

Docket Nos.: 50-369
and 50-370

September 29, 1986

Mr. H. B. Tucker, Vice President
Nuclear Production Department
Duke Power Company
422 South Church Street
Charlotte, North Carolina 28242

Dear Mr. Tucker:

Subject: Issuance of Amendment No. 63 to Facility Operating License NPF-9 and
Amendment No. 44 to Facility Operating License NPF-17 - McGuire
Nuclear Station, Units 1 and 2

The Nuclear Regulatory Commission has issued the enclosed Amendment No. 63 to Facility Operating License NPF-9 and Amendment No. 44 to Facility Operating License NPF-17 for the McGuire Nuclear Station, Units 1 and 2. These amendments consist of changes to the Technical Specifications in response to your letters dated July 15, 1985, March 12, May 14, and July 14, 1986. Other changes requested in these letters will be addressed in a future amendment.

The amendments change Technical Specification Table 3.6-2 related to containment isolation valves. The amendments are effective as of their date of issuance.

A copy of the related safety evaluation supporting Amendment No. 63 to Facility Operating License NPF-9 and Amendment No. 44 to Facility Operating License NPF-17 is enclosed.

Notice of issuance of amendments will be included in the Commission's next bi-weekly Federal Register notice.

Sincerely,

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Darl Hood, Project Manager
PWR Project Directorate #4
Division of PWR Licensing-A

Enclosures:

1. Amendment No. 63 to NPF-9
2. Amendment No. 44 to NPF-17
3. Safety Evaluation

cc w/enclosures: See next page

DISTRIBUTION:

See attached page

MD
PWR#4/DPWR-A
MDuncan/mac
09/15/86

DW
PWR#4/DPWR-A
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09/15/86

DH
PWR#4/DPWR-A
DHood
09/15/86

DW
PWR#4/DPWR-A
BJYoungblood
09/15/86

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PDR ADOCK 05000369
P PDR

Mr. H. B. Tucker
Duke Power Company

McGuire Nuclear Station

cc:

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DATED: September 29, 1986

AMENDMENT NO. 63 TO FACILITY OPERATING LICENSE NPF-9 - McGuire Nuclear Station, Unit 1
AMENDMENT NO. 44 TO FACILITY OPERATING LICENSE NPF-17 - McGuire Nuclear Station, Unit 2

DISTRIBUTION:

Docket File 50-369/370

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Local PDR

PRC System

NSIC

PWR#4 R/F

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MDuncan

DHood

HThompson

OELD

JPartlow

BGrimes

EJordan

LHarmon

WJones

TBarnhart (8)

ACRS (10)

OPA

LFMB

NThompson

EButcher

RGiardina



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

DUKE POWER COMPANY

DOCKET NO. 50-369

McGUIRE NUCLEAR STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 63
License No. NPF-9

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The applications for amendment to the McGuire Nuclear Station, Unit 1 (the facility) Facility Operating License No. NPF-9 filed by the Duke Power Company (the licensee) dated July 15, 1985, March 12, May 14, and July 14, 1986, comply with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's regulations as set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, as amended, the provisions of the Act, and the regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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2. Accordingly, the license is hereby amended by page changes to the Technical Specifications as indicated in the attachments to this license amendment, and Paragraph 2.C.(2) of Facility Operating License No. NPF-9 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 63, are hereby incorporated into the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

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Darl Hood, Project Manager
PWR Project Directorate #4
Division of PWR Licensing-A

Attachment:
Technical Specification
Changes

Date of Issuance: September 29, 1986

PWR#4/DPWR-A
MDuncan:mac
09/15/86

DW
PWR#4/DPWR-A
DWigginton
09/15/86

SSH
PWR#4/DPWR-A
DHood
09/15/86

OGC/BETH
JOHNSON
09/22/86
JH

DW
PWR#4/DPWR-A
BJYoungblood
09/25/86



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

DUKE POWER COMPANY

DOCKET NO. 50-370

McGUIRE NUCLEAR STATION, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 44
License No. NPF-17

