



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

March 18, 1985

Docket Nos: 50-369
and 50-370

Mr. H. B. Tucker, Vice President
Nuclear Production Department
Duke Power Company
422 South Church Street
Charlotte, North Carolina 28242

Dear Mr. Tucker:

Subject: Issuance of Amendment No.41 to Facility Operating License
NPF-9 and Amendment No. 22 to Facility Operating License
NPF-17 - McGuire Nuclear Station, Units 1 and 2

The Nuclear Regulatory Commission has issued the enclosed Amendment No.41 to Facility Operating License NPF-9 and Amendment No.22 to Facility Operating License NPF-17 for the McGuire Nuclear Station, Units 1 and 2.

The amendments change the Technical Specifications to delete the surveillance requirements relating to steam generator blowdown valves (Table 3.6-2). The amendments are effective as of their dates of issuance.

A copy of the related safety evaluation supporting Amendment No.41 to Facility Operating License NPF-9 and Amendment No. 22 to Facility Operating License NPF-17 is enclosed.

Sincerely,

Elinor G. Adensam, Chief
Licensing Branch No. 4
Division of Licensing

Enclosures:

1. Amendment No.41 to NPF-9
2. Amendment No.22 to NPF-17
3. Safety Evaluation

cc w/encl:
See next page

DESIGNATED OFFICIAL
Certified By

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March 18, 1985

AMENDMENT NO. 41 TO FACILITY OPERATING LICENSE NPF-9 - McGUIRE NUCLEAR STATION, UNIT 1
AMENDMENT NO. 22 TO FACILITY OPERATING LICENSE NPF-17 - McGUIRE NUCLEAR STATION, UNIT 2

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The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Elinor G. Adensam, Chief
Licensing Branch No. 4
Division of Licensing

Attachment:
Technical Specification Changes

Date of Issuance: March 18, 1985



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

DUKE POWER COMPANY

DOCKET NO. 50-370

McGUIRE NUCLEAR STATION, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 22
License No. NPF-17

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment to the McGuire Nuclear Station, Unit 2 (the facility) Facility Operating License No. NPF-17 filed by the Duke Power Company (licensee) dated September 28, 1984, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's regulations as set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, as amended, the provisions of the Act, and the regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public;
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is hereby amended by page changes to the Technical Specifications as indicated in the attachments to this license amendment and paragraph 2.C.(2) of Facility Operating License No. NPF-17 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 22, are hereby incorporated into this license.

The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Elinor G. Adensam, Chief
Licensing Branch No. 4
Division of Licensing

Attachment:
Technical Specification Changes

Date of Issuance: March 18, 1985

ATTACHMENT TO LICENSE AMENDMENT NO. 41

FACILITY OPERATING LICENSE NO. NPF-9

DOCKET NO. 50-369

AND

TO LICENSE AMENDMENT NO. 22

FACILITY OPERATING LICENSE NO. NPF-17

DOCKET NO. 50-370

Replace the following page of the Appendix "A" Technical Specifications with the enclosed page. The revised page is identified by Amendment number and contains a vertical line indicating the area of change. The corresponding overleaf page is being provided to maintain document completeness.

Amended
Page

3/4 6-24

Overleaf
Page

3/4 6-23

CONTAINMENT SYSTEMS

SURVEILLANCE REQUIREMENTS (Continued)

4.6.3.2 Each isolation valve specified in Table 3.6-2 shall be demonstrated OPERABLE during the COLD SHUTDOWN or REFUELING MODE at least once per 18 months by:

- a. Verifying that on a Phase A Containment Isolation test signal, each Phase A isolation valve actuates to its isolation position,
- b. Verifying that on a Phase B Containment Isolation test signal, each Phase B isolation valve actuates to its isolation position, and
- c. Verifying that on a Containment Radioactivity-High test signal, each purge and exhaust valve actuates to its isolation position.

4.6.3.3 The isolation time of each power operated or automatic valve of Table 3.6-2 shall be determined to be within its limit when tested pursuant to Specification 4.0.5.

TABLE 3.6-2
CONTAINMENT ISOLATION VALVES

<u>VALVE NUMBER</u>	<u>FUNCTION</u>	<u>MAXIMUM ISOLATION TIME (SEC)</u>
1. Phase "A" Isolation		
BB-1B#	Steam Generator A Blowdown Containment Outside Isolation	<10
BB-2B#	Steam Generator B Blowdown Containment Outside Isolation	<10
BB-3B#	Steam Generator C Blowdown Containment Outside Isolation	<10
BB-4B#	Steam Generator D Blowdown Containment Outside Isolation	<10
BB-5A#	Steam Generator A Blowdown Containment Inside Isolation	<10
BB-6A#	Steam Generator B Blowdown Containment Inside Isolation	<10
BB-7A#	Steam Generator C Blowdown Containment Inside Isolation	<10
BB-8A#	Steam Generator D Blowdown Containment Inside Isolation	<10
CF-26AB#	Steam Generator D Feedwater Containment Isolation	<5
CF-28AB#	Steam Generator C Feedwater Containment Isolation	<5
CF-30AB#	Steam Generator B Feedwater Containment Isolation	<5
CF-35AB#	Steam Generator A Feedwater Containment Isolation	<5
CF-126B	Steam Generator A Main Feedwater to Auxiliary Feedwater Nozzle Isolation	<10
CF-127B	Steam Generator B Main Feedwater to Auxiliary Feedwater Nozzle Isolation	<10
CF-128B	Steam Generator C Main Feedwater to Auxiliary Feedwater Nozzle Isolation	<10
CF-129B	Steam Generator D Main Feedwater to Auxiliary Feedwater Nozzle Isolation	<10
CF-134A	Steam Generator A Feedwater Containment Isolation Bypass	<10
CF-135A	Steam Generator B Feedwater Containment Isolation Bypass	<10
CF-136A	Steam Generator C Feedwater Containment Isolation Bypass	<10
CF-137A	Steam Generator D Feedwater Containment Isolation Bypass	<10
CF-151A	Auxiliary Nozzle Temper SG A	<10



