



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

April 1, 1986

Docket Nos.: 50-369
and 50-370

Mr. H. B. Tucker, Vice President
Nuclear Production Department
Duke Power Company
422 South Church Street
Charlotte, North Carolina 28242

Dear Mr. Tucker:

Subject: Issuance of Amendment No.53 to Facility Operating License NPF-9,
Amendment No.34 to Facility Operating License NPF-17, and
Exemption to 10 CFR Part 50, Appendix J - McGuire Nuclear Station,
Units 1 and 2

The Nuclear Regulatory Commission has issued the enclosed Amendment No.53 to Facility Operating License NPF-9 and Amendment No.34 to Facility Operating License NPF-17 for the McGuire Nuclear Station, Units 1 and 2. These amendments consist of changes to the Technical Specifications in response to your application dated January 21, 1986, and revised March 17, 1986.

The amendments revise a surveillance requirement associated with Technical Specification 3/4.6.1.2d to permit an alternate means of leak testing two containment penetrations associated with the ice condenser refrigeration system.

In connection with this action, the Commission has granted an exemption which allows Type C tests for containment penetration numbers M-372 and M-373, performed without draining the glycol-water mixture from the seats of their diaphragm valves NF-228A, NF-233B and NF-234A, and meeting a zero indicated leakage rate (not including instrument error) for these diaphragm valves, to constitute an acceptable alternate to Type C testing using air or nitrogen as the test medium. Related information in support of your exemption request was submitted in your letters dated September 24, 1985, and February 14, 1986.

We find that granting the proposed exemption from the requirements of Appendix J is authorized by law and will not present an undue risk to the public health and safety, and is consistent with the common defense and security. We further find that special circumstances justify the exemption, namely that application of the regulation in the particular circumstances is not necessary to achieve the underlying purpose of the rule.

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Mr. H. B. Tucker, Vice President

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April 1, 1986


A copy of the related safety evaluation supporting Amendment No.53 to Facility Operating License NPF-9, Amendment No.34 to Facility Operating License NPF-17, and the Exemption from 10 CFR Part 50, Appendix J is enclosed.

Notice of issuance of amendments will be included in the Commission's next bi-weekly Federal Register notice. The Exemption has been forwarded to the Office of the Federal Register for publication.

Sincerely,

Original signed by:

D. Hood



B. J. Youngblood, Director
PWR Project Directorate #4
Division of PWR Licensing-A

Enclosures:

1. Amendment No.53 to NPF-9
2. Amendment No.34 to NPF-17
3. Exemption
4. Safety Evaluation

cc w/enclosures: See next page

DISTRIBUTION:

See attached page

PWR#4/DPWR-A
MDuncan/kab
03/21/86

DSH
PWR#4/DPWR-A
DHood
03/31/86

DSH/m
PWR#4/DPWR-A
BJYoungblood
03/31/86

APR 01 1986

Mr. H. B. Tucker
Duke Power Company

McGuire Nuclear Station

cc:

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

DUKE POWER COMPANY

DOCKET NO. 50-369

McGUIRE NUCLEAR STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 53
License No. NPF-9

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment to the McGuire Nuclear Station, Unit 1 (the facility) Facility Operating License No. NPF-9 filed by the Duke Power Company (the licensee) dated January 21, 1986 and revised March 17, 1986, and further supported by related letters dated September 24, 1985 and February 14, 1986, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's regulations as set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, as amended, the provisions of the Act, and the regulations of the Commission, and the duly authorized exemption from Section III.C.2(a) of Appendix J, 10 CFR Part 50, issued March 31, 1986;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I, except as exempted from compliance with Section III.C.2(a) of Appendix J to 10 CFR Part 50 by exemption issued March 31, 1986;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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2. Accordingly, the license is hereby amended by page changes to the Technical Specifications as indicated in the attachments to this license amendment, and Paragraph 2.C.(2) of Facility Operating License No. NPF-9 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No.53, are hereby incorporated into the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Original signed by:
D. Hood

for

B. J. Youngblood, Director
PWR Project Directorate #4
Division of PWR Licensing-A

Attachment:
Technical Specification
Changes

Date of Issuance: April 1, 1986

MSH
PWR#4/DPWR-A
MDuncan:kab
03/21/86

DSH
PWR#4/DPWR-A
DHood
03/31/86

OELD
Johnson
03/31/86

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PWR#4/DPWR-A
BJYoungblood
03/31/86