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The applications for amendment to the McGuire Nuclear Station, Unit 2 (the facility) Facility Operating License No. NPF-17 filed by the Duke Power Company (the licensee) dated July 15, 1985, March 12, May 14, and July 14, 1986, comply with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's regulations as set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, as amended, the provisions of the Act, and the regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is hereby amended by page changes to the Technical Specifications as indicated in the attachments to this license amendment, and Paragraph 2.C.(2) of Facility Operating License No. NPF-17 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No.44, are hereby incorporated into the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

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Darl Hood, Project Manager
PWR Project Directorate #4
Division of PWR Licensing-A

Attachment:
Technical Specification
Changes

Date of Issuance: September 29, 1986

MD
PWR#4/DPWR-A
MDuncan:mac
09/15/86

ANN
PWR#4/DPWR-A
DWigginton
09/15/86

DSH
PWR#4/DPWR-A
DHood
09/15/86

*See OBC cover sheet
on Commission
NPPA Amendment*
OGC/BETH
09/ /86

AWait
PWR#4/DPWR-A
BJYoungblood
09/25/86

ATTACHMENT TO LICENSE AMENDMENT NO. 63

FACILITY OPERATING LICENSE NO. NPF-9

DOCKET NO. 50-369

AND

TO LICENSE AMENDMENT NO. 44

FACILITY OPERATING LICENSE NO. NPF-17

DOCKET NO. 50-370

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the areas of change. The corresponding overleaf page is also provided to maintain document completeness.

Amended
Page

Overleaf
Page

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3/4 6-25

3/4 6-26

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3/4 6-28

TABLE 3.6-2
CONTAINMENT ISOLATION VALVES

| <u>VALVE NUMBER</u> | <u>FUNCTION</u> | <u>MAXIMUM ISOLATION TIME (SEC)</u> |
|------------------------|-----------------------------------------------------------------------------|-------------------------------------|
| 1. Phase "A" Isolation | | |
| BB-1B# | Steam Generator A Blowdown Containment Outside Isolation | <10 |
| BB-2B# | Steam Generator B Blowdown Containment Outside Isolation | <10 |
| BB-3B# | Steam Generator C Blowdown Containment Outside Isolation | <10 |
| BB-4B# | Steam Generator D Blowdown Containment Outside Isolation | <10 |
| BB-5A# | Steam Generator A Blowdown Containment Inside Isolation | <10 |
| BB-6A# | Steam Generator B Blowdown Containment Inside Isolation | <10 |
| BB-7A# | Steam Generator C Blowdown Containment Inside Isolation | <10 |
| BB-8A# | Steam Generator D Blowdown Containment Inside Isolation | <10 |
| CF-26AB# | Steam Generator D Feedwater Containment Isolation | <5 |
| CF-28AB# | Steam Generator C Feedwater Containment Isolation | <5 |
| CF-30AB# | Steam Generator B Feedwater Containment Isolation | <5 |
| CF-35AB# | Steam Generator A Feedwater Containment Isolation | <5 |
| CF-126B | Steam Generator A Main Feedwater to Auxiliary Feedwater Nozzle Isolation | <10 |
| CF-127B | Steam Generator B Main Feedwater to Auxiliary Feedwater Nozzle Isolation | <10 |
| CF-128B | Steam Generator C Main Feedwater to Auxiliary Feedwater Nozzle Isolation | <10 |
| CF-129B | Steam Generator D Main Feedwater to Auxiliary Feedwater Nozzle Isolation | <10 |
| CF-134A | Steam Generator A Feedwater Containment Isolation Bypass | <10 |
| CF-135B | Steam Generator B Feedwater Containment Isolation Bypass | <10 |
| CF-136A | Steam Generator C Feedwater Containment Isolation Bypass | <10 |
| CF-137A | Steam Generator D Feedwater Containment Isolation Bypass | <10 |
| CF-151B | Auxiliary Nozzle Temper SG A | <10 |

MCGUIRE - UNITS 1 and 2

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Amendment No. 63 (Unit 1)
Amendment No. 44 (Unit 2)

