unit 2)

TABLE 4.11-2

## RADIOACTIVE GASEOUS WASTE SAMPLING AND ANALYSIS PROGRAM

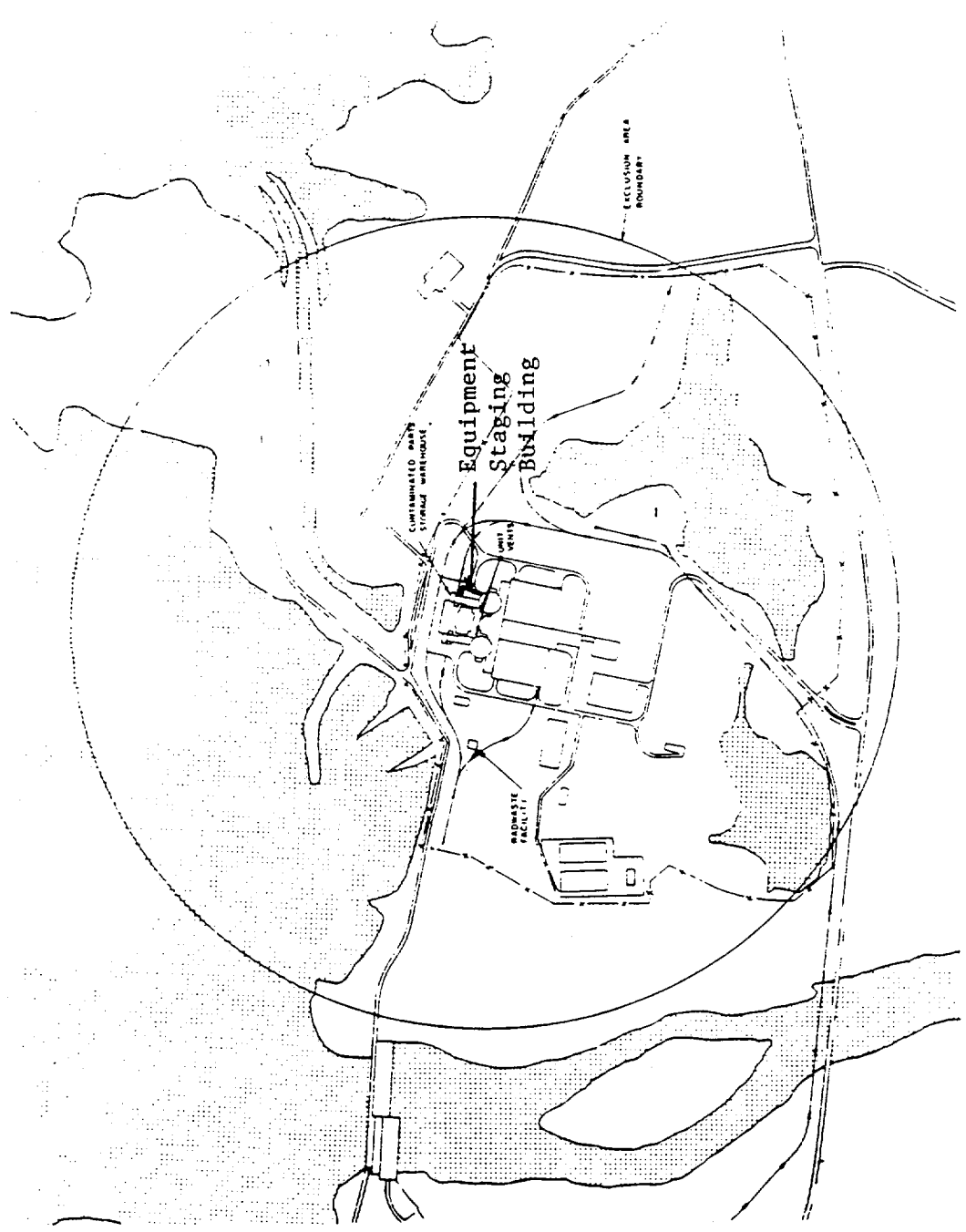
GASEOUS RELEASE TYPE	SAMPLING FREQUENCY	MINIMUM ANALYSIS FREQUENCY	TYPE OF ACTIVITY ANALYSIS	LOWER LIMIT OF DETECTION (LLD) ( $\mu\text{Ci/ml}$ ) <sup>(1)</sup>
1. Waste Gas Storage Tank	P Each Tank Grab Sample	P Each Tank	Principal Gamma Emitters <sup>(7)</sup>	$1 \times 10^{-4}$
2. Containment Purge	P Each Purge <sup>(2)</sup> Grab Sample	P Each Purge <sup>(2)</sup>	Principal Gamma Emitters <sup>(7)</sup>	$1 \times 10^{-4}$
3. Unit Vent	W <sup>(2),(3),(5)</sup> Grab Sample	M	H-3	$1 \times 10^{-6}$
		W <sup>(2)</sup>	Principal Gamma Emitters <sup>(7)</sup>	$1 \times 10^{-4}$
4. a. Radwaste Facility Vent	W Grab Sample	W	Principal Gamma Emitters <sup>(7)</sup>	$1 \times 10^{-4}$
			H-3	$1 \times 10^{-6}$
b. Contaminated Parts Warehouse				
c. Equipment Staging Building				
5. All Release Types as listed in 3. and 4. above.	Continuous <sup>(6)</sup>	D <sup>(4)</sup>	I-131	$1 \times 10^{-11}$
		Charcoal Sample	I-133	$1 \times 10^{-9}$
	Continuous <sup>(6)</sup>	D <sup>(4)</sup>	Principal Gamma Emitters <sup>(7)</sup> (I-131, Others)	$1 \times 10^{-10}$
		Particulate Sample	Gross Alpha <sup>(8)</sup>	$1 \times 10^{-11}$
	Continuous <sup>(6)</sup>	M		
Continuous <sup>(6)</sup>	Composite Particulate Sample			
	Q	Sr-89, Sr-90	$1 \times 10^{-11}$	
		Composite Particulate Sample		

