

Docket Nos: 50-369  
and 50-370

December 2, 1983

Mr. H. B. Tucker, Vice President  
Nuclear Production Department  
Duke Power Company  
422 South Church Street  
Charlotte, North Carolina 28242

Dear Mr. Tucker:


Subject: Issuance of Amendment No. 27 to Facility Operating License  
NPF-9 and Amendment No. 8 to Facility Operating License  
NPF-17 - McGuire Nuclear Station, Units 1 and 2

The Nuclear Regulatory Commission has issued the enclosed Amendment No. 27 to Facility Operating License NPF-9 and Amendment No. 8 to Facility Operating License NPF-17 for the McGuire Nuclear Station, Units 1 and 2. These amendments consist of changes to the Technical Specifications in response to your application which was originally submitted on September 22, 1983. When the need for the amendments became critical, additional information was telecopied to us on October 26, 1983. The amendments were authorized by telephone on October 26, 1983, and were confirmed by letter that same day.

The amendments change the Technical Specifications related to the containment lower compartment temperature to allow the temperature limit to be increased from 120°F to 125°F for up to 90 cumulative days a year provided that the lower compartment temperature had averaged less than 120°F over the previous 365 days. The amendments were issued on an expedited basis to prevent plant shutdown.

A copy of the related safety evaluation report supporting Amendment No. 27 to Facility Operating License NPF-9 and Amendment No. 8 to Facility Operating License NPF-17 is enclosed. Also enclosed is a copy of a related Notice of Issuance of Amendment which has been forwarded to the Office of the Federal Register for publication.

Sincerely,

  
Elinor G. Adensam, Chief  
Licensing Branch No. 4  
Division of Licensing

Enclosures:

1. Amendment No. 27 to NPF-9
2. Amendment No. 8 to NPF-17
3. Safety Evaluation
4. F. R. Notice

cc w/encl:  
See next page

LA:DL:LB #4  
MDuncan/hmc  
10/1/83

DL:LB #4  
RBirkel  
10/1/83

DL:LB #4  
EAdensam  
11/1/83

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PDR ADOCK 05000369  
P PDR

McGuire

Mr. H. B. Tucker, Vice President  
Nuclear Production Department  
Duke Power Company  
422 South Church Street  
Charlotte, North Carolina 28242

cc: Mr. A. Carr  
Duke Power Company  
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Raleigh, North Carolina 27602

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Radiation Protection Branch  
Division of Facility Services  
Department of Human Resources  
P.O. Box 12200  
Raleigh, North Carolina 27605

DUKE POWER COMPANY

DOCKET NO. 50-369

McGUIRE NUCLEAR STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 27  
License No. NPF-9

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment to the McGuire Nuclear Station, Unit 1 (the facility) Facility Operating License No. NPF-9 filed by the Duke Power Company (licensee) dated September 22 and supplemented October 26, 1983, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's regulations as set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, as amended, the provisions of the Act, and the regulations of the Commission;
  - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
  - D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public;
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is hereby amended by page changes to the Technical Specifications as indicated in the attachments to this license amendment and paragraph 2.C.(2) of Facility Operating License No. NPF-9 is hereby amended to read as follows:

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P PDR

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 27, are hereby incorporated into this license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment was effective October 26, 1983.

FOR THE NUCLEAR REGULATORY COMMISSION

51

Elinor G. Adensam, Chief  
Licensing Branch No. 4  
Division of Licensing

Attachment:  
Technical Specification  
Change

Date of Issuance: December 2, 1983

LA:DL:LB #4  
MDuncan/hmc  
10/1/83

DL:LB #4  
RBirkel  
10/1/83

OEL  
R. Rawson  
11/23/83

DL:LB #4  
EAdensam  
12/1/83

AD:DL  
TNovak  
12/1/83

*concurrent condition on  
indicated changes*

DUKE POWER COMPANY

DOCKET NO. 50-370

McGUIRE NUCLEAR STATION, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 8  
License No. NPF-17