TABLE 3.6-2 (Continued)
CONTAINMENT ISOLATION VALVES

| <u>VALVE NUMBER</u> | <u>FUNCTION</u> | <u>MAXIMUM ISOLATION TIME (SEC)</u> |
|------------------------------------|-------------------------------------------------------------------|-------------------------------------|
| 1. Phase "A" Isolation (continued) | | |
| CF-153A | Auxiliary Nozzle Temper SG B | <10 |
| CF-155B | Auxiliary Nozzle Temper SG C | <10 |
| CF-157B | Auxiliary Nozzle Temper SG D | <10 |
| KC-305B# | Excess Letdown Hx Supply Pent. Isolation Outside | <30 |
| KC-315B# | Excess Letdown Hx Ret. Hdr. Pent. Isolation Outside | <30 |
| KC-320A | NCDT Hx Supply Hdr. Pent. Isolation Outside | <15 |
| KC-332B | NCDT Hx Supply Hdr. Pent. Isolation Inside | <15 |
| KC-333A | NCDT Hx Return Hdr. Pent. Isolation Outside | <15 |
| KC-429B | RB Drain Header Inside Containment Isolation | <15 |
| KC-430A | RB Drain Header Outside Containment Isolation | <15 |
| NB-260B | Reactor Makeup Water Tank to NV System | <15 |
| NC-53B | Nitrogen to Pressurizer Relief Tank Containment Isolation Outside | <10 |
| NC-54A | Nitrogen to Pressurizer Relief Tank Containment Isolation Inside | <10 |
| NC-56B | PRT Makeup | <10 |
| NC-195B | NC Pump Motor Oil Containment Isolation Outside | <15 |
| NC-196A | NC Pump Motor Oil Containment Isolation Inside | <15 |
| NF-228A | Air Handling Units Glycol Supply Containment Isolation Outside | <15 |
| NF-233B | Air Handling Units Glycol Supply Containment Isolation Inside | <15 |
| NF-234A | Air Handling Units Glycol Supply Containment Isolation Outside | <15 |
| NI-47A | Accumulator Nitrogen Supply Outside Containment Isolation | <15 |
| NI-95A | Test HDR Inside Containment Isolation | <10 |

MCGUIRE - UNITS 1 and 2

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Amendment No. 63 (Unit 1)
Amendment No. 44 (Unit 2)

TABLE 3.6-2 (Continued)

CONTAINMENT ISOLATION VALVES

| <u>VALVE NUMBER</u> | <u>FUNCTION</u> | <u>MAXIMUM ISOLATION TIME (SEC)</u> |
|------------------------------------|------------------------------------------------------------|-------------------------------------------------------------|
| 1. Phase "A" Isolation (continued) | | |
| NI-96B | Test HDR Outside Containment Isolation | <10 |
| NI-120B | Safety Injection Pump to Accumulator Fill Line Isolation | <10 |
| NI-255B | Note 1 | <10 |
| NI-258A | | <10 |
| NI-264B | | <10 |
| NI-266A | | <10 |
| NI-267A | | <10 |
| NM-3AC | | Pressurizer Liquid Sample Line Inside Containment Isolation |
| NM-6A | Pressurizer Steam Sample Line Inside Containment Isolation | <15 |
| NM-7B | Pressurizer Sample Header Outside Containment Isolation | <15 |
| NM-22AC | NC Hot Leg #1 Sample Line Inside Containment Isolation | <15 |
| NM-25AC | NC Hot Leg #4 Sample Line Inside Containment Isolation | <15 |
| NM-26B | NC Hot Legs Sample Hdr. Outside Containment Isolation | <15 |
| NM-72B | NI Accumulator A Sample Line Inside Containment Isolation | <15 |
| NM-75B | NI Accumulator B Sample Line Inside Containment Isolation | <15 |
| NM-78B | NI Accumulator C Sample Line Inside Containment Isolation | <15 |
| NM-81B | NI Accumulator D Sample Line Inside Containment Isolation | <15 |
| NM-82A | NI Accumulator Sample Hdr. Outside Containment Isolation | <15 |
| NM-187A# | SG A Upper Shell Sample Containment Isolation Inside | <15 |
| NM-190A# | SG A Blowdown Line Sample Containment Isolation Inside | <15 |
| NM-191B# | SG A Sample Hdr. Containment Isolation Outside | <15 |
| NM-197B# | SG B Upper Shell Sample Containment Isolation Inside | <15 |
| NM-200B# | SG B Blowdown Line Sample Containment Isolation Inside | <15 |
| NM-201A# | SG B Sample Hdr. Containment Isolation Inside | <15 |
| NM-207A# | SG C Upper Shell Sample Containment Isolation Inside | <15 |
| NM-210A# | SG C Blowdown Line Sample Containment Isolation Inside | <15 |
| NM-211B# | SG C Sample Hdr. Containment Isolation Outside | <15 |
| NM-217B# | SG D Upper Shell Sample Containment Isolation Inside | <15 |
| NM-220B# | SG D Blowdown Line Sample Containment Isolation Inside | <15 |

Note 1: Upon the deactivation of the UHI System by removal of related components and piping and modifications to the Cold Leg Accumulators, this specification is no longer applicable.