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

DUKE POWER COMPANY

DOCKET NO. 50-370

McGUIRE NUCLEAR STATION, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 34
License No. NPF-17

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment to the McGuire Nuclear Station, Unit 2 (the facility) Facility Operating License No. NPF-17 filed by the Duke Power Company (the licensee) dated January 21, 1986, and revised March 17, 1986, and further supported by related letters dated September 24, 1985, and February 14, 1986, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's regulations as set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, as amended, the provisions of the Act, and the regulations of the Commission, and the duly authorized exemption from Section III.C.2(a) of Appendix J, 10 CFR Part 50, issued March 31, 1986;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I, except as exempted from compliance with Section III.C.2(a) of Appendix J, 10 CFR Part 50, by exemption issued March 31, 1986;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is hereby amended by page changes to the Technical Specifications as indicated in the attachments to this license amendment, and Paragraph 2.C.(2) of Facility Operating License No. NPF-17 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No.34, are hereby incorporated into the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Original signed by:

D. Hood

for
B. J. Youngblood, Director
PWR Project Directorate #4
Division of PWR Licensing-A

Attachment:
Technical Specification
Changes

Date of Issuance: April 1, 1986

PWR#4/DPWR-A
MDuncan:kab
03/21/86

DSH
PWR#4/DPWR-A
DHood
03/31/86

OELD
Jonasen
03/21/86

DSH for
PWR#4/DPWR-A
BJYoungblood
03/31/86

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ATTACHMENT TO LICENSE AMENDMENT NO. 53

FACILITY OPERATING LICENSE NO. NPF-9

DOCKET NO. 50-369

AND

TO LICENSE AMENDMENT NO. 34

FACILITY OPERATING LICENSE NO. NPF-17

DOCKET NO. 50-370

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the areas of change.

Amended
Page

3/4 6-3

3/4 6-4

CONTAINMENT SYSTEMS

SURVEILLANCE REQUIREMENTS (Continued)

- a. Three Type A tests (Overall Integrated Containment Leakage Rate) shall be conducted at 40 ± 10 month intervals during shutdown at either P_a , 14.8 psig, or at P_t , 7.4 psig, during each 10-year service period. The third test of each set shall be conducted during the shutdown for the 10-year plant inservice inspection;
- b. If any periodic Type A test fails to meet either $0.75 L_a$ or $0.75 L_t$, the test schedule for subsequent Type A tests shall be reviewed and approved by the Commission. If two consecutive Type A tests fail to meet either $0.75 L_a$ or $0.75 L_t$, a Type A test shall be performed at least every 18 months until two consecutive Type A tests meet either $0.75 L_a$ or $0.75 L_t$ at which time the above test schedule may be resumed;
- c. The accuracy of each Type A test shall be verified by a supplemental test which:
 - 1) Confirms the accuracy of the Type A test by verifying that the difference between supplemental and Type A test data is within $0.25 L_a$, or $0.25 L_t$;
 - 2) Has a duration sufficient to establish accurately the change in leakage rate between the Type A test and the supplemental test; and
 - 3) Requires the quantity of gas injected into the containment or bled from the containment during the supplemental test to be equivalent to at least 25% of the total measured leakage at P_a , 14.8 psig, or P_t , 7.4 psig.
- d. Type B and C tests shall be conducted with gas at P_a , 14.8 psig, at intervals no greater than 24 months except for tests involving:
 - 1) Air locks,
 - 2) Dual-ply bellows assemblies on containment penetrations between the containment building and the annulus, and
 - 3) Purge supply and exhaust isolation valves with resilient material seals.
 - 4) Type C tests performed on containment penetrations M372, M373 without draining the glycol-water mixture from the seats of their diaphragm valves (MF-228A, MF-233B, and MF-234A), if meeting a zero indicated leakage rate (not including instrument error) for the diaphragm valves. These tests may be used in lieu of tests which are otherwise required by Section III.C.2(a) of 10 CFR 50, Appendix J to use air or nitrogen as the test medium. The above required test pressure (P_a) and test interval are not changed by this exception.
- e. Purge supply and exhaust isolation valves with resilient material seals shall be tested and demonstrated OPERABLE by the requirements of Specification 4.6.1.9.3 or 4.6.1.9.4, as applicable;

CONTAINMENT SYSTEMS

SURVEILLANCE REQUIREMENTS (Continued)

