

September 23, 1981

Docket No. 50-369

Mr. William O. Parker, Jr.  
Vice President Steam Production  
Duke Power Company  
P.O. Box 2178  
422 South Church Street  
Charlotte, North Carolina 28242

Dear Mr. Parker:

Subject: Issuance of Amendment No. 5 to Facility Operating License  
NPF-9 - McGuire Nuclear Station, Unit 1

The Nuclear Regulatory Commission has issued Amendment No. 5 to Facility Operating License NPF-9 for the McGuire Nuclear Station, Unit 1, located in Mecklenburg County, North Carolina.

This amendment is in response to your letter dated September 21, 1981. This amendment corrects the setpoint associated with the reactor trip initiated by a turbine trip.

A copy of the related safety evaluation report supporting Amendment No. 5 to Facility Operating License NPF-9 is enclosed. Also enclosed is a copy of a related notice which has been forwarded to the Office of the Federal Register for publication.

Sincerely,

*S/*

Elinor G. Adensam, Acting Chief  
Licensing Branch No. 4  
Division of Licensing  
Office of Nuclear Reactor Regulation

Enclosures:

- 1. Amendment No. 5
- 2. Safety Evaluation
- 3. Federal Register Notice

cc w/encl:  
See next page

8110130023 810923  
PDR ADDCK 05000369  
P PDR

*ELD E.K. 9/22*

*CONSTRUCTION PERMIT*

OFFICE	LA:DL:LB #4	DL:LB #4	DL:LB #4				
SURNAME	<i>md</i> MDuncan/hmc...	<i>RB</i> RBirke...	<i>EAdensam</i> EAdensam				
DATE	9/22/81	9/22/81	9/23/81				

Mr. William O. Parker, Jr.  
 Vice President - Steam Production  
 Duke Power Company  
 P.O. Box 2178  
 422 South Church Street  
 Charlotte, North Carolina 28242

cc: Mr. W. L. Porter  
 Mr. A. Carr  
 Duke Power Company  
 P.O. Box 2178  
 422 South Church Street  
 Charlotte, North Carolina 28242

Mr. R. S. Howard  
 Power Systems Division  
 Westinghouse Electric Corp.  
 P.O. Box 355  
 Pittsburgh, Pennsylvania 1530

Mr. E. J. Keith  
 EDS Nuclear Incorporated  
 220 Montgomery Street  
 San Francisco, California 94104

Mr. J. E. Houghtaling  
 NUS Corporation  
 2536 Countryside Boulevard  
 Clearwater, Florida 33515

Mr. Jesse L. Riley, President  
 The Carolina Environmental Study Group  
 854 Henley Place  
 Charlotte, North Carolina 28207

J. Michael McGarry, III, Esq.  
 DeBevoise & Liberman  
 1200 Seventeenth Street, N.W.  
 Washington, D. C. 20036

Ms. M. J. Graham  
 Resident Inspector McGuire NPS  
 c/o U.S. Nuclear Regulatory Commission  
 P.O. Box 216  
 Cornelius, North Carolina 28031

Shelley Blum, Esq.  
 1716 Scales Street  
 Raleigh, North Carolina 27608

Mr. David E. Smith  
 City of Charlotte  
 Legal Department

Attorney General  
 Department of Justice  
 Justice Building  
 Raleigh, North Carolina 27602

Office of Intergovernmental Relations  
 116 West Jones Street  
 Raleigh, North Carolina 27603

County Manager of Mecklenburg County  
 720 East Fourth Street  
 Charlotte, North Carolina 28202

Mr. Bruce Blanchard  
 Environmental Projects Review  
 Department of the Interior  
 Room 4256  
 18th and C Street, N.W.  
 Washington, D. C. 20240

U.S. Environmental Protection Agency  
 ATTN: Ms. Elizabeth V. Jankus  
 Office of Environmental Review  
 Room 2119 M, A-104  
 401 M Street, S.W.  
 Washington, D. C. 20460

Director, Criteria and Standards Div.  
 Office of Radiation Programs  
 (ANR-460)  
 U.S. Environmental Protection Agency  
 Washington, D. C. 20460

