

April 22, 1992

Docket No. 50-364

Mr. W. G. Hairston, III
Senior Vice President
Southern Nuclear Operating
Company, Inc.
Post Office Box 1295
Birmingham, Alabama 35201-1295

Dear Mr. Hairston:

SUBJECT: CORRECTION TO TECHNICAL SPECIFICATIONS FOR AMENDMENT
NO. 87 FOR JOSEPH M. FARLEY NUCLEAR PLANT, UNIT 2
(TAC NO. M82810)

On April 1, 1992, the Commission issued Amendment No. 87 to the Joseph M. Farley Nuclear Plant, Unit 2, Technical Specifications (TS). Page 3/4 4-17 had an error in 3.4.7.2.d. The "10 GPM UNIDENTIFIED LEAKAGE" should have read "10 GPM IDENTIFIED LEAKAGE." This change has been made and a corrected page is enclosed for your use.

Sincerely,
Original signed by:

Stephen T. Hoffman, Project Manager
Project Directorate II-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosure:
TS Page 3/4 4-17

cc w/enclosure:
See next page

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|------|------------------------------|------------------------------|------------------------------|--------|
| OFC | PDII-I/II <i>[Signature]</i> | PDII-I/II <i>[Signature]</i> | PDII-I/II <i>[Signature]</i> | OGC |
| NAME | PAnderson | SHoffman:dw | EAdensam | |
| DATE | 4/21/92 | 4/22/92 | 4/22/92 | 4/ /92 |

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Mr. W. G. Hairston, III
Southern Nuclear Operating
Company, Inc.

Joseph M. Farley Nuclear Plant

cc:

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OPERATIONAL LEAKAGE

LIMITING CONDITION FOR OPERATION

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3.4.7.2 Reactor Coolant System leakage shall be limited to:

- a. No PRESSURE BOUNDARY LEAKAGE,
- b. 1 GPM UNIDENTIFIED LEAKAGE,
- c. For the Ninth Operating Cycle only, primary-to-secondary leakage through all steam generators shall be limited to 450 gallons per day and 150 gallons per day through any one steam generator.

For subsequent cycles, 1 GPM total primary-to-secondary leakage through all steam generators and 500 gallons per day through any one steam generator,
- d. 10 GPM IDENTIFIED LEAKAGE from the Reactor Coolant System, and
- e. 31 GPM CONTROLLED LEAKAGE at a Reactor Coolant System pressure of 2235 ± 20 psig.
- f. The maximum allowable leakage of any Reactor Coolant System Pressure Isolation Valve shall be as specified in Table 3.4-1 at a pressure of 2235 ± 20 psig.

APPLICABILITY: MODES 1, 2, 3 and 4

ACTION:

- a. With any PRESSURE BOUNDARY LEAKAGE, be in at least HOT STANDBY within 6 hours and in COLD SHUTDOWN within the following 30 hours.
- b. With any Reactor Coolant System leakage greater than any one of the above limits, excluding PRESSURE BOUNDARY LEAKAGE, reduce the leakage rate to within limits within 4 hours or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.
- c. With any Reactor Coolant System Pressure Isolation Valve leakage greater than the limit specified in Table 3.4-1, isolate the high pressure portion of the affected system from the low pressure portion within 4 hours by use of at least two closed manual or deactivated automatic valves, or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.