



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

April 7, 1986

Docket No. 50-369  
and 50-370

Mr. H. B. Tucker, Vice President  
Nuclear Production Department  
Duke Power Company  
422 South Church Street  
Charlotte, North Carolina 28242

Dear Mr. Tucker:

Subject: Issuance of Amendment No.54 to Facility Operating License NPF-9  
and Amendment No.35 to Facility Operating License NPF-17 -  
McGuire Nuclear Station, Units 1 and 2

The Nuclear Regulatory Commission has issued the enclosed Amendment No.54 to Facility Operating License NPF-9 and Amendment No.35 to Facility Operating License NPF-17 for the McGuire Nuclear Station, Units 1 and 2. These amendments consists of changes to the Technical Specifications in response to your application dated July 22, 1985, and supplemented September 11, 1985.

The amendments change the Technical Specifications to increase the allowed out-of-service times for Reactor Trip System analog channels. The amendments are effective as of their date of issuance.

The changes authorized by these amendments have been accepted on the basis of the staff's Safety Evaluation Report of February 21, 1985, regarding WCAP-10271, "Evaluation of Surveillance Frequencies and Out-of-Service Times for the Reactor Protection Instrumentation System," and Supplement 1 thereto. One of your proposed changes would modify Action Statement 9 of Specification Table 3.3-1 so as to increase, from two to four hours, the time one channel may be bypassed to allow testing. We note, however, that Action Statement 9 is referenced within the table for reactor trip breakers and automatic trip logic. In our review of the WCAP, we have specifically not approved increased outage times for reactor trip breakers and logic at this time. We understand from Mr. Bob Gill of your Company that this portion of your request was intended to be deleted in the final draft of your present request. Therefore, this portion of your request is not included in these amendments.

These amendments also do not include that portion of your proposed change to Technical Specification Table 3.3-1 which would change the action statement and the minimum channels operable (from 1 to 4) for turbine stop valve closure/ turbine trip. Our discussions with your staff have indicated a need for clarification regarding how the conditions of proposed action statement 6 would be satisfied with an inoperable channel. Accordingly, Mr. Bob Gill has also requested that this portion of your request not be concluded pending further consideration and potential alteration of the proposed change by Duke Power Company.

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PDR ADOCK 05000369  
P PDR


Mr. H. B. Tucker, Vice President - 2 -

April 7, 1986

A copy of the related safety evaluation supporting Amendment No.54 to Facility Operating License NPF-9 and Amendment No.35 to Facility Operating License NPF-17 is enclosed.

Notice of issuance will be included in the Commission's next bi-weekly Federal Register notice.

Sincerely,

  
B. J. Youngblood, Director  
PWR Project Directorate #4  
Division of PWR Licensing-A

Enclosures:

1. Amendment No.54 to NPF-9
2. Amendment No.35 to NPF-17
3. Safety Evaluation

cc w/enclosures: See next page

Mr. H. B. Tucker, Vice President - 2 -

April 7, 1986

A copy of the related safety evaluation supporting Amendment No.54 to Facility Operating License NPF-9 and Amendment No.35 to Facility Operating License NPF-17 is enclosed.

Notice of issuance will be included in the Commission's next bi-weekly Federal Register notice.

Sincerely,

151

B. J. Youngblood, Director  
PWR Project Directorate #4  
Division of PWR Licensing-A

Enclosures:

1. Amendment No. 54 to NPF-9
2. Amendment No. 35 to NPF-17
3. Safety Evaluation

cc w/enclosures: See next page

DISTRIBUTION:

See attached page

DJH  
PWR#4/DPWR-A  
DHood/kab  
03/13/86

PWR#4/DPWR-A  
MDuncan  
03/12/86

DJH  
PWR#4/DPWR-A  
BJYoungblood  
03/13/86  
4/2/86

Mr. H. B. Tucker  
Duke Power Company

McGuire Nuclear Station

cc:

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

DUKE POWER COMPANY

DOCKET NO. 50-369

McGUIRE NUCLEAR STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 54  
License No. NPF-9

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment to the McGuire Nuclear Station, Unit 1 (the facility) Facility Operating License No. NPF-9 filed by the Duke Power Company (the licensee) dated July 22, 1985, and supplemented September 11, 1985, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's regulations as set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, as amended, the provisions of the Act, and the regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