EIS Coordinator  
 U.S. Environmental Protection Agency  
 Region IV Office  
 345 Courtland Street, N.E.  
 Atlanta, Georgia 30308

Chairman, North Carolina  
 Utilities Commission  
 430 North Salisbury Street  
 Dobbs Building  
 Raleigh, North Carolina 27602

OFFICE	City Hall	Dr. John M. Barry
SURNAME	600 E. Trade Street	Department of Environmental Health
DATE	Charlotte, North Carolina 28211	Mecklenburg County
		1200 Blythe Boulevard
		Charlotte, North Carolina 28203

DUKE POWER COMPANY

DOCKET NO. 50-369

MCGUIRE NUCLEAR STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 5  
License No. NPF-9

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment to the McGuire Nuclear Station, Unit 1 (the facility) Facility Operating License No. NPF-9 filed by the Duke Power Company (licensee) dated September 21, 1981, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's regulations as set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, as amended, the provisions of the Act, and the regulations of the Commission;
  - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
  - D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public;
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is hereby amended by page changes to the Technical Specifications as indicated in the attachments to this license amendment and paragraph 2.C.(2) of Facility Operating License No. NPF-9 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 5, are hereby incorporated into this license. The licensee shall operate the facility in accordance with the Technical Specifications

and the Environmental Protection Plan.

8110130025 810923  
PDR ADECK 05000369  
PDR

OFFICE	.....	.....	.....	.....	.....	.....	.....
SURNAME	.....	.....	.....	.....	.....	.....	.....
DATE	.....	.....	.....	.....	.....	.....	.....

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

*S/*

Elinor G. Adensam, Acting Chief  
Licensing Branch No. 4  
Division of Licensing

Attachment:  
Technical Specification  
change

Date of Issuance: September 23, 1981

*E.K - ON Amendment  
& Notice of Letter  
ONLY.*

OFFICE	LA:DL:LB#4	DL:LB #4	TOSB	OELD	DL:LB #4	AD:DL	
SURNAME	M.Duncan/hmc	RBirkeJ	RStevens	Ketchen	EAdensam	RTedesco	
DATE	9/23/81	9/23/81	9/23/81	9/23/81	9/23/81	9/23/81	

ATTACHMENT TO LICENSE AMENDMENT NO. 5

FACILITY OPERATING LICENSE NO. NPF-9

DOCKET NO. 50-369

Replace the following page of the Appendix "A" Technical Specifications with the enclosed page. The revised page is identified by Amendment number and contains a vertical line indicating the area of change. The corresponding overleaf page is also provided to maintain document completeness.

Overleaf  
Page

2-6

Amended  
Page

2-5

OFFICE ▶	.....	.....	.....	.....	.....	.....	.....
SURNAME ▶	.....	.....	.....	.....	.....	.....	.....
DATE ▶	.....	.....	.....	.....	.....	.....	.....

TABLE 2.2-1

REACTOR TRIP SYSTEM INSTRUMENTATION TRIP SETPOINTS

<u>FUNCTIONAL UNIT</u>	<u>TRIP SETPOINT</u>	<u>ALLOWABLE VALUES</u>
1. Manual Reactor Trip	Not Applicable	Not Applicable
2. Power Range, Neutron Flux	Low Setpoint - $\leq$ 25% of RATED THERMAL POWER	Low Setpoint - $\leq$ 26% of RATED THERMAL POWER
	High Setpoint - $\leq$ 109% of RATED THERMAL POWER	High Setpoint - $\leq$ 110% of RATED THERMAL POWER
3. Power Range, Neutron Flux, High Positive Rate	$\leq$ 5% of RATED THERMAL POWER with a time constant $\geq$ 2 seconds	$\leq$ 5.5% of RATED THERMAL POWER with a time constant $\geq$ 2 seconds
4. Power Range, Neutron Flux, High Negative Rate	$\leq$ 5% of RATED THERMAL POWER with a time constant $\geq$ 2 seconds	$\leq$ 5.5% of RATED THERMAL POWER with a time constant $\geq$ 2 seconds
5. Intermediate Range, Neutron Flux	$\leq$ 25% of RATED THERMAL POWER	$\leq$ 30% of RATED THERMAL POWER
6. Source Range, Neutron Flux	$\leq$ $10^5$ counts per second	$\leq$ $1.3 \times 10^5$ counts per second
7. Overtemperature $\Delta T$	See Note 1	See Note 3
8. Overpower $\Delta T$	See Note 2	See Note 3
9. Pressurizer Pressure--Low	$\geq$ 1945 psig	$\geq$ 1935 psig
10. Pressurizer Pressure--High	$\leq$ 2385 psig	$\leq$ 2395 psig
11. Pressurizer Water Level--High	$\leq$ 92% of instrument span	$\leq$ 93% of instrument span
12. Turbine Trip		
A. Low System Pressure Trip	$>$ 45 psig	$>$ 42 psig
B. Turbine Stop Valve Closure	$\geq$ 1% open	$\geq$ 1% open