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment to the McGuire Nuclear Station, Unit 2 (the facility) Facility Operating License No. NPF-17 filed by the Duke Power Company (licensee) dated September 22 and supplemented October 26, 1983, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's regulations as set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, as amended, the provisions of the Act, and the regulations of the Commission;
  - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
  - D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public;
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is hereby amended by page changes to the Technical Specifications as indicated in the attachments to this license amendment and paragraph 2.C.(2) of Facility Operating License No. NPF-17 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 8, are hereby incorporated into this license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment was effective October 26, 1983.

FOR THE NUCLEAR REGULATORY COMMISSION

5 |  
Elinor G. Adensam, Chief  
Licensing Branch No. 4  
Division of Licensing

Attachment:  
Technical Specification  
Change

Date of Issuance: December 2, 1983

LA:DL:LB #4  
MDuncan/hmc  
10/1/83

DL:LB #4  
RBirke1  
10/1/83

OEC *pro*  
R. Rawson  
11/23/83

DL:LB #4  
EAdensam  
11/1/83

AD:DL  
TNovak  
11/1/83

*concurrent with  
or indicated change*

ATTACHMENT TO LICENSE AMENDMENT NO. 27

FACILITY OPERATING LICENSE NO. NPF-9

DOCKET NO. 50-369

AND

TO LICENSE AMENDMENT NO. 8

FACILITY OPERATING LICENSE NO. NPF-17

DOCKET NO. 50-370

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain a vertical line indicating the area of change. The corresponding overleaf pages are also provided to maintain document completeness.

Amended  
Page

3/4 6-13

Overleaf  
Page

3/4 6-14

## CONTAINMENT SYSTEMS

### AIR TEMPERATURE

#### LIMITING CONDITION FOR OPERATION

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3.6.1.5 Primary containment average air temperature shall be maintained:

- a. Between 75\* and 100°F in the containment upper compartment, and
- b. Between 100\* and 120°F\*\*\* in the containment lower compartment.

APPLICABILITY: MODES 1, 2, 3, and 4.

#### ACTION:

With the containment average air temperature not conforming to the above limits, restore the air temperature to within the limits within 8 hours or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

#### SURVEILLANCE REQUIREMENTS

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4.6.1.5.1 The primary containment upper compartment average air temperature shall be the weighted average\*\* of ambient air temperature monitoring stations located in the upper compartment. Temperature readings will be obtained at least once per 24 hours from the elevation of 826 feet at the inlet of each upper containment ventilation unit.

4.6.1.5.2 The primary containment lower compartment average air temperature shall be the weighted average\*\* of ambient air temperature monitoring stations located in the lower compartment. Temperature readings will be obtained at least once per 24 hours from the elevation of 745 feet at the inlet of each lower containment ventilation unit.

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\*Lower limit may be reduced to 60°F in MODES 2, 3, and 4.

\*\*The weighted average is the sum of each temperature multiplied by its respective containment volume fraction. In the event of inoperative temperature sensor(s), the weighted average shall be taken as the reduced total divided by one minus the volume fraction represented by the sensor(s) out of service.

\*\*\*Containment lower compartment temperature may be between 120 and 125°F for up to 90 cumulative days per calendar year provided the lower compartment temperature average over the previous 365 days is less than 120°F.



## CONTAINMENT SYSTEMS

### CONTAINMENT VESSEL STRUCTURAL INTEGRITY

#### LIMITING CONDITION FOR OPERATION

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3.6.1.6 The structural integrity of the containment vessel shall be maintained at a level consistent with the acceptance criteria in Specification 4.6.1.6.

APPLICABILITY: MODES 1, 2, 3, and 4.

#### ACTION:

With the structural integrity of the containment vessel not conforming to the above requirements, restore the structural integrity to within the limits prior to increasing the Reactor Coolant System temperature above 200°F.