- f. The combined bypass leakage rate shall be determined to be less than 0.07 L_a by applicable Type B and C tests at least once per 24 months except^a for penetrations which are not individually testable; penetrations not individually testable shall be determined to have no detectable leakage when tested with soap bubbles while the containment is pressurized to P_a, 14.8 psig, or P_t, 7.4 psig, during each Type A test;
- g. Air locks shall be tested and demonstrated OPERABLE per Specification 4.6.1.3;
- h. The space between each dual-ply bellows assembly on containment penetrations between the containment building and the annulus shall be vented to the annulus during Type A tests. Following completion of each Type A test, the space between each dual-ply bellows assembly shall be subjected to a low pressure test at 3-5 psig to verify no detectable leakage or the dual-ply bellows assembly shall be subjected to a leak test with the pressure on the containment side of the dual-ply bellows assembly at P_a, 14.8 psig, or P_t, 7.4; psig, to verify the leakage to be within the limits of t^t Specification 4.6.1.2f.;
- i. All test leakage rates shall be calculated using observed data converted to absolute values. Error analyses shall be performed to select a balanced Integrated Leakage Measurement System; and
- j. The provisions of Specification 4.0.2 are not applicable.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO.53 TO FACILITY OPERATING LICENSE NPF-9
AMENDMENT NO.34 TO FACILITY OPERATING LICENSE NPF-17

DUKE POWER COMPANY

DOCKET NOS. 50-369 AND 50-370

McGUIRE NUCLEAR STATION, UNITS 1 AND 2

INTRODUCTION

By letter dated January 21, 1986, and revised March 17, 1986, Duke Power Company (the licensee) requested a revision to Technical Specification 4.6.1.2d to permit an alternate means of local leak testing certain mechanical containment penetration diaphragm valves in the ice condenser refrigeration system without draining the glycol mixture from the system. Because the alternate test would use the glycol mixture instead of air or nitrogen as the testing medium, an exemption to Section III C.2(a) of Appendix J to 10 CFR 50 is associated with the request and is addressed concurrently by the NRC staff by separate document. The information submitted by the licensee's letters dated September 24, 1985, and February 14, 1986, regarding the exemption request is also used by the NRC staff for its review of the licensee's request for technical specification changes.

EVALUATION

Surveillance Specification 4.6.1.2 of the facility Technical Specifications requires that primary containment leak rates periodically be demonstrated in conformance with criteria specified in Appendix J of 10 CFR 50. Subparagraph d of this Specification states that Type B and C tests (local tests of penetrations and valves) are to be conducted with gas at a specific pressure and test interval with three indicated exceptions. These amendments add, as a fourth exception to Subparagraph d:

"Type C tests performed on containment penetrations M372, M373 without draining the glycol-water mixture from the seats of their diaphragm valves (NF-228A, NF-233B, and NF-234A), if meeting a zero indicated leakage rate (not including instrument error) for the diaphragm valves. These tests may be used in lieu of tests which are otherwise required by Section III C.2(a) of 10 CFR Part 50, Appendix J to use air or nitrogen as the test medium. The above required test pressure (Pa) and test interval are not changed by this exception."

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Section III C.2.(a) of Appendix J to 10 CFR 50 requires that the leakage rate testing of containment isolation valves be conducted using air or nitrogen as the testing medium. The licensee has requested a technical specification change and an exemption from this requirement for penetrations M-372 and M-373. The local leakage rate test (Type C test) would be performed without draining the glycol from the seats of the diaphragm valves in these penetrations.

The McGuire design includes an ice condenser to suppress the peak accident pressure in the reactor containment building. The ice condenser is refrigerated by recirculating a 50% - 50% mixture of ethylene glycol and water through a series of air handling units located inside the containment building and chiller units located in the auxiliary building. Typically, draining, testing and refilling the system requires 24 to 36 hours of downtime for the ice condenser refrigeration system. This extended downtime potentially diminishes the amount of ice in the baskets. Draining the glycol consumes a significant number of manhours and creates toxic waste (glycol) which requires disposal.

As an alternative to draining approximately 200 gallons of glycol for each McGuire unit as is necessary to perform this test using gas, the licensee has proposed to test three diaphragm valves (NF-228A, NF-233B, and NF-234A) without draining the glycol mixture from the valve seats. The leakage rate acceptance criterion that would be imposed on these diaphragm valves would be zero indicated leakage (not including instrument error). In other words, the display device of the measurement system must read zero. Otherwise, if the leakage rate is greater than zero, the penetration will be fully drained and the valves leak tested in accordance with Appendix J.

