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SI

October 29, 1982

Docket No. 50-369

Mr. H. B. Tucker, Vice President
Nuclear Production Department
Duke Power Company
422 South Church Street
Charlotte, North Carolina 28242

Dear Mr. Tucker:

Subject: Issuance of Amendment No. 18 to Facility Operating License
NPF-9 - McGuire Nuclear Station, Unit 1

The Nuclear Regulatory Commission has issued Amendment No. 18 to Facility Operating License NPF-9 for the McGuire Nuclear Station, Unit 1, located in Mecklenburg County, North Carolina. This amendment is in response to your letter dated October 21, 1982.

The amendment makes changes to the Technical Specifications related to primary containment leakage testing. In connection with this action, the Commission has granted an exemption which allows the licensee to extend by one month the time interval between containment penetration type C leak rate tests in order that it coincide with a scheduled shutdown. This is an exemption from the portion of Paragraph III.D.3 of Appendix J to 10 CFR 50 which states: "Type C tests shall be performed during each reactor shutdown for refueling but in no case at intervals greater than 2 years."

We have reviewed the proposed one-month extension and find that there is no reason to assume that the penetration will degrade beyond the allowable limit by November 30, 1982. We find that granting the proposed exemption from the requirements of Appendix J is authorized by law and will not endanger life or property or the common defense and security and is otherwise in the public interest.

A copy of the related safety evaluation report supporting Amendment No. 18 to Facility Operating License NPF-9 is enclosed. Also enclosed is a copy of a related notice which has been forwarded to the Office of the Federal Register for publication.

Sincerely,

S/R. Purple for

Darrell G. Eisenhut, Director
Division of Licensing
Office of Nuclear Reactor Regulation

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Enclosures:

1. Amendment No. 18
2. Safety Evaluation
3. Federal Register Notice

OFFICE	DL:LB.#4	DL:LB.#4	DL:LB.#4	DL:LB.#4	DL:LB.#4	DL:LB.#4
SURNAME	MDuncan/hmc	RBirke1	EAdensam	INovak	DEisenhut	
DATE	10/26/82	10/26/82	10/26/82	10/26/82	10/27/82	

McGuire

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OFFICE

SURNAME

DATE

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USGPO: 1981-335-960

DUKE POWER COMPANY

DOCKET NO. 50-369

MCGUIRE NUCLEAR STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No.18
License No. NPF-9

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. Pursuant to 10 CFR 50.12 of the Commission's regulations, the Commission has authorized an exemption from the requirements of Appendix J to 10 CFR Part 50 to allow deferral of leak testing of penetration M320 until November 30, 1982;
 - B. The application for amendment to the McGuire Nuclear Station, Unit 1 (the facility) Facility Operating License No. NPF-9 filed by the Duke Power Company (licensee) dated October 21, 1982, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's regulations as set forth in 10 CFR Chapter I;
 - C. The facility will operate in conformity with the application, as amended, the provisions of the Act, and the regulations of the Commission, except as exempted from compliance by paragraph 1.A.;
 - D. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I, except as exempted from compliance by paragraph 1.A.;
 - E. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public;
 - F. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is hereby amended by page changes to the Technical Specifications as indicated in the attachments to this license amendment and paragraph 2.C.(2) of Facility Operating License No. NPF-9 is hereby amended to read as follows:

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OFFICIAL RECORD COPY

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 18, are hereby incorporated into this license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

s/R. Purple for

Darrell G. Eisenhut, Director
Division of Licensing
Office of Nuclear Reactor Regulation

Attachment:
Technical Specification
Changes

Date of Issuance: October 29, 1982

OFFICE	LA:DL:LB #4	DL:LB #4	CSB	OELD	DL:LB #4	AD:L:DL	DIR/DL
SURNAME	MDuncan/hmc	RBirkel	WButler	CUTCHIN	EAdensam	TNovak	DEisenhut
DATE	10/29/82	10/29/82	10/28/82	10/29/82	10/29/82	10/29/82	10/29/82

ATTACHMENT TO LICENSE AMENDMENT NO. 18

FACILITY OPERATING LICENSE NO. NPF-9

DOCKET NO. 50-369

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised page is identified by Amendment number and contains a vertical line indicating the area of change. The corresponding overleaf page is also provided to maintain document completeness.

Amended
Page

3/4 6-7

Overleaf
Page

3/4 6-8

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SURNAME ▶
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TABLE 3.5-1

SECONDARY CONTAINMENT BYPASS LEAKAGE PATHS

<u>PENETRATION NUMBER</u>	<u>SERVICE</u>	<u>RELEASE LOCATION</u>	<u>TEST TYPE</u>
M317	Instrument Air	Auxiliary Building	Type C
M243	Containment Air Release	Auxiliary Building	Type C
M384	Containment Air Addition	Auxiliary Building	Type C
M361	Reactor Coolant Pump Motor Oil Supply	Auxiliary Building	Type C
M353	Fire Protection Header	Auxiliary Building	Type C
M376	Component Cooling Water to Reactor Coolant Drain Tank Heat Exchanger	Auxiliary Building	Type C
M355	Component Cooling Water from Reactor Coolant Drain Tank Heat Exchanger	Auxiliary Building	Type C
M327	Component Cooling Water to Reactor Vessel Support Coolers and RCP Coolers	Auxiliary Building	Type C