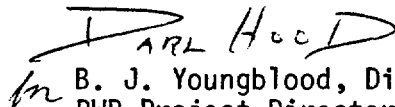
2. Accordingly, the license is hereby amended by page changes to the Technical Specifications as indicated in the attachments to this license amendment, and Paragraph 2.C.(2) of Facility Operating License No. NPF-9 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No.54, are hereby incorporated into the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

  
B. J. Youngblood, Director  
PWR Project Directorate #4  
Division of PWR Licensing-A

Attachment:  
Technical Specification  
Changes

Date of Issuance: April 7, 1986

2. Accordingly, the license is hereby amended by page changes to the Technical Specifications as indicated in the attachments to this license amendment, and Paragraph 2.C.(2) of Facility Operating License No. NPF-9 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 54, are hereby incorporated into the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

151

B. J. Youngblood, Director  
PWR Project Directorate #4  
Division of PWR Licensing-A

Attachment:  
Technical Specification  
Changes

Date of Issuance: April 7, 1986

PWR#4/DPWR-A  
MDuncan:kab  
03/12/86

DSIT  
PWR#4/DPWR-A  
DHood  
03/12/86

OELD  
GE Johnson  
03/19/86  
w/ changes  
what to  
add to SEC  
Dove  
DSIT

DSIT  
PWR#4/DPWR-A  
BJYoungblood  
03/12/86  
4/2/86



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

DUKE POWER COMPANY

DOCKET NO. 50-370

McGUIRE NUCLEAR STATION, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 35  
License No. NPF-17

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment to the McGuire Nuclear Station, Unit 2 (the facility) Facility Operating License No. NPF-17 filed by the Duke Power Company (the licensee) dated July 22, 1985, as supplemented September 11, 1985, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's regulations as set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, as amended, the provisions of the Act, and the regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.



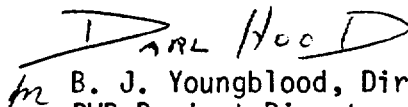
2. Accordingly, the license is hereby amended by page changes to the Technical Specifications as indicated in the attachments to this license amendment, and Paragraph 2.C.(2) of Facility Operating License No. NPF-17 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No.35, are hereby incorporated into the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

  
B. J. Youngblood, Director  
PWR Project Directorate #4  
Division of PWR Licensing-A

Attachment:  
Technical Specification  
Changes

Date of Issuance: April 7, 1986

07 APR 1986

- 2. Accordingly, the license is hereby amended by page changes to the Technical Specifications as indicated in the attachments to this license amendment, and Paragraph 2.C.(2) of Facility Operating License No. NPF-17 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 35, are hereby incorporated into the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

- 3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

151

B. J. Youngblood, Director  
 PWR Project Directorate #4  
 Division of PWR Licensing-A

Attachment:  
 Technical Specification  
 Changes

Date of Issuance: April 7, 1986

PWR#4/DPWR-A  
 MDuncan:kab  
 03/12/86

DSH  
 PWR#4/DPWR-A  
 DHood  
 03/13/86

OELD  
 03/19/86  
 changes added to letter & form  
 Done DSH

DSH for  
 PWR#4/DPWR-A  
 BJYoungblood  
 03/13/86  
 4/2/86

ATTACHMENT TO LICENSE AMENDMENT NO. 54

FACILITY OPERATING LICENSE NO. NPF-9

DOCKET NO. 50-369

AND

TO LICENSE AMENDMENT NO. 35

FACILITY OPERATING LICENSE NO. NPF-17

DOCKET NO. 50-370

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the areas of change. The corresponding overleaf pages are also provided to maintain document completeness.

Amended  
Page

Overleaf  
Page

3/4 3-3

3/4 3-4

3/4 3-6

3/4 3-7

B 3/4 3-1

B 3/4 3-1a (new page)

3/4 3-5

B 3/4 3-2

TABLE 3.3-1 (Continued)