McGUIRE - UNITS 1 and 2

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Amendment No. 63 (Unit 1)
Amendment No. 44 (Unit 2)

TABLE 3.6-2 (Continued)
CONTAINMENT ISOLATION VALVES

| <u>VALVE NUMBER</u> | <u>FUNCTION</u> | <u>MAXIMUM ISOLATION TIME (SEC)</u> |
|------------------------------------|---------------------------------------------------------------------|-------------------------------------|
| 1. Phase "A" Isolation (continued) | | |
| NM-221A# | SG D Sample Hdr. Containment Isolation Outside | <15 |
| NV-7B# | Letdown Containment Isolation Outside | <10 |
| NV-94A# | NC Pumps Seal Return Containment Isolation Inside | <10 |
| NV-95B# | NC Pumps Seal Return Containment Isolation Outside | <10 |
| NV-457A# | 45 gpm Letdown Orifice Outlet - Containment Isolation | <15 |
| NV-458A# | 75 gpm Letdown Orifice Outlet - Containment Isolation | <15 |
| NV-459A# | High Pressurizer Letdown Orifice Outlet - Containment Isolation | <15 |
| NV-842AC | Standby Makeup Pump Suction Containment Isolation Inside | <15 |
| NV-849AC | Standby Makeup Pump to RCS seals | <15 |
| RF-821A | Containment Isolation Outside (Unit 1) | <15 |
| RF-832A | Containment Isolation Outside (Unit 2) | <15 |
| VB-49B | Breathing Air Containment Isolation | <15 |
| VE-5A | Containment #2 Purge to Annulus Inside Containment Isolation | <15 |
| VE-6B | Containment #2 Purge to Annulus Outside Containment Isolation | <15 |
| VE-10A | Containment #2 Purge Blower Outlet, Containment Isolation (outside) | <15 |
| VI-148B | Instru. Air Upper Containment Outside Isolation | <15 |

MCGUIRE - UNITS 1 and 2

3/4 6-27

Amendment No. 63 (Unit 1)
Amendment No. 44 (Unit 2)

TABLE 3.6-2 (Continued)
CONTAINMENT ISOLATION VALVES

| <u>VALVE NUMBER</u> | <u>FUNCTION</u> | <u>MAXIMUM ISOLATION TIME (SEC)</u> |
|------------------------------------|------------------------------------------------------|-------------------------------------|
| 1. Phase "A" Isolation (continued) | | |
| VI-362A | RB Isolation Valve for VI Supply to Annulum Vent. | <15 |
| VP-1B** | Upper Containment Purge Supply #1 Outside Isolation | <3 |
| VP-2A** | Upper Containment Purge Supply #1 Inside Isolation | <3 |
| VP-3B** | Upper Containment Purge Supply #2 Outside Isolation | <3 |
| VP-4A** | Upper Containment Purge Supply #2 Inside Isolation | <3 |
| VP-6B** | Lower Containment Purge Supply #1 Outside Isolation | <3 |
| VP-7A** | Lower Containment Purge Supply #1 Inside Isolation | <3 |
| VP-8B** | Lower Containment Purge Supply #2 Outside Isolation | <3 |
| VP-9A** | Lower Containment Purge Supply #2 Inside Isolation | <3 |
| VP-10A** | Upper Containment Purge Exhaust #1 Inside Isolation | <3 |
| VP-11B** | Upper Containment Purge Exhaust #1 Outside Isolation | <3 |
| VP-12A** | Upper Containment Purge Exhaust #2 Inside Isolation | <3 |
| VP-13B** | Upper Containment Purge Exhaust #2 Outside Isolation | <3 |
| VP-15A** | Lower Containment Purge Exhaust #1 Inside Isolation | <3 |
| VP-16B** | Lower Containment Purge Exhaust #1 Outside Isolation | <3 |
| VP-17A** | Incore Instru. Room Purge Supply Inside Isolation | <3 |
| VP-18B** | Incore Instru. Room Purge Supply Outside Isolation | <3 |
| VP-19A** | Incore Instru. Room Purge Exhaust Inside Isolation | <3 |
| VP-20B** | Incore Instru. Room Purge Exhaust Outside Isolation | <3 |
| VQ-1A | Containment Air Release Inside Isolation | <3 |
| VQ-2B | Containment Air Release Outside Isolation | <3 |
| VQ-5B | Containment Air Addition Outside Isolation | <3 |
| VQ-6A | Containment Air Addition Inside Isolation | <3 |
| VS-12B | Containment Station Air Outside Isolation | <15 |
| VX-31A | Containment Atmosphere Inside Isolation | <5 |
| VX-33B | Containment Atmosphere Inside Isolation | <5 |