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SAFETY EVALUATION REPORT

RELATED TO AMENDMENT NO. 41 TO FACILITY OPERATING LICENSE NPF-9

AND TO AMENDMENT NO. 22 TO FACILITY OPERATING LICENSE NPF-17

DUKE POWER COMPANY

MCGUIRE NUCLEAR STATION, UNITS 1 AND 2

I. INTRODUCTION

By letter dated September 28, 1984, the licensee proposed amendments to license Nos. NPF-9 and NPF-17 which would change the Technical Specifications by deleting the surveillance requirements for four Steam Generator Blowdown Valves (BB140, BB141, BB142, BB143) listed in Specification Table 3.6-2, Containment Isolation Valves. The licensee is making modifications to the plant which will make it acceptable to eliminate these valves and replace them with spool pieces.

II. EVALUATION

The purpose of the Steam Generator Blowdown Recycle System (referred to by the licensee as the BB system) is to maintain acceptable steam generator shell-side chemistry and to conserve the blowdown water through recycling. Each steam generator has one BB line exiting the containment and entering the Auxiliary Building, resulting in four containment penetrations for each unit. Each of these containment penetrations has containment isolation capability provided by valves BB1 through BB4 on the Auxiliary Building side and valves BB5 through BB8 on the Containment side. These eight valves are included in the leak test program pursuant to Appendix J to 10 CFR 50.

Steam generator blowdown valves BB140 through BB143, which are located inside containment near the steam generators, were originally installed to permit pipe class changes from licensee's Class B to F, which were necessary to accommodate non-safety related BB subcooling heat exchangers in the lines between the steam generators and the containment wall. The licensee's Class B piping is ASME Class 2 or Quality Group B, while their Class F is not safety grade. In order for the Main Steam System (steam generators, main steam lines, etc.) to be considered a closed system inside containment as required by General Design Criterion 57, the non-safety grade heat exchangers and their Class F piping had to be isolated from the rest of the Main Steam System by valves BB140 through BB143. However, the licensee has removed the heat exchangers. The piping spools that replaced the heat exchangers and the remaining Class F piping are being upgraded to Class B. Thus, all the BB system inside containment (and that portion to and including valves BB1 through BB4 outside containment) will be Class B. Therefore, the whole Main Steam System qualifies as a closed system inside containment, and the four

valves, BB140 through BB143, are no longer necessary to serve the safety (isolation) function for which they were originally installed. They are being removed from the lines and, consequently, must be deleted from the table to maintain consistency between the design and the Technical Specifications.

Based on our review, we conclude that because of the plant modifications discussed above, valves BB140, BB141, BB142, and BB143 are no longer necessary to serve their earlier safety (isolation) function (i.e., the modified Steam Generator Blowdown Recycle System is such that the Main Steam System will satisfy the requirements of General Design Criterion 57 and Appendix J to 10 CFR 50 without these four valves), and their deletion from Specification Table 3.6-2, Containment Isolation Valves, is therefore acceptable.

III. ENVIRONMENTAL CONSIDERATION

These amendments involve a change in use of facility components located within the restricted area as defined in 10 CFR Part 20 and change a surveillance requirement. The staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that these amendments involve no significant hazards consideration, and there has been no public comment on such finding. Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR Section 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of these amendments.

IV. CONCLUSION

The Commission made a proposed determination that the amendments involve no significant hazards consideration which was published in the Federal Register (49 FR 50802) on December 31, 1984, and consulted with the state of North Carolina. No public comments were received, and the state of North Carolina did not have any comments.

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributors: D. S. Hood, Licensing Branch No. 4, DL
J. Pulsipher, Containment Systems Branch, DSI

Dated: March 18, 1985



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

DUKE POWER COMPANY

DOCKET NO. 50-369

McGUIRE NUCLEAR STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 41
License No. NPF-9

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment to the McGuire Nuclear Station, Unit 1 (the facility) Facility Operating License No. NPF-9 filed by the Duke Power Company (licensee) dated September 28, 1984, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's regulations as set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, as amended, the provisions of the Act, and the regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public;
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is hereby amended by page changes to the Technical Specifications as indicated in the attachments to this license amendment and paragraph 2.C.(2) of Facility Operating License No. NPF-9 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 41, are hereby incorporated into this license.