#### SURVEILLANCE REQUIREMENTS

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4.6.1.6 The structural integrity of the containment vessel shall be determined during the shutdown for each Type A containment leakage rate test (reference Specification 4.6.1.2) by a visual inspection of the exposed accessible interior and exterior surfaces of the vessel. This inspection shall be performed prior to the Type A containment leakage rate test to verify no apparent changes in appearance of the surfaces or other abnormal degradation. Any abnormal degradation of the containment vessel detected during the above required inspections shall be reported to the Commission pursuant to Specification 6.9.1.

## SAFETY EVALUATION REPORT

RELATED TO AMENDMENT NO. 27 TO FACILITY OPERATING LICENSE NPF-9

AND TO AMENDMENT NO. 8 TO FACILITY OPERATING LICENSE NPF-17

DUKE POWER COMPANY

### INTRODUCTION

By letter dated September 22, 1983, the Duke Power Company (licensee) requested amendments to Appendix A of Operating Licenses NPF-9 and NPF-17. The proposed change involves Technical Specification 3.6.1.5, "Containment Systems Air Temperature". If the primary containment lower compartment air temperature exceeds 120°F, the action statement of the existing technical specification requires restoration of the air temperature to 120°F within 8 hours or be in at least hot standby within the next 6 hours and in cold shutdown within the following 30 hours. The proposed change would allow the air temperature to increase to 125°F for up to 90 cumulative days per calendar year provided the lower compartment temperature averaged over the previous 365 days is less than 120°F.

The limitations on containment average air temperature ensure that the containment air mass is limited to an initial mass sufficiently low to prevent exceeding the design pressure during LOCA conditions, and the ambient air temperature does not exceed that temperature allowable for the continuous duty rating specified for equipment and instrumentation located within containment.

McGuire uses cooling water from Lake Norman to circulate through air handling units which cool the containment atmosphere. The cooling water is drawn from the hypolimnion region near the bottom of the lake which is much cooler than the water near the lake surface during the summer. As cooler weather approaches in early autumn, however, the hypolimnion region mixes with the surface water temporarily causing higher cooling water temperatures. The licensee estimates, based upon historical temperature data, that the lower compartment temperatures at McGuire might exceed 120°F for a short period during the autumn until cool weather reduces the bulk temperature of the lake. During 1982, significant efforts were required on McGuire Unit 1 to maintain lower compartment temperature below 120°F. These efforts included thoroughly cleaning the coolers while the unit was shutdown, installing a water spray system to improve the efficiency of the coolers, and inspecting and improving insulation. Despite similar efforts on McGuire Unit 2, the 120°F limit was exceeded on October 26, 1983, and the Unit entered the Action Statement of the Technical Specification. Based on McGuire Station operation, the licensee's letter of October 26, 1983, requested that the proposed amendments be issued and immediately effective as emergency license amendments, pursuant to 10 CFR 50.91.(a).(5). The imminent potential for derating either one or both McGuire Units has been demonstrated by the licensee and commensurate with this potential, the staff agrees with the licensee's basis for an emergency license amendment without delay.

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## EVALUATION

The impact of the proposed change on the McGuire FSAR safety analysis of increasing the allowable lower compartment temperature to 125°F has been evaluated by the licensee. Potential impacts of this change affect the containment design basis analysis and the containment backpressure used in the LOCA analyses.

The containment design basis analysis assumed a low initial temperature. Use of a low initial temperature results in maximum containment air mass and, hence, higher peak containment pressures. Thus, increasing the allowable lower compartment temperature does not affect the containment design basis analysis.

Increased lower compartment temperatures would, however, result in lower containment backpressures for use in the LOCA analysis. The minimum containment backpressure analysis for LOCA backpressure is given in FSAR Figure 15.4.1-62. The licensee estimates, and the staff concurs, that the higher lower compartment temperature would decrease the LOCA backpressure by less than 0.15 psi.