MCGUIRE - UNITS 1 and 2

3/4 11-10

Amendment No.75 (Unit 1)  
Amendment No.56 (Unit 2)





SITE BOUNDARY FOR  
GASEOUS EFFLUENTS  
McGUIRE NUCLEAR STATION

FIGURE 5.1-3



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
RELATED TO AMENDMENT NO. 75 TO FACILITY OPERATING LICENSE NPF-9  
AND AMENDMENT NO. 56 TO FACILITY OPERATING LICENSE NPF-17

DUKE POWER COMPANY

DOCKET NOS. 50-369 AND 50-370

McGUIRE NUCLEAR STATION, UNITS 1 AND 2

INTRODUCTION

By letter dated July 2, 1987, Duke Power Company (the licensee) proposed amendments to change the Technical Specifications (TSs) for McGuire Nuclear Station, Units 1 and 2 to incorporate the ventilation system of the Equipment Staging Building (ESB) as a new gaseous effluent release point, to specify limiting conditions for operation and surveillance requirements for this ventilation system and its monitoring instrumentation, and to add associated requirements to the gaseous waste sampling and analysis program.

Specifically, TS Figure 5.1-3 "Site Boundary for Gaseous Effluents," which shows locations within the Exclusion Area Boundary for radioactive gaseous effluents released to unrestricted areas, would be revised to reflect the addition of the ESB. Changes to TS Table 3.3-13 "Radioactive Gaseous Effluent Monitoring Instrumentation" would add minimum channel operability requirements applicable during gaseous effluent releases, and associated actions required with the number of operable channels less than specified, for the noble gas activity monitor (EMF-59), flow rate monitor, and sampler minimum flow device of the ESB ventilation system. Similarly, changes to TS Table 4.3-9 "Effluent Monitoring Instrumentation Surveillance Requirements" would add surveillance requirements (channel check, source check, channel calibration frequency, and analog channel operational test frequency) for these same three monitors, applicable at all times except when the ventilation system isolation valve is closed and locked. TS Table 4.11-2 "Radioactive Gaseous Waste Sampling and Analysis Program" would be revised to reflect the addition of the ESB as new item 4c, and to reflect sampling and analysis requirements corresponding to those presently specified for the Radwaste Facility Vent (item 4a) and Contaminated Materials Warehouse (item 4b).

An additional change would correct inconsistent names for the same structure; the reference to "Contaminated Materials Warehouse" in TS Table 4.11-2 (item 4b) and to "Contaminated Parts Storage Warehouse" in Figure 5.1-3 would both be changed to "Contaminated Parts Warehouse."

The licensee's application of July 2, 1987, utilized information from a previous letter dated May 4, 1987. Additional supporting information was provided by the licensee in a supplemental letter dated August 26, 1987.

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## EVALUATION

The licensee has recently completed construction of an ESB located outside the Unit 2 Containment equipment hatch and adjacent to the Fuel Building. The ESB is a steel framed structure, built upon a reinforced concrete slab at grade level, with upper walls consisting of metal siding and the lower exterior consisting of brick. It is designed to withstand seismic events and is structurally independent of the adjacent Containment and Fuel Building, being connected to them by flexible joints. It is approximately 50 feet wide, 100 feet long and 60 feet high.

The ESB serves no function regarding the prevention or mitigation of accidents. Its purpose is to provide increased laydown area for the Containment during outages; storage of outage equipment; equipment decontamination; disassembly, servicing and assembly of reactor coolant system components; and environmental protection for equipment and personnel during an outage. The need for the ESB results from limited space for such activities required during an outage and which are presently performed in the Containment, Spent Fuel Building and Hot Machine Shop. Typical activities which would be conducted inside the ESB include vessel head stud cleaning, valve maintenance, cutting of discontinued piping (such as the upper head injection piping) into smaller sections for storage or shipment, and parts and component repair such as reactor coolant pump internals replacement or motor repair. The licensee finds that performing such activities in the ESB would result in a reduction in radiation exposure to workers, reduced outage time and a safer working environment.

Because planned ESB activities involve dry brushing, cutting, grinding and welding of contaminated components and such activities create airborne contamination, the ESB includes a heating, ventilation and air conditioning (HVAC) system with a pre-filter and a high efficiency particulate absolute (HEPA) filter to collect and remove particulates prior to release of exhaust gases to the atmosphere through the new ESB HVAC discharge vent. The ESB also includes a contaminated parts wash down area with provisions to route potentially contaminated liquids to existing station liquid radwaste treatment systems.

The planned ESB activities, if not conducted in the new ESB, would be performed elsewhere in the plant (as is presently the case). Thus, the proposed change does not represent a significant change in the amounts or types of radioactive material in effluents released from the station or associated doses. The gaseous effluents limits of TS 3/4.11.2 which limit the dose rates due to radioactive materials released from the plant in gaseous effluents to areas at and beyond the site boundary are not affected by the change. The only solid waste generated due to ESB usage, that would not otherwise be generated, will result from periodic changeout of the pre-filter and HEPA filter units; this will add less than 100 cubic feet of waste per year. We find this to be an insignificant addition to McGuire's annual solid waste generation total which, in 1986, was 28,194 cubic feet, and therefore to be acceptable.