REACTOR TRIP SYSTEM INSTRUMENTATION

| <u>FUNCTIONAL UNIT</u>                   | <u>TOTAL NO. OF CHANNELS</u> | <u>CHANNELS TO TRIP</u>                | <u>MINIMUM CHANNELS OPERABLE</u>     | <u>APPLICABLE MODES</u> | <u>ACTION</u>           |
|--|------------------------------|--|--------------------------------------|-------------------------|-------------------------|
| 8. Overpower WT                          |                              |  |                                      |                         |                         |
| Four Loop Operation                      | 4                            | 2                                      | 3                                    | 1, 2                    | 6 <sup>#</sup>          |
| Three Loop Operation                     | (**)                         | (**)                                   | (**)                                 | (**)                    | (**)                    |
| 9. Pressurizer Pressure-Low              | 4                            | 2                                      | 3                                    | 1                       | 6 <sup>#</sup><br>(***) |
| 10. Pressurizer Pressure--High           | 4                            | 2                                      | 3                                    | 1, 2                    | 6 <sup>#</sup><br>(***) |
| 11. Pressurizer Water Level--High        | 3                            | 2                                      | 2                                    | 1                       | 6 <sup>#</sup>          |
| 12. Low Reactor Coolant Flow             |                              |  |                                      |                         |                         |
| a. Single Loop (Above P-8)               | 3/loop                       | 2/loop in any operating loop           | 2/loop in each operating loop        | 1                       | 6 <sup>#</sup>          |
| b. Two Loops (Above P-7 and below P-8)   | 3/loop                       | 2/loop in two operating loops          | 2/loop each operating loop           | 1                       | 6 <sup>#</sup>          |
| 13. Steam Generator Water Level--Low-Low | 4/stm. gen.                  | 2/stm. gen. in any operating stm. gen. | 3/stm. gen. each operating stm. gen. | 1, 2                    | 6 <sup>#</sup><br>(***) |

McGUIRE - UNITS 1 and 2

3/4 3-3

Amendment No54(Unit 1)  
Amendment No35(Unit 2)

TABLE 3.3-1 (Continued)

REACTOR TRIP SYSTEM INSTRUMENTATION

| <u>FUNCTIONAL UNIT</u>                               | <u>TOTAL NO. OF CHANNELS</u> | <u>CHANNELS TO TRIP</u> | <u>MINIMUM CHANNELS OPERABLE</u> | <u>APPLICABLE MODES</u> | <u>ACTION</u> |
|--|------------------------------|-------------------------|----------------------------------|-------------------------|---------------|
| 14. Undervoltage-Reactor Coolant Pumps (above P-7)   | 4-1/bus                      | 2                       | 3                                | 1                       | 6#            |
| 15. Underfrequency-Reactor Coolant Pumps (above P-7) | 4-1/bus                      | 2                       | 3                                | 1                       | 6#            |
| 16. Turbine Trip                                     |                              |                         |                                  |                         |               |
| a. Low Fluid Oil Pressure                            | 3                            | 2                       | 2                                | 1                       | 6#            |
| b. Turbine Stop Valve Closure                        | 4                            | 4                       | 1                                | 1                       | 11#           |
| 17. Safety Injection Input from ESF                  | 2                            | 1                       | 2                                | 1, 2                    | 9             |
| 18. Reactor Trip System Interlocks                   |                              |                         |                                  |                         |               |
| a. Intermediate Range Neutron Flux, P-6              | 2                            | 1                       | 2                                | 2##                     | 8             |
| b. Low Power Reactor Trips Block, P-7                |                              |                         |                                  |                         |               |
| P-10 Input   | 4                            | 2                       | 3                                | 1                       | 8             |
| or   |                              |                         |                                  |                         |               |
| P-13 Input   | 2                            | 1                       | 2                                | 1                       | 8             |
| c. Power Range Neutron Flux, P-8                     | 4                            | 2                       | 3                                | 1                       | 8             |
| d. Low Setpoint Power Range Neutron Flux, P-10       | 4                            | 2                       | 3                                | 1, 2                    | 8             |
| e. Turbine Impulse Chamber Pressure, P-13            | 2                            | 1                       | 2                                | 1                       | 8             |

McGUIRE - UNITS 1 and 2

3/4 3-4

Amendment No. 54 (Unit 1)  
Amendment No. 35 (Unit 2)

TABLE 3.3-1 (Continued)

REACTOR TRIP SYSTEM INSTRUMENTATION

| <u>FUNCTIONAL UNIT</u>                 | <u>TOTAL NO. OF CHANNELS</u> | <u>CHANNELS TO TRIP</u> | <u>MINIMUM CHANNELS OPERABLE</u> | <u>APPLICABLE MODES</u> | <u>ACTION</u> |
|--|------------------------------|-------------------------|----------------------------------|-------------------------|---------------|
| 19. Reactor Trip Breakers              | 2                            | 1                       | 2                                | 1, 2                    | 9             |
|  | 2                            | 1                       | 2                                | 3*, 4*, 5*              | 10            |
| 20. Automatic Trip and Interlock Logic | 2                            | 1                       | 2                                | 1, 2                    | 9             |
|  | 2                            | 1                       | 2                                | 3*, 4*, 5*              | 10            |

TABLE 3.3-1 (Continued)

TABLE NOTATION

- \* With the Reactor Trip System breakers in the closed position, the Control Rod Drive System capable of rod withdrawal.
- \*\* Values left blank pending NRC approval of three loop operation.
- \*\*\* Comply with the provisions of Specification 3.3.2 for any portion of the channel required to be OPERABLE by Specification 3.3.2.
- # The provisions of Specification 3.0.4 are not applicable.
- ## Below the P-6 (Intermediate Range Neutron Flux Interlock) Setpoint.
- ### Below the P-10 (Low Setpoint Power Range Neutron Flux Interlock) Setpoint.