The estimated minimum backpressure curve has been compared, by the licensee, to the containment backpressure utilized in the LOCA analysis. The comparison showed that the LOCA analysis utilized a lower containment backpressure than the estimated curve except between 70-90 seconds and 105 - 120 seconds following the LOCA. The licensee concludes that the effect on the beginning of the reflood phase and on peak cladding temperature as a result of the small reduction in containment backpressure over these time periods is minimal.

The staff agrees that the reduction in containment backpressure will not result in a significant increase in peak cladding temperatures. The evaluations performed by the licensee to estimate the impact of the higher compartment temperature are conservative. The staff further believes that the LOCA analysis performed by the licensee contains sufficient conservatism in the containment backpressure calculation to assure that any effect of the lower backpressure would not result in a violation of 10 CFR 50.46.

The staff finds that the proposed Tech Spec change is acceptable. However, the staff has requested that the licensee provide confirmatory information either as, (1) confirmatory calculations, utilizing the higher lower compartment air temperature, documenting conformance to 10 CFR 50.46, or (2) documented justification that the containment backpressure used in the LOCA analysis contains sufficient excess of statistical uncertainties such that this margin is more than sufficient to bound the backpressure resulting from the higher lower compartment temperature for purposes of documentation.

## FINAL NO SIGNIFICANT HAZARDS CONSIDERATION (SHC) DETERMINATION

The Commission made a proposed determination that these amendments involve no significant hazards consideration (SHC). Notice of Consideration of Issuance of Amendments to Facility Operating Licenses and Proposed No Significant Hazards Consideration Determinations and Opportunity for Hearing was published in the Federal Register on October 25, 1983 (48 FR 49394). The amendments are being

issued before expiration of the 30-day comment period because failure to do so would result in plant shutdown. The State of North Carolina was consulted and did not have any comments. Based on the Commission's final review, the Commission has made a final determination that these amendments involve no significant hazards consideration. Because the increase in lower compartment temperature would be small, less than 5% compared to the allowable temperature of 120°F under the current Technical Specification, the proposed amendments do not involve a significant increase in the probability of an accident previously evaluated or a significant reduction in a margin of safety. Because no changes in any accident analysis will result from the increase in lower compartment temperature, the proposed amendments do not involve any increase in the consequences of an accident previously evaluated nor do they create the possibility of a new or different kind of accident.

#### ENVIRONMENTAL CONSIDERTION

We have determined that the amendments do not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendments involve an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of these amendments.

#### CONCLUSION

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributors: Ralph Birkel, Licensing Branch No. 4, DL  
R. C. Jones, Reactor Systems Branch, DSI  
J. C. Pulsipher, Containment Systems Branch, DSI

Dated: December 2, 1983

\*NOTE: See Previous White for Concurrence

DL:LB #4	LA:DL:LB #4	RSB	CSB	DL:LB #4	AD:DL
*RBirkel/hmc	*MDuncan	*BSheron	*WButler	EAdensam	TNovak
11/01/83	11/01/83	11/01/83	11/01/83	11/1/83	11/1/83

issued before expiration of the 30-day comment period because failure to do so would result in plant shutdown. The State of North Carolina was consulted and did not have any comments. Based on the Commission's final review and the absence of State comments, the Commission has made a final determination that these amendments involve no significant hazards consideration.

#### ENVIRONMENTAL CONSIDERATION

We have determined that the amendments do not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendments involve an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of these amendments.

