

#### UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

February 28, 1985

Docket Nos: 50-369 and 50-370

Mr. H. B. Tucker, Vice President Nuclear Production Department Duke Power Company 422 South Church Street Charlotte, North Carolina 28242

Dear Mr. Tucker:

Subject: Issuance of Amendment No. 40 to Facility Operating License NPF-9 and Amendment No. 21 to Facility Operating License NPF-17 - McGuire Nuclear Station, Units 1 and 2

The Nuclear Regulatory Commission has issued the enclosed Amendment No. 40 to Facility Operating License NPF-9 and Amendment No. 21 to Facility Operating License NPF-17 for the McGuire Nuclear Station, Units 1 and 2. These amendments are in response to your application dated July 31, 1984.

The amendments change the Technical Specifications to expand Tables 3.3-10 and 4.3-7 concerning accident monitoring instrumentation and surveillance requirements to include the recently installed Reactor Vessel Level Instrumentation System and to include both channels of the Subcooling Margin Monitoring System.

A copy of the related safety evaluation supporting Amendment No. 40 to Facility Operating License NPF-9 and Amendment No. 21 to Facility Operating License NPF-17 is enclosed.

Sincerely,

Elinor G. Adensam, Chief Licensing Branch No. 4 Division of Licensing

Enclosures: 1. Amendment No. 40to NPF-9

- 2. Amendment No. 21to NPF-17
- 3. Safety Evaluation

cc w/encl: See next page

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#### McGUIRE (Amendment & Order)

Mr. H. B. Tucker, Vice President Nuclear Production Department Duke Power Company 422 South Church Street Charlotte, North Carolina 28242

cc: Mr. A. Carr Duke Power Company P. O. Box 33189 422 South Church Street Charlotte, North Carolina 28242

> Mr. F. J. Twogood Power Systems Division Westinghouse Electric Corp. P. O. Box 355 Pittsburgh, Pennsylvania 15230

Mr. Robert Gill Duke Power Company Nuclear Production Department P. O. Box 33189 Charlotte, North Carolina 28242

J. Michael McGarry, III, Esq. Bishop, Liberman, Cook, Purcell and Reynolds 1200 Seventeenth Street, N.W. Washington, D. C. 20036

Mr. Wm. Orders Senior Resident Inspector c/o U.S. Nuclear Regulatory Commission Route 4, Box 529 Hunterville, North Carolina 28078

James P. O'Reilly, Regional Admin. U.S. Nuclear Regulatory Commission, Region II 101 Marietta Street, N.W., Suite 2900 Atlanta, Georgia 30323

R. S. Howard
Operating Plants Projects
Regional Manager
Westinghouse Electric Corporation - R&D 701
P. O. Box 2728
Pittsburgh, Pennsylvania 15230

Dr. John M. Barry Department of Environmental Health Mecklenburg County 1200 Blythe Boulevard Charlotte, North Carolina 28203

Attorney General Department of Justice Justice Building Raleigh, North Carolina 27602

County Manager of Mecklenburg County 720 East Fourth Street Charlotte, North Carolina 28202

EIS Coordinator U.S. Environmental Protection Agency Region IV Office 345 Courtland Street, N.E. Atlanta, Georgia 30365

Chairman, North Carolina Utilities Commission 430 North Salisbury Street Dobbs,Building Raleigh, North Carolina 27602

Mr. Dayne H. Brown, Chief Radiation Protection Branch Division of Facility Services Department of Human Resources P.O. Box 12200 Raleigh, North Carolina 27605



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#### UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

# DUKE POWER COMPANY

# DOCKET NO. 50-369

### MCGUIRE NUCLEAR STATION, UNIT 1

### AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 40 License No. NPF-9

- 1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment to the McGuire Nuclear Station, Unit 1 (the facility) Facility Operating License No. NPF-9 filed by the Duke Power Company (licensee) dated July 31, 1984, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's regulations as set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, as amended, the provisions of the Act, and the regulations of the Commission;
  - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
  - D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public;
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
- Accordingly, the license is hereby amended by page changes to the Technical Specifications as indicated in the attachments to this license amendment and paragraph 2.C.(2) of Facility Operating License No. NPF-9 is hereby amended to read as follows:

# (2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 40, are hereby incorporated into this license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

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Elinor G. Adensam, Chief Licensing Branch No. 4 Division of Licensing

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Attachment: Technical Specification Changes

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Date of Issuance: February 28, 1985



#### UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

DUKE POWER COMPANY

### DOCKET NO. 50-370

## McGUIRE NUCLEAR STATION, UNIT 2

### AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 21 License No. NPF-17

- 1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment to the McGuire Nuclear Station, Unit 2 (the facility) Facility Operating License No. NPF-17 filed by the Duke Power Company (licensee) dated July 31, 1984, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's regulations as set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, as amended, the provisions of the Act, and the regulations of the Commission;
  - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
  - D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public;
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
- 2. Accordingly, the license is hereby amended by page changes to the Technical Specifications as indicated in the attachments to this license amendment and paragraph 2.C.(2) of Facility Operating License No. NPF-17 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 21, are hereby incorporated into this license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Elina D. allenson