During fuel movement, the Containment equipment hatch will be closed in accordance with existing TS 3.9.4. This prevents any release from the Unit 2



containment to the ESB in the event of a fuel handling accident. At other times during a refueling outage, control of the open equipment hatch is not adversely affected by the ESB. When the Unit 2 equipment hatch is open, the Containment purge system will maintain a more negative pressure inside Containment than the negative pressure maintained by the ESB HVAC system. These negative pressures maintain air flow from outside through the ESB and into Containment. Thus, in the event of a radioactive release inside the Unit 2 Containment with its equipment hatch open, the hatch can be closed in a timely manner while the release is filtered and monitored through the existing Containment purge system. Therefore, we find that the ESB does not adversely affect potential releases from the Containment.

Although the single ESB is located on the Unit 2 side of the station, it will also be utilized during Unit 1 refueling outages. The transport route for items from Unit 1 will be through the Unit 1 equipment hatch to the outside, along the building exteriors and into the ESB through its truck bay. Any contaminated items will be sealed during transport to prevent the spread of contamination. Thus, the route does not involve the transport of heavy loads over or near safety-related equipment or fuel, and the staff's previous conclusions regarding the handling and transport of heavy loads is not adversely affected by this change.

A pipe trench containing safety-related piping from the refueling water storage tank (RWST) to the safety injection pumps (plus other piping such as the return line to the RWST and piping to and from the reactor makeup water storage tank) penetrates the ESB about floor level. Seismic separation at the penetration is provided by flexible material in a 3 inch lateral space around the pipe trench. Within the ESB, the trench is covered with 8 inch concrete blocks and 2 inch steel plate as missile shielding. The covers have no lifting access. Thus, the safety-related trench is not adversely impacted by the ESB and it is protected inside the ESB as well as outside.

The changes to Tables 3.3-13 and 4.3-9 add the ESB HVAC system noble gas activity monitor, flow rate monitor, and sampler minimum flow device to the TSs. The changes add TS requirements on the system identical to Items 8 and 9 of the tables (the Contaminated Parts Warehouse ventilation system and the Radwaste Facility ventilation system, respectively). The monitor on the ESB is of similar design and will function under similar conditions as the monitors on the Contaminated Parts Warehouse and the Radwaste Facility. The specification requires the operability of the monitor during gaseous effluent releases with sampling and flow estimates required if the monitor is inoperable. The surveillance required is the same as for the Contaminated Parts Warehouse and the Radwaste Facility ventilation systems and again, the system and operational conditions are similar. These changes are therefore equally acceptable. This similarity is also the basis for the proposed change to Table 4.11-2 which requires additional sampling and analysis of the released effluents. This requires that total dose rate as calculated using methodology and parameters of the Offsite Dose Calculation Manual be maintained within the existing limits specified in TS 3.11.2.1. Because of the similarity and because existing dose rate limits will continue to be met, the change to Table 4.11-2 is acceptable. The change to Figure 5.1-3 to designate the new gaseous effluent release point, coupled with the requirements of existing TS 3.3.3.9

(Tables 3.3-13 and 4.3-9) and 3.11.2.1 (Table 4.11-2) means that control of effluent releases from the facility is to be at levels that are as low as is reasonably achievable. This change is therefore acceptable.

The change to TS Table 4.11-2 (item 4b) and to TS Figure 5.1-3 to correct the name of the Contaminated Parts Warehouse is a purely administrative change to achieve consistency throughout the TS. It has no adverse safety implication and is, therefore, acceptable.

#### ENVIRONMENTAL CONSIDERATION

These amendments involve changes to the installation or use of facilities' components located within the restricted area as defined in 10 CFR Part 20 and changes in surveillance requirements. The staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite and that there is no significant increase in individual or cumulative occupational exposure. The NRC staff has made a determination that the amendments involve no significant hazards consideration, and there has been no public comment on such finding. Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of these amendments.

#### CONCLUSION

The Commission made a proposed determination that the amendments involve no significant hazards consideration which was published in the Federal Register (52 FR 28624) on July 31, 1987 and consulted with the state of North Carolina. No public comments were received, and the state of North Carolina did not have any comments.

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributors: Darl S. Hood, PD#II-3  
J. Lee, SPLB

Dated: September 9, 1987