#### CONCLUSION

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributors: Ralph Birkel, Licensing Branch No. 4, DL  
R. C. Jones, Reactor System Branch, DSI  
J. C. Pulsipher, Containment Systems Branch, DSI

Dated:

DL:LB #4  
RBirkel/hmc  
10/1/83

LA:DL:AB #4  
MDuncan  
10/1/83

RSB  
Sheron  
11/1/83

CSB  
WButler  
11/6/83

DL:LB #4  
EAdensam  
11/ /83

AD:L:DL  
TNovak  
11/ /83

*Ralph Birkel*  
*Carachin*  
*WUS*

U.S. NUCLEAR REGULATORY COMMISSION  
DUKE POWER COMPANY  
DOCKET NOS. 50-369 AND 50-370  
NOTICE OF ISSUANCE OF AMENDMENT TO FACILITY  
OPERATING LICENSE  
AND FINAL DETERMINATION OF NO SIGNIFICANT  
HAZARDS CONSIDERATION

The U.S. Nuclear Regulatory Commission (Commission) has issued Amendment No. 27 to Facility Operating License No. NPF-9 and Amendment No. 8 to Facility Operating License No. NPF-17, issued to Duke Power Company (the licensee), which revised the Technical Specifications for operation of the McGuire Nuclear Station, Units 1 and 2, (the facilities) located in Mecklenburg County, North Carolina. The amendments were effective October 26, 1983.

The amendments change the Technical Specifications related to the containment lower compartment temperature to allow the temperature limit to be increased from 120°F to 125°F for up to 90 cumulative days a year provided that the lower compartment temperature had averaged less than 120°F over the previous 365 days.

The application for the amendments complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendments.

Notice of Consideration of Issuance of Amendment and Proposed No Significant Hazards Consideration Determination and Opportunity for Hearing in connection with this action was published in the FEDERAL REGISTER on October 25, 1983 (48 FR 49394). The amendments were issued before expiration of the 30-day comment period because failure to do so would result in plant shutdown.

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Under its regulations, the Commission may issue and make an amendment immediately effective, notwithstanding the pendency before it of a request for a hearing from any person, in advance of the holding and completion of any required hearing, where it has determined that no significant hazards consideration is involved.

The Commission has applied the standards of 10 CFR 50.92 and has made a final determination that the amendments involve no significant hazards consideration. The basis for this determination is contained in the Safety Evaluation related to this action. Because the increase in lower compartment temperature would be small, less than 5% compared to the allowable temperature of 120°F under the current Technical Specification, the proposed amendments do not involve a significant increase in the probability of an accident previously evaluated or a significant reduction in a margin of safety. Because no changes in any accident analysis will result from the increase in lower compartment temperature, the proposed amendments do not involve any increase in the consequences of an accident previously evaluated nor do they create the possibility of a new or different kind of accident. Accordingly, as described above, the amendment has been issued and made immediately effective and any hearing will be held after issuance.

The Commission has determined that the issuance of the amendments will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of the amendments.

For further details with respect to the action see (1) the application for amendment dated September 22 and supplemented October 26, 1983, (2) Amendment No. 27 to Facility Operating License NPF-9, (3) Amendment No. 8 to Facility Operating License No. NPF-17, and (4) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D. C., and at the Atkins Library, University of North Carolina, Charlotte (UNCC Station), North Carolina 28242.

A copy of items (2), (3), and (4) may be obtained upon request addressed to the U.S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland this 2<sup>nd</sup> day of December 1983.

FOR THE NUCLEAR REGULATORY COMMISSION

5/

Elinor G. Adensam, Chief  
Licensing Branch No. 4  
Division of Licensing

\*See previous concurrence.

DL:LB#4\*

MDuncan/hmc

11/1/83

DL:LB#4

RABirke1

11/1/83

OEL *ORA*

R. RAWSON

11/23/83

DL:LB#4

EAdensam

11/1/83

*concurrence conditional  
on initiated change*



A copy of items (2), (3), and (4) may be obtained upon request addressed to the U.S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland this            day of

FOR THE NUCLEAR REGULATORY COMMISSION

Elinor G. Adensam, Chief  
Licensing Branch No. 4  
Division of Licensing

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Decemb 2, 1983

AMENDMENT NO. 27 TO FACILITY OPERATING LICENSE NPF-9 - McGUIRE NUCLEAR STATION, UNIT 1  
AMENDMENT NO. 8 TO FACILITY OPERATING LICENSE NPF-17 - McGUIRE NUCLEAR STATION, UNIT 2