Historically, the staff has not accepted the use of a liquid (usually water) in place of air or nitrogen as a testing medium for Type C tests (i.e., local tests of containment isolation valves). This is because it has not been possible to develop a sufficiently conservative yet practically useful, conversion factor for converting water leakage to an equivalent air leakage. However, for the proposed testing, no conversion factor is used; the acceptance criterion of zero leakage of glycol can be assumed to be equivalent to zero leakage of air, or, at worst, possibly a very small leakage of air. This is compared to the acceptance criterion provided by Appendix J for air tests, which is that the total of all local leakage rate tests must not exceed $0.6 L_a$, where L_a is the maximum allowable leakage rate of the containment as a whole. Thus, Appendix J does not impose leakage rate limits on individual valves, but rather on the total leakage rate for all valves and penetrations. Therefore, the staff finds that an acceptance criterion of zero leakage of glycol, applied individually to each of the three valves is at least as conservative as the acceptance criterion of Appendix J. The alternative method of testing with the glycol mixture also accomplishes the same underlying purpose as Appendix J, Section III C.(2)(a) because it provides a conservative assurance of continued leak-tight integrity of the three affected valves. For these reasons, the staff finds that the use of the glycol mixture for the test medium under the conditions stated in revised Specification 4.6.1.2 is acceptable.

If a valve fails the zero leakage criterion, the licensee will proceed to fully drain the penetration and test the valves with air or nitrogen in accordance with Appendix J. This is, of course, acceptable.

ENVIRONMENTAL CONSIDERATION

These amendments involve changes to the installation or use of facility components located within the restricted area as defined in 10 CFR Part 20 and a change in surveillance requirement. The staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite and that there is no significant increase in individual or cumulative occupational exposure. The NRC staff has made a proposed determination that the amendments involve no significant hazards consideration, and there has been no public comment on such findings. Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of these amendments.

CONCLUSION

The Commission made a proposed determination that the amendments involve no significant hazards consideration which was published in the Federal Register (50 FR 6475) on February 24, 1986, and consulted with the state of North Carolina. No public comments were received, and the state of North Carolina did not have any comments.

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributors: Darl S. Hood, PWR Project Directorate #4, PWR-A
J. Pulsipher, Mechanical Engineering Section, PWR-A

Dated: April 1, 1986

UNITED STATES OF AMERICA
NUCLEAR REGULATORY COMMISSION

In the Matter of

DUKE POWER COMPANY
(McGuire Nuclear Station,
Units 1 and 2)Docket Nos. 50-369
and 50-370EXEMPTION PROVIDING FOR CONTAINMENT PENETRATION
TESTING USING GLYCOL

I.

Duke Power Company (the licensee) is the holder of Facility Operating License No. NPF-9 and Facility Operating License No. NPF-17, which authorize the operation of the McGuire Nuclear Station, Units 1 and 2 (the facility) at steady state reactor power levels not in excess of 3411 megawatts thermal. The facility consists of pressurized water reactors located in Mecklenburg County, North Carolina.

II.

Section III C.2(a) of Appendix J to 10 CFR Part 50 which addresses the test pressure to be used in the performance of local leak rate tests for systems and components penetrating primary containment pressure boundary provides: "Valves, unless pressurized with fluid (e.g., water, nitrogen) from a seal system, shall be pressurized with air or nitrogen at a pressure of Pa" (emphasis added). Section II.H of 10 CFR Part 50, Appendix J, defines "Type C Tests" as tests intended to measure containment isolation valve leakage rates. These valves help maintain the leak-tight integrity of the containment at design basis accident conditions.

III.

By letters dated September 24, 1985, and February 14, 1986, the licensee requested an exemption from the requirements of Section III C.2(a) of Appendix

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J to 10 CFR 50, to provide for the use of glycol instead of air or nitrogen as the testing medium for the leakage rate testing of certain containment isolation valves in the Ice Condenser Refrigeration System. The requested exemption is for penetrations M-372 and M-373. The local leakage rate test (Type C test) would be performed without draining the glycol mixture from the seats of the diaphragm valves in these penetrations.

The design of the reactor containment building at McGuire includes an ice condenser to suppress the peak accident pressure. The ice condenser is refrigerated by recirculating a 50% - 50% mixture of ethylene glycol and water through a series of air handling units located inside the containment building and chiller units located in the auxiliary building. The licensee notes that draining, testing, and refilling the system typically requires 24 to 36 hours of downtime for the ice condenser refrigeration system. This extended downtime potentially diminishes the amount of ice in the baskets. The licensee also notes that draining the glycol consumes a significant number of manhours and creates toxic waste (glycol) which has to be disposed.