ACTION STATEMENTS

- ACTION 1 - With the number of OPERABLE channels one less than the Minimum Channels OPERABLE requirement, restore the inoperable channel to OPERABLE status within 48 hours or be in HOT STANDBY within the next 6 hours.
  
- ACTION 2 - With the number of OPERABLE channels one less than the Total Number of Channels, STARTUP and/or POWER OPERATION may proceed provided the following conditions are satisfied:
  - a. The inoperable channel is placed in the tripped condition within 6 hours,
  - b. The Minimum Channels OPERABLE requirement is met; however, the inoperable channel may be bypassed for up to 4 hours for surveillance testing of other channels per Specification 4.3.1.1, and
  - c. Either, THERMAL POWER is restricted to less than or equal to 75% of RATED THERMAL POWER and the Power Range Neutron Flux Trip Setpoint is reduced to less than or equal to 85% of RATED THERMAL POWER within 4 hours; or, the QUADRANT POWER TILT RATIO is monitored at least once per 12 hours per Specification 4.2.4.2.

TABLE 3.3-1 (Continued)

ACTION STATEMENTS (Continued)

- ACTION 3 - With the number of channels OPERABLE one less than the Minimum Channels OPERABLE requirement and with the THERMAL POWER level:
- a. Below the P-6 (Intermediate Range Neutron Flux Interlock) Setpoint, restore the inoperable channel to OPERABLE status prior to increasing THERMAL POWER above the P-6 Setpoint, and
  - b. Above the P-6 (Intermediate Range Neutron Flux Interlock) Setpoint but below 10% of RATED THERMAL POWER, restore the inoperable channel to OPERABLE status prior to increasing THERMAL POWER above 10% of RATED THERMAL POWER.
- ACTION 4 - With the number of OPERABLE channels one less than the Minimum Channels OPERABLE requirement suspend all operations involving positive reactivity changes.
- ACTION 5 - With the number of OPERABLE channels one less than the Minimum Channels OPERABLE requirement, verify compliance with the SHUTDOWN MARGIN requirements of Specification 3.1.1.1 or 3.1.1.2, as applicable, within 1 hour and at least once per 12 hours thereafter.
- ACTION 6 - With the number of OPERABLE channels one less than the Total Number of Channels, STARTUP and/or POWER OPERATION may proceed provided the following conditions are satisfied:
- a. The inoperable channel is placed in the tripped condition within 6 hours, and
  - b. The Minimum Channels OPERABLE requirement is met; however, the inoperable channel may be bypassed for up to 4 hours for surveillance testing of other channels per Specification 4.3.1.1 and Specification 4.3.2.1.
- ACTION 7- Delete
- ACTION 8 - With less than the Minimum Number of Channels OPERABLE, within 1 hour determine by observation of the associated permissive annunciator window(s) that the interlock is in its required state for the existing plant condition, or apply Specification 3.0.3.



### 3/4.3 INSTRUMENTATION

#### BASES

#### 3/4.3.1 and 3/4.3.2 REACTOR TRIP AND ENGINEERED SAFETY FEATURES ACTUATION SYSTEM INSTRUMENTATION

The OPERABILITY of the Reactor Trip and Engineered Safety Features Actuation System instrumentation and interlocks ensure that: (1) the associated ACTION and/or Reactor trip will be initiated when the parameter monitored by each channel or combination thereof reaches its Setpoint, (2) the specified coincidence logic and sufficient redundancy is maintained to permit a channel to be out-of-service for testing or maintenance consistent with maintaining an appropriate level of reliability of the Reactor Protection and Engineered Safety Features Instrumentation and (3) sufficient system functions capability is available from diverse parameters.