DISTRIBUTION:

✓ Docket Nos. 50-369/370  
NRC PDR  
Local PDR  
NSIC  
LB #4 r/f  
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R. Birkel  
M. Duncan  
Attorney, OELD  
D. Eisenhower/R. Purple  
R. Hartfield, MPA  
R. Diggs, ADM  
T. Barnhart (8)  
J. Souder  
A. Rosenthal, ASLAB  
W. Jones, OA (10)  
ACRS (16)  
ASLBP  
E. L. Jordan, DEQA:I&E  
J. M. Taylor, DRP:I&E  
L. J. Harmon, I&E File (2)  
D. Brinkman  
H. Denton  
B. Sheron, RSB  
W. Butler, CSB

INITIAL

NO SIGNIFICANT HAZARDS CONSIDERATION DETERMINATION  
AND NOTICING ACTION

Docket No. 50-369 and 50-370 Facility: McGuire 1 & 2

Licensee: Duke Power Company Date of application: 9/22/83

Request for: Technical Specification change to allow the upper limit for containment lower compartment <sup>temperature</sup> to be increased for up to 90 cumulative days per year.

(See attached notice or press release for more details.)

Initial Determination:

- (x) Proposed determination - amendment request involves no significant hazards considerations (NSHC).
- ( ) Final determination - amendment request involves significant hazards considerations (SHC).

Basis for Determination

- (x) Licensee's NSHC discussion has been reviewed and is accepted. See attached amendment request.
- ( ) Basis for this determination is presented in the attached notice.
- ( ) Other (state):

(Attach additional sheets as needed.)

Initial Noticing Action: (Attach appropriate notice or input for monthly FRN)

1. ( ) Monthly FRN. Notice of opportunity for hearing (30 days) and request for comments on proposed NSHC determination - monthly FRN input is attached (Attachment 8).
2. (x) Individual FRN (30 days). Same notice matter as above. Time does not allow waiting for next monthly FRN (Attachments 9a and 9b).

(THIS FORM SHOULD BE TYPED EXCEPT FOR UNUSUAL, URGENT CIRCUMSTANCES.)

3. ( ) Local media notice. Valid exigent circumstances exist (evaluated below).  
Local media notice requesting public comments on proposed NSHC determination is attached (Attachment 10).
4. ( ) No notice. A valid emergency situation exists (evaluated below) and there is no time for public notice on proposed NSHC determination.  
(No attachment.)
5. ( ) Individual FRN (30-days). Licensee's claim of exigent or emergency circumstances is invalid (evaluated below). Notice of opportunity for hearing (30 days) and request for comments on proposed NSHC determination is attached (Attachments 9a and 9b). Letter of explanation to licensee is also attached.
6. ( ) Individual FRN (30-days). The amendment request involves SHC. Notice of opportunity for prior hearing is attached (Attachment 5). Letter to licensee also attached.
7. ( ) Individual Short FRN. Valid emergency circumstances exist (evaluated below). There is no time for the usual 30-day FRN. (Attachment 16).

Evaluation of exigent or emergency circumstances (if applicable):

(attach additional sheets as needed)

Approvals:

Date

1. Ralph A. Birkel *RA*  
(Project Manager)
2. Elinor G. Adensam *E. Adensam*  
(Branch Chief)
3. J.R. GRAY *JR Gray*  
OELDY

9/27/83  
~~9/1/83~~ 10/5/83  
10/7/83

Additional approval (for noticing actions types 3, 4, 5, 6 and 7):

4. \_\_\_\_\_  
(Assistant Director)

Additional approval (for noticing action types 4 and 5):

5. \_\_\_\_\_  
(Director, Division of Licensing)

Attachment: as indicated

cc: Original - Docket File (with note "Docket File only")  
Project Manager  
Licensing Assistant  
Branch Files