Elinor G. Adensam, Chief Licensing Branch No. 4 Division of Licensing

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Attachment: Technical Specification Changes

Date of Issuance: February 28, 1985

ATTACHMENT TO LICENSE AMENDMENT NO. 40

### FACILITY OPERATING LICENSE NO. NPF-9

# DOCKET NO. 50-369

# AND

### TO LICENSE AMENDMENT NO. 21

### FACILITY OPERATING LICENSE NO. NPF-17

# DOCKET NO. 50-370

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain a vertical line indicating the area of change. The corresponding overleaf page is also provided to maintain document completeness.

Amended		Overleaf Page	
3/4 3-56		_3/4 3-55	
3/4 3-57			

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#### INSTRUMENTATION

ACCIDENT MONITORING INSTRUMENTATION

#### LIMITING CONDITION FOR OPERATION

3.3.3.6 The accident monitoring instrumentation channels shown in Table 3.3-10 shall be OPERABLE.

APPLICABILITY: MODES 1, 2, and 3.

ACTION:

- a. With the number of OPERABLE accident monitoring instrumentation channels less than the Required Number of Channels shown in Table 3.3-10, restore the inoperable channel(s) to OPERABLE status within 7 days, or be in at least HOT SHUTDOWN within the next 12 hours.
- b. With the number of OPERABLE accident monitoring instrumentation channels less than the Minimum Channels OPERABLE requirements of Table 3.3-10, restore the inoperable channel(s) to OPERABLE status within 48 hours or be in at least HOT SHUTDOWN within the next 12 hours.

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c. The provisions of Specification 3.0.4 are not applicable.

#### SURVEILLANCE REQUIREMENTS

4.3.3.6 Each accident monitoring instrumentation channel shall be demonstrated OPERABLE by performance of the CHANNEL CHECK and CHANNEL CALIBRATION operations at the frequencies shown in Table 4.3-7.

ACCIDENT MONITORING INSTRUMENTATION McGUIRE REQUIRED MINIMUM NO. OF **CHANNELS** ł INSTRUMENT CHANNELS **OPERABLE** UNITS 2 1. **Containment Pressure**  Reactor Coolant Temperature - T<sub>HOT</sub> and T<sub>COLD</sub> (Wide Range)
 Reactor Coolant Pressure - Wide Range 2 ш Pressurizer Water Level 4. and 1/steam generator 2/steam generator 5. Steam Line Pressure Steam Generator Water Level - Narrow Range 2/steam generator 1/steam generator 6. N 7. Refueling Water Storage Tank Water Level Auxiliary Feedater Flow Rate 2/steam generator 1/steam generator 8. Reactor Coolant System Subcooling Margin Monitor 2\*\*\* 9. l/valve PORV Position Indicator\* 2/valve 10. 11. PORV Block Valve Position Indicator\*\* 1/valve 1/valve 12. Safety Valve Position Indicator 2/valve 1/valve 13. Containment Water Level (Wide Range) 1 3/4 In Core Thermocouples 14. 4/core quadrant 2/core guadrant 15. Unit Vent - High Range Noble Gas Monitor 3-56 (High-High Range - EMF-36) Steam Relief - High Range Monitor 1/steam line 1/steam line 16. (Unit 1 - EMF-24, 25, 26, 27) (Unit 2 - EMF-10, 11, 12, 13) 17. Containment Atmosphere - High Range Monitor 1 1 (EMF-51a or 51b) Reactor Vessel Level Instrumentation\*\*\*\* 18. Wide Range 2 a. 2 Narrow Range b.

TABLE 3.3-10

\*Not applicable if the associated block valve is in the closed position. \*\*Not applicable if the associated block valve is in the closed position and power is removed.

\*\*\*Only one channel per unit is required until the end of the first refueling outage following 1/86 for each unit. \*\*\*\*Not applicable until the beginning of Cycle 4 for Unit 1 and Cycle 3 for Unit 2.