The OPERABILITY of these systems is required to provide the overall reliability, redundancy, and diversity assumed available in the facility design for the protection and mitigation of accident and transient conditions. The integrated operation of each of these systems is consistent with the assumptions used in the accident analyses. The Surveillance Requirements specified for these systems ensure that the overall system functional capability is maintained comparable to the original design standards. The periodic surveillance tests performed at the minimum frequencies are sufficient to demonstrate this capability.

Specified surveillance intervals and surveillance and maintenance outage times have been determined in accordance with WCAP-10271, "Evaluation of Surveillance Frequencies and Out of Service Times for the Reactor Protection Instrumentation System," and supplements to that report. Surveillance intervals and out of service times were determined based on maintaining an appropriate level of reliability of the Reactor Protection System and Engineered Safety Features instrumentation. (Implementation of quarterly testing of RTS is being postponed until after approval of a similar testing interval for ESFAS.) The NRC Safety Evaluation Report for WCAP-10271 was provided in a letter dated February 21, 1985 from C. O. Thomas (NRC) to J. J. Sheppard (WOG-CP&L).

The measurement of response time at the specified frequencies provides assurance that the Reactor trip and the Engineered Safety Feature actuation associated with each channel is completed within the time limit assumed in the accident analyses. No credit was taken in the analyses for those channels with response times indicated as not applicable. Response time may be demonstrated by any series of sequential, overlapping, or total channel test measurements provided that such tests demonstrate the total channel response time as defined. Sensor response time verification may be demonstrated by either: (1) in-place, onsite, or offsite test measurements, or (2) utilizing replacement sensors with certified response times.

The Engineered Safety Features Actuation System senses selected plant parameters and determines whether or not predetermined limits are being exceeded. If they are, the signals are combined into logic matrices sensitive to combinations indicative of various accidents, events, and transients. Once the required logic combination is completed, the system sends actuation signals to those Engineered Safety Features components whose aggregate function best serves the requirements of the condition. As an example, the

## INSTRUMENTATION

### BASES

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#### 3/4.3.1 and 3/4.3.2 REACTOR TRIP AND ENGINEERED SAFETY FEATURES ACTUATION SYSTEM INSTRUMENTATION (Continued)

following actions may be initiated by the Engineered Safety Features Actuation System to mitigate the consequences of a steam line break or loss-of-coolant accident: (1) Safety Injection pumps start and automatic valves position, (2) Reactor trip, (3) feedwater isolation, (4) startup of the emergency diesel generators, (5) containment spray pumps start and automatic valves position, (6) containment isolation, (7) steam line isolation, (8) Turbine trip, (9) auxiliary feedwater pumps start and automatic valves position, and (10) nuclear service water pumps start and automatic valves position.

## INSTRUMENTATION

### BASES

#### REACTOR PROTECTION SYSTEM AND ENGINEERED SAFETY FEATURES ACTUATION SYSTEM INSTRUMENTATION (Continued)

The Engineered Safety Features Actuation System interlocks perform the following functions:

- P-4 Reactor tripped - Actuates Turbine trip, closes main feedwater valves on  $T_{avg}$  below Setpoint, prevents the opening of the main feedwater valves which were closed by a Safety Injection or High Steam Generator Water Level signal, allows Safety Injection block so that components can be reset or tripped.  
Reactor not tripped - prevents manual block of Safety Injection.
- P-11 Defeats the manual block of Safety Injection actuation on low pressurizer pressure and low steamline pressure and defeats steamline isolation on negative steamline pressure rate. Defeats the manual block of the motor-driven auxiliary feedwater pumps on trip of main feedwater pumps and low-low steam generator water level.
- P-12 On increasing reactor coolant loop temperature, P-12 automatically provides an arming signal to the steam dump system. On decreasing reactor coolant loop temperature, P-12 automatically removes the arming signal from the steam dump system.
- P-14 On increasing steam generator level, P-14 automatically trips all feedwater isolation valves and inhibits feedwater control valve modulation.

#### 3/4.3.3 MONITORING INSTRUMENTATION

##### 3/4.3.3.1 RADIATION MONITORING FOR PLANT OPERATIONS

The OPERABILITY of the radiation monitoring instrumentation for plant operations ensures that: (1) the associated action will be initiated when the radiation level monitored by each channel or combination thereof reaches its Setpoint, (2) the specified coincidence logic is maintained, and (3) sufficient redundancy is maintained to permit a channel to be out-of-service for testing or maintenance. The radiation monitors for plant operations senses radiation levels in selected plant systems and locations and determines whether or not predetermined limits are being exceeded. If they are, the signals are combined into logic matrices sensitive to combinations indicative of various accidents and abnormal conditions. Once the required logic combination is completed, the system sends actuation signals to initiate alarms or automatic isolation action and actuation of Emergency Exhaust or Ventilation Systems.