McGUIRE - UNIT 1

2-5

Amendment No. 5

TABLE 2.2-1 (Continued)

REACTOR TRIP SYSTEM INSTRUMENTATION TRIP SETPOINTS

<u>FUNCTIONAL UNIT</u>	<u>TRIP SETPOINT</u>	<u>ALLOWABLE VALUES</u>
13. Loss of Flow	$\geq$ 90% of design flow per loop*	$\geq$ 89% of design flow per loop*
14. Steam Generator Water Level--Low-Low	$\geq$ 12% of span from 0 to 30% of RATED THERMAL POWER, increasing linearly to $\geq$ 54.9% of span at 100% of RATED THERMAL POWER.	$\geq$ 11% of span from 0 to 30% of RATED THERMAL POWER, increasing linearly to $\geq$ 53.9% of span at 100% of RATED THERMAL POWER.
15. Undervoltage-Reactor Coolant Pumps	$\geq$ 5082 volts-each bus	$\geq$ 5016 volts-each bus
16. Underfrequency-Reactor Coolant Pumps	$\geq$ 56.4 Hz - each bus	$\geq$ 55.9 Hz - each bus
17. Safety Injection Input from ESF	Not Applicable	Not Applicable
18. Intermediate Range Neutron Flux - (P-6) Enable Block Source Range Reactor Trip	$\geq 1 \times 10^{-10}$ amps	$\geq 6 \times 10^{-11}$ amps
19. Power Range Neutron Flux (not P-10) Input to Low Power Reactor Trips Block P-7	$\leq$ 10% of RATED THERMAL POWER	$\leq$ 11% of RATED THERMAL POWER
20. Turbine Impulse Chamber Pressure - (P-13) Input to Low Power Reactor Trips Block P-7	$<$ 10% Turbine Impulse Pressure Equivalent	$<$ 11% Turbine Impulse Pressure Equivalent

\* Design flow is 97,500 gpm per loop.

SAFETY EVALUATION BY THE  
OFFICE OF NUCLEAR REACTOR REGULATION  
RELATED TO AMENDMENT NO. 5  
TO LICENSE NPF-9  
DUKE POWER COMPANY

INTRODUCTION

By letter dated September 21, 1981, the licensee (Duke Power Company) proposed a modification to the Technical Specification for the McGuire Nuclear Station, Unit 1, regarding the setpoint associated with the reactor trip initiated by a turbine trip.

The design of the Turbine Trip/Reactor Trip circuit is based on the control oil system which is used to control the Digital Electro-Hydraulic (DEH) system oil pressure through an interface valve. When a turbine trip signal is initiated, a section of the control oil system is bled off which in turn dumps the pressure on the DEH system to close the turbine control and stop valves. This control oil system generates a low pressure signal at 45 psig through 2 out of 3 logic to trip the reactor above the P-8 setpoint. The DEH low pressure trip at 900 psig serves to close the turbine control and stop valves which in turn would trip the reactor through the turbine stop valve closure switch at 1% open.