Amendment Amendment No. 21 (Unit 40 (Unit

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INST	RUMENT	CHANNEL CHECK	CHANNEL CALIBRATION
1.	Containment Pressure	M	R
2.	Reactor Coolant Temperature - $T_{HOT}$ and $T_{COLD}$ (Wide Range)	М	R
3.	Reactor Coolant Pressure - Wide Range	М	R
4.	Pressurizer Water Level	М	R
5.	Steam Line Pressure	М	R
6.	Steam Generator Water Level - Narrow Range	Μ	R
7.	Refueling Water Storage Tank Water Level	М	R
8.	Auxiliary Feedwater Flow Rate	м	R
9.	Reactor Coolant System Subcooling Margin Monitor	м	R
10.	PORV Position Indicator	м	R
11.	PORV Block Valve Position Indicator	М	R
12.	Safety Valve Position Indicator	М	R
13.	Containment Water Level (Wide Range)	М	R
14.	In Core Thermocouples	М	R
15.	Unit Vent - High Range Noble Gas Monitor ( (High-High Range - EMF-36)	M	R
16.	Steam Relief - High Range Monitor (Unit 1 - EMF-24, 25, 26, 27) (Unit 2 - EMF-10, 11, 12, 13)	M	R
17.	Containment Atmosphere - High Range Monitor (EMF-51a or 51b)	М	R
18.	Reactor Vessel Level Instrumentation a. Wide Range b. Narrow Range	M M	R R

 TABLE 4.3-7

 ACCIDENT MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS

3/4 3-57

Amendment No. 21 (Unit 2) Amendment No. 40 (Unit 1)

McGUIRE - UNITS 1 and 2



UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

# SAFETY EVALUATION REPORT

RELATED TO AMENDMENT NO. 40 TO FACILITY OPERATING LICENSE NPF-9

AND TO AMENDMENT NO. 21 TO FACILITY OPERATING LICENSE NPF-17

## DUKE POWER COMPANY

McGUIRE NUCLEAR STATION, UNITS 1 AND 2

### INTRODUCTION

In November 1980, the staff issued NUREG-0737, "Clarification of TMI Action Plan Requirements," which included all TMI Action Plan items approved by the Commission for implementation at nuclear power reactors. NUREG-0737 identifies those items for which Technical Specifications are required. A number of items which require Technical Specifications (TSs) were scheduled for implementation after December 31, 1981. The staff provided guidance on the scope of Technical Specifications for these items in Generic Letter 83-37, which was issued to all Pressurized Water Reactor licensees on November 1, 1983. In this Generic Letter, the staff requested licensees to:

- 1. review their facility's Technical Specifications to determine if they were consistent with the guidance provided in the Generic Letter, and
- 2. submit an application for a license amendment where deviations or absence of Technical Specifications were found.

By letter dated July 31, 1984, Duke Power Company (the licensee) proposed license amendments to change Technical Specifications for McGuire Units 1 and 2 related to TMI Action Plan Item II.F.2 - Instrumentation for Inadequate Core Cooling. The Inadequate Core Cooling Instrumentation System at McGuire Nuclear Station, Units 1 and 2, consists of a subcooling margin monitor, reactor vessel level instrumentation system and core exit thermocouples.

### EVALUATION

Generic Letter 83-37 provided the guidance on TSs for the subcooling margin monitors, a reactor cooling inventory tracking system and core exit thermocouples. The TSs regarding operability and surveillance requirements for the subcooling margin monitors and core exit thermocouples are already included in existing Technical Specifications for McGuire Units 1 and 2. However, the licensee has proposed to increase the number of operable channels for subcooling margin monitors from one to two, and has proposed the TSs regarding operability and surveillance requirements for reactor vessel level instrumentation. We have reviewed the proposed TSs for the subcooling margin monitors and reactor vessel level instrumentation, and conclude that the proposed TSs are consistent with guidance contained in Generic Letter 83-37 and, therefore, are acceptable.

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### ENVIRONMENTAL CONSIDERATION

These amendments involve a change in the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and in surveillance requirements. The staff has determined that these amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that these amendments involve no significant hazards consideration, and there has been no public comment on such finding. Accordingly, these amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of these amendments.

#### CONCLUSION

The Commission made a proposed determination that the amendments involve no significant hazards consideration which was published in the Federal Register on October 24, 1984 (49 FR 42817) and consulted with the state of North Carolina. No public comments were received, and the state of North Carolina did not have any comments.

We have concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health, and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributors: Darl Hood, Licensing Branch No. 4, DL Chandu Patel, Operating Reactors Assessment Branch, DL

Dated: February 28, 1985

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AMENDMENT NO. 40 TO FACILITY OPERATING LICENSE NPF-9 - McGUIRE NUCLEAR STATION, UNIT 1 AMENDMENT NO. 21 TO FACILITY OPERATING LICENSE NPF-17 - McGUIRE NUCLEAR STATION, UNIT 2

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Docket Nos. 50-369/370 NRC PDR Local PDR NSIC LB #4 r/f E. Adensam D. Hood M. Duncan Attorney, OELD R. Diggs, ADM T. Barnhart (8) E. L. Jordan, DEQA:I&E J. M. Taylor, DRP:I&E L. J. Harmon, I&E File (2) D. Brinkman C. Patel, ORAB

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