##### 3/4.3.3.2 MOVABLE INCORE DETECTORS

The OPERABILITY of the movable incore detectors with the specified minimum complement of equipment ensures that the measurements obtained from use of this system accurately represent the spatial neutron flux distribution



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
RELATED TO AMENDMENT NO.54 TO FACILITY OPERATING LICENSE NPF-9  
AND AMENDMENT NO.35 TO FACILITY OPERATING LICENSE NPF-17  
DUKE POWER COMPANY  
McGUIRE NUCLEAR STATION, UNITS 1 AND 2

INTRODUCTION

On February 21, 1985, the NRC staff issued its Safety Evaluation Report (SER) on the Westinghouse Technical Specification Optimization Program for increased surveillance intervals and out-of-service times for testing and maintenance of the Reactor Trip System (RTS). The Optimization Program proposal was set forth in WCAP-10271, "Evaluation of Surveillance Frequencies and Out-of-Service Times for Reactor Protection Instrumentation System," and Supplement 1 thereto.

By letter dated July 22, 1985, Duke Power Company (the licensee) proposed several changes to the McGuire Technical Specifications based on the Optimization Program. One of these proposed changes, which sought to allow RTS analog channel testing in a bypassed condition instead of a tripped condition, was withdrawn by a subsequent letter from the licensee, dated September 11, 1985. The remaining proposed changes have been evaluated by the NRC staff. Those which we find acceptable are the subject of this evaluation report and are included in these amendments.

EVALUATION

These amendments increase, from one to six hours, the time during which an inoperable RTS analog channel may be maintained in an untripped condition (See Table 3.3-1, Action Statements 2a and 6a in the revised Technical Specifications). The time an inoperable RTS analog channel may be bypassed to allow testing of another channel in the same function is increased from two to four hours (See Table 3.3-1, Action Statements 2b and 6b in the revised Technical Specifications). A cautionary note is added to the action statements referencing the more stringent requirements for Engineering Safety Feature Actuation System (ESFAS) channels for RTS analog channels common to ESFAS (See Table 3.3-1, Functional Units 9, 10, and 13 and Note \*\*\*). We find these changes to be consistent with those which we reviewed and accepted for the Optimization Program proposal. Therefore, the staff finds these changes acceptable on the bases set forth in the staff's February 21, 1985, SER.

The amendments delete Action Statement 7 and substitute a reference to Action Statement 6 for those reactor trip channels which referenced Action Statement 7 (See Table 3.3-1, Functional Units 11, 12 and 16a in the revised Technical Specifications). This change, is discussed in Supplement 1 to WCAP-10271, and results from the similarity and the intent of Action Statements 6 and 7. Further, the staff in a July 24, 1985, letter from Harold R. Denton to L. B. Butterfield identified this to be a specific acceptable revision in the model technical specifications provided to the Westinghouse Owners Group (WOG). On this basis, the staff finds this change acceptable as proposed by the licensee.

The amendments revise Bases Sections 3/4.3.1 and 3/4.3.2 to include a discussion of WCAP-10271. We find the revised Bases, as proposed in the licensee's letter of September 11, 1985, to be equivalent to those in the "WOG Guidance for Preparation of Submittals Requesting Revisions to RPS Technical Specifications," Revision 1, dated September 3, 1985, and that the revised Bases encompass the specific comments in the staff's July 24, 1985, letter. Therefore, the revised Bases are acceptable.

#### ENVIRONMENTAL CONSIDERATION

The amendments involve a change in use of facility components located within the restricted area as defined in 10 CFR Part 20 and a change in surveillance requirements. The staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite and that there is no significant increase in individual or cumulative occupational exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration, and there have been no public comments on such findings. Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

#### CONCLUSION

The Commission made a proposed determination that the amendments involve no significant hazards consideration which was published in the Federal Register (50 FR 51622) on December 18, 1985, and consulted with the state of North Carolina. No public comments were received, and the state of North Carolina did not have any comments.

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

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Dated: April 4, 1986

DATED: April 7, 1986

AMENDMENT NO.54 TO FACILITY OPERATING LICENSE NPF-9 - McGuire Nuclear Station, Unit 1  
AMENDMENT NO.35 TO FACILITY OPERATING LICENSE NPF-17 - McGuire Nuclear Station, Unit 2

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