EVALUATION

The existing Technical Specification, Table 2.2-1, Reactor Trip System Instrumentation Trip Setpoints, item 12.A., specifies a DEH pressure trip setpoint for the turbine control and stop valves. Since the DEH system low pressure trip does not directly feed the reactor trip circuit, the setpoint specified should accurately reflect the operation of the control oil pressure switches which directly initiate the reactor trip as described above.

We agree with the licensee's conclusion that the revised turbine trip low system pressure trip correctly reflects the reactor trip function and does not result in any adverse safety implications.

ENVIRONMENTAL CONSIDERATION

We have determined that the amendment does not authorize a change in effluent types of total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR Section 51.5(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

8110130027 810923  
 PDR ADDCK 05000369  
 PDR

OFFICE ▶	.....	.....	.....	.....	.....	.....	.....
SURNAME ▶	.....	.....	.....	.....	.....	.....	.....
DATE ▶	.....	.....	.....	.....	.....	.....	.....



CONCLUSION

We have concluded, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the amendment does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

DATE: September 23, 1981

OFFICE ▶	.....	.....	.....	.....	.....	.....	.....
SURNAME ▶	.....	.....	.....	.....	.....	.....	.....
DATE ▶	.....	.....	.....	.....	.....	.....	.....

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NO. 50-369

DUKE POWER COMPANY

NOTICE OF ISSUANCE OF AMENDMENT

FACILITY OPERATING LICENSE NO. NPF-9

The U.S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 5 to Facility Operating License No. NPF-9, issued to Duke Power Company (licensee) for the McGuire Nuclear Station, Unit 1 (the facility) located in Mecklenburg County, North Carolina. This amendment corrects the setpoint associated with the reactor trip initiated by a turbine trip. The amendment is effective as of its date of issuance.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations. The Commission has made appropriate findings as required by the Act and the Commission's regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment was not required since the amendment does not involve a significant hazards consideration.

The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR

51.5(d)(4) an environmental impact statement, or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of this amendment.

For further details with respect to this action, see (1) Duke Power Company letter dated September 21, 1981, (2) Amendment No. 5 to Facility Operating License No. NPF-9 with Appendix A Technical Specification page change, and (3) the Commission's related Safety Evaluation.

8110130031 81092  
PDR ADDCK 05000369  
PDR

OFFICE ▶	.....	.....	.....	.....	.....	.....	.....
SURNAME ▶	.....	.....	.....	.....	.....	.....	.....
DATE ▶	.....	.....	.....	.....	.....	.....	.....

All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D. C., and the Atkins Library, University of North Carolina, Charlotte (UNCC Station), North Carolina 28223. A copy of items 2 and 3 may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 23<sup>rd</sup> day of September 1981.

FOR THE NUCLEAR REGULATORY COMMISSION

*S/*

Elinor G. Adensam, Acting Chief  
Licensing Branch No. 4  
Division of Licensing, NRR

\*See previous concurrence

OFFICE ▶	*DL:LB#4	*DL:LB#4	*OELD	<i>DL:LB#4</i>			
SURNAME ▶	9/22/81	9/22/81	9/22/81	9/22/81			
DATE ▶	MDuncan:hmc	RBirkel	EKetchen	EAdensam			

All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D. C., and the Atkins Library, University of North Carolina, Charlotte (UNCC Station), North Carolina 28223. A copy of Amendment No. 5 may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this            day of

FOR THE NUCLEAR REGULATORY COMMISSION

Elinor G. Adensam, Acting Chief  
Licensing Branch No. 4  
Division of Licensing, NRR

*OK - Notice, subject statement  
only to ordering the SE  
to whoever requests  
one at the appropriate  
address.*

OFFICE	LA:DL:LB #4	DL:LB #4	OELD	DL:LB #4			
SURNAME	MDuncan/hmc	RBirke1	KETCHEN	EAdensam			
DATE	9/27/81	9/27/81	9/29/81	9/27/81			

To: - Ralph Birkel  
x 28516

**DUKE POWER COMPANY**

POWER BUILDING  
422 SOUTH CHURCH STREET, CHARLOTTE, N. C. 28212

From: Skip Copp  
(2 pages)

WILLIAM O. PARKER, JR.  
VICE PRESIDENT  
STEAM PRODUCTION

September 21, 1981

TELEPHONE: AREA 704  
373-4083

Mr. Harold R. Denton, Director  
Office of Nuclear Reactor Regulation  
U. S. Nuclear Regulatory Commission  
Washington, D. C. 20555

Attention: Ms. E. G. Adensam, Chief  
Licensing Branch No. 4

Re: McGuire Nuclear Station  
Proposed Amendment to License NFF-9  
Docket No. 50-369

Ralph,

This one needs to  
be expedited. We cannot  
exceed 48 to power until  
this is approved.

Skip

Dear Mr. Denton:

Attached is a proposed change to the McGuire Nuclear Station, Unit 1, Technical Specifications. This change corrects the setpoint associated with the reactor trip initiated by a turbine trip.

This change has been reviewed and it has been determined that there are no adverse safety or environmental impacts associated with the proposed change. The proposed change is considered to be a Class III amendment pursuant to 10 CFR 170.22. Therefore, enclosed is a check in the amount of \$4000.

Very truly yours,

s/William O. Parker, Jr.  
William O. Parker, Jr.

GAC/smh

cc: Ms. M. J. Graham  
Resident Inspector  
McGuire Nuclear Station

Mr. James P. O'Reilly, Director  
U. S. Nuclear Regulatory Commission  
Region II  
101 Marietta Street, Suite 3100  
Atlanta, Georgia 30303

TELECOM-BR-DFOS

1981 SEP 21 PM 3 5

U.S. NUCLEAR REGULATORY  
COMMISSION

## Technical Specification 2.2 - Reactor Trip Instrumentation Setpoints

### Proposed Change

Change Values on Item 12B in Table 2.2-1 to read:

- 12A) Low System Pressure Trip - Trip Setpoint  $\geq$  45 psig  
Allowable Value  $\geq$  42 psig

### Justification and Safety Analysis

*Electric  
Digital Electro-Hydraulic Digital Electro-Hydraulic*

The design of the Turbine Trip/Reactor Trip circuit is based on the control oil system which is used to control DEH system oil pressure through an interface valve. When a turbine trip signal is initiated, a section of the control oil system is bled off which in turn dumps the pressure on the DEH system to close the turbine control and stop valves. This control oil system generates a low pressure signal at 45 psig through 2 out of 3 logic to trip the reactor above the P-8 setpoint. The DEH low pressure trip at 900 psig serves to close the turbine control and stop valves which in turn would trip the reactor through the turbine stop valve closure switch at 1X open. However, the DEH system low pressure trip does not directly feed the reactor trip circuit. Therefore, the setpoint specified should be for the control oil pressure switches which directly initiate the reactor trip.

This proposed change corrects the current Technical Specifications to correctly specify the trip setpoint and as such does not result in any adverse safety implications.

→ M. DONCAR

To: - Ralph Birkel  
x 28516

**DUKE POWER COMPANY**

POWER BUILDING  
422 SOUTH CHURCH STREET, CHARLOTTE, N. C. 28242

From: Skip Copp  
(2 pages)

WILLIAM O. PARKER, JR.  
VICE PRESIDENT  
STEAM PRODUCTION

September 21, 1981

TELEPHONE: AREA 704  
373-4083

Mr. Harold R. Denton, Director  
Office of Nuclear Reactor Regulation  
U. S. Nuclear Regulatory Commission  
Washington, D. C. 20555

Attention: Ms. E. G. Adensam, Chief  
Licensing Branch No. 4

Re: McGuire Nuclear Station  
Proposed Amendment to License NFF-9  
Docket No. 50-369

Ralph,

This one needs to  
be expedited. We cannot  
exceed 48% power until  
this is approved.

Skip

Dear Mr. Denton:

Attached is a proposed change to the McGuire Nuclear Station, Unit 1, Technical Specifications. This change corrects the setpoint associated with the reactor trip initiated by a turbine trip.

This change has been reviewed and it has been determined that there are no adverse safety or environmental impacts associated with the proposed change. The proposed change is considered to be a Class III amendment pursuant to 10 CFR 170.22. Therefore, enclosed is a check in the amount of \$4000.

Very truly yours,

s/William O. Parker, Jr.  
William O. Parker, Jr.

GAC/smh

cc: Ms. M. J. Graham  
Resident Inspector  
McGuire Nuclear Station

Mr. James P. O'Reilly, Director  
U. S. Nuclear Regulatory Commission  
Region II  
101 Marietta Street, Suite 3100  
Atlanta, Georgia 30303

U.S. NUCLEAR REGULATORY  
COMMISSION

1981 SEP 21 PM 3 5

TELECOM-BR-DFO5

8110130034 810923  
PDR ADDCK 05000369  
P PDR

## Technical Specification 2.2 - Reactor Trip Instrumentation Setpoints

### Proposed Change

Change Values on Item 12B in Table 2.2-1 to read:

12A) Low System Pressure Trip - Trip Setpoint  $\geq$  45 psig  
Allowable Value  $\geq$  42 psig

### Justification and Safety Analysis

The design of the Turbine Trip/Reactor Trip circuit is based on the control oil system which is used to control DEH system oil pressure through an interface valve. When a turbine trip signal is initiated, a section of the control oil system is bled off which in turn dumps the pressure on the DEH system to close the turbine control and stop valves. This control oil system generates a low pressure signal at 45 psig through 2 out of 3 logic to trip the reactor above the P-8 setpoint. The DEH low pressure trip at 900 psig serves to close the turbine control and stop valves which in turn would trip the reactor through the turbine stop valve closure switch at 1X open. However, the DEH system low pressure trip does not directly feed the reactor trip circuit. Therefore, the setpoint specified should be for the control oil pressure switches which directly initiate the reactor trip.

This proposed change corrects the current Technical Specifications to correctly specify the trip setpoint and as such does not result in any adverse safety implications.

---



DISTRIBUTION:  
 Docket No. 50-369  
 LB #4 r/f  
 E. Adensam  
 R. Birkel  
 M. Duncan  
 D. Eisenhut  
 E. Ketchen

September 22, 1981

MEMORANDUM FOR: Robert. L. Tedesco, Assistant Director  
 for Licensing  
 Division of Licensing

THRU: Elinor G. Adensam, Acting Chief  
 Licensing Branch No. 4  
 Division of Licensing

FROM: Ralph Birkel, Project Manager  
 Licensing Branch No. 4  
 Division of Licensing

SUBJECT: ISSUANCE OF AMENDMENT NO. 5 TO FACILITY OPERATING  
 LICENSE NPF-9 - McGUIRE NUCLEAR STATION, UNIT 1

Regarding the issuance of the subject amendment, there is no known public  
 correspondence or irreversible impact associated with this subject.

5/  
 Ralph Birkel, Project Manager  
 Licensing Branch No. 4  
 Division of Licensing

8110130038 810923  
 PDR ADDCK 05000369  
 P PDR

OFFICE	DL:LB#4	DL:LB#4	DL:LB#4				
SURNAME	MDuncan:hmc	RBirkel	EAdensam				
DATE	9/22/81	9/22/81	9/22/81				

AMENDMENT NO. 5 TO  
FACILITY OPERATING LICENSE NPF-9 - McGUIRE NUCLEAR STATION, UNIT NO. 1

DISTRIBUTION w/enclosures:

✓ Docket No. 50-369  
LB #4 r/f  
R. Birkel  
M. Duncan  
I&E (5)  
E. Ketchen, OELD  
G. Deegan (4)  
E. Adensam  
MPA  
A. Toalston, DE  
I. Dinitz, DE  
R. Diggs, DE  
D. Eisenhut  
R. Purple  
R. Tedesco  
T. Novak  
F. Miralgia  
B. J. Youngblood  
J. Miller  
A. Schwencer  
S. Hanauer  
R. Vollmer  
R. Mattson  
R. Murley  
NMSS

bcc w/enclosures:

NRC PDR  
Local PDR  
NSIC  
TERA  
A. Rosenthal, ASLAB  
ASLBP  
ACRS (16)  
B. Scharf - 10