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 63 TO FACILITY OPERATING LICENSE NPF-9
AND AMENDMENT NO. 44 TO FACILITY OPERATING LICENSE NPF-17

DUKE POWER COMPANY

DOCKET NOS. 50-369 AND 50-370

McGUIRE NUCLEAR STATION, UNITS 1 AND 2

INTRODUCTION

By letters dated July 15, 1985 and March 12, 1986, Duke Power Company, the licensee for McGuire Nuclear Station, Units 1 and 2, requested that valve NI-122B be deleted from Table 3.6-2 in the Technical Specifications, and, in addition, proposed a station modification which would remove the containment isolation signal to this valve. The licensee's justification for the change is that valve operation is administratively controlled, and during plant operation the valve is normally closed, which makes the containment isolation signal to this valve unnecessary.

By letters dated May 14, 1986 and July 14, 1986, the licensee also requested, among other things, that seven valve designations listed in Table 3.6-2 be corrected. These changes are to correct an error and do not change plant operation or any safety analysis. The remaining changes proposed by the licensee in the May and July 1986 letters will be handled separately.

EVALUATION

Changes associated with valve NI-122B

The staff has reviewed the licensee's submittals, relevant piping and instrumentation diagrams and the containment isolation provisions on associated fluid lines. Valve NI-122B is inside containment in a 3/4-inch line that connects the reactor hot leg safety injection line at a point between the containment isolation valves, with the check valve test header downstream of the inboard containment isolation valve. Its primary function is to act as a block valve to isolate the safety injection line from the non-safety grade check valve test header.

The staff concludes that the containment isolation signal to valve NI-122B is unnecessary and the plant administrative controls are adequate to ensure that valve NI-122B is closed during plant power operations. Therefore, the requested deletion of valve NI-122B from Table 3.6-2 in the Technical Specifications and the plant modification to McGuire Nuclear Station are acceptable.

Changes associated with Table 3.6-2 valve designations

The licensee has proposed to correct an error in the valve designations as follows:

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Change:

CF-151A to CF-151B
CF-135A to CF-135B
NF-233A to NF-233B
NM-25A to NM25AC
NV-849A to NV-849AC
NM-3A to NM-3AC
NM-22A to NM-22AC

We agree with the licensee that the existing valve designations are in error and are appropriately corrected by the proposed changes. We find that these changes are strictly administrative in nature and do not impact plant safety or operation. Accordingly, these changes are acceptable.

ENVIRONMENTAL CONSIDERATION

These amendments involve changes to the installation or use of facilities' components located within the restricted area as defined in 10 CFR Part 20. The staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite and that there is no significant increase in individual or cumulative occupational exposure. The Commission has previously issued a proposed finding that these amendments involve no significant hazards consideration, and there has been no public comment on such finding. Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of these amendments.

CONCLUSION

The Commission made proposed determinations that the amendments involve no significant hazards consideration which were published in the Federal Register (51 FR 30569 and 30571) on August 27, 1986, and consulted with the state of North Carolina. No public comments were received, and the state of North Carolina did not have any comments.

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributors: R. J. Giardina
D. Wigginton

Dated: September 29, 1986