As an alternative to draining approximately 200 gallons of glycol as is necessary to perform this test in accordance with Appendix J, the licensee has proposed to test three diaphragm valves (NF-228A, NF-233B, and NF-234A) without draining the glycol mixture from the valve seats. The leakage rate acceptance criterion that would be imposed on these diaphragm valves would be zero indicated leakage (not including instrument error). In other words, the display device of the measurement system must read zero. Otherwise, if the leakage rate is greater than zero, the penetration will be fully drained and the valves leak tested in accordance with Appendix J.

Historically, the staff has not accepted the use of a liquid (usually water) in place of air or nitrogen as a testing medium for Type C tests (i.e., local tests of containment isolation valves). This is because it has not been possible to develop a sufficiently conservative, yet practically useful, conversion factor for converting water leakage to an equivalent air leakage. However, for the proposed testing, no conversion factor is used; the acceptance criterion of zero leakage of glycol can be assumed to be equivalent to zero leakage of air, or, at worst, possibly a very small leakage of air. This is compared to the acceptance criterion provided by Appendix J for air tests, which is that the total of all local leakage rate tests must not exceed $0.6 L_a$, where L_a is the maximum allowable leakage rate of the containment as a whole. Thus, Appendix J does not impose leakage rate limits on individual valves, but rather on the total leakage rate for all valves and penetrations. Therefore, the staff finds that an acceptance criterion of zero leakage of glycol, applied individually to each of the three valves, is at least as conservative as the acceptance criterion of Appendix J. For this reason, the staff finds that the requested exemption is acceptable.

If a valve fails the zero leakage criterion, the licensee will proceed to fully drain the penetration and test the valves with air or nitrogen in accordance with Appendix J. This is, of course, acceptable.

IV.

Accordingly, the Commission has determined that, pursuant to 10 CFR 50.12(a)(1) this exemption is authorized by law, will not present an undue risk to the public health and safety, and is consistent with the common defense and security. The Commission further determines that special circumstances, as provided in 10 CFR 50.12(a)(2)(ii), are present justifying

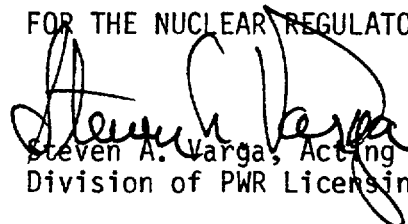
the exemption, namely that application of the regulation in the particular circumstances is not necessary to achieve the underlying purpose of the rule. The underlying purpose of the rule is to require local leak rate testing at periodic intervals of certain types of containment isolation valves to determine whether there has been degradation in the leakage characteristics of these valves which might adversely affect containment integrity. The proposed alternative test method is sufficient to achieve this underlying purpose in that it provides a conservative assurance of continued leak-tight integrity of the three affected valves, NF-228A, NF-233B and NF-234A.

Accordingly, the Commission hereby grants an exemption as described in Section III above from Section III C.2(a) of Appendix J of 10 CFR 50 to the extent that Type C tests for containment penetration numbers M-372 and M-373, performed without draining the glycol-water mixture from the seats of their diaphragm valves NF-228A, NF-233B and NF-234A, and meeting a zero indicated leakage rate (not including instrument error) for these diaphragm valves, shall constitute an acceptable alternate to Type C tests using air or nitrogen as the test medium.

Pursuant to 10 CFR 51.32, the Commission has determined that the granting of this Exemption will have no significant impact on the environment (March 7, 1986, 51 FR 8053).

This Exemption is effective upon issuance.

FOR THE NUCLEAR REGULATORY COMMISSION


Steven A. Varga, Acting Deputy Director
Division of PWR Licensing-A

Dated at Bethesda, Maryland
this 31st day of March 1986.

DATED: April 1, 1986

AMENDMENT NO. 53 TO FACILITY OPERATING LICENSE NPF-9 - McGuire Nuclear Station, Unit 1
AMENDMENT NO. 34 TO FACILITY OPERATING LICENSE NPF-17 - McGuire Nuclear Station, Unit 2

DISTRIBUTION:

Docket File 50-369/370

NRC PDR

Local PDR

PRC System

NSIC

PWR#4 R/F

BJYoungblood R/F

MDuncan

DHood

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JPartlow

BGrimes

EJordan

LHarmon

WJones

TBarnhart (8)

ACRS (10)

OPA

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VBenaroya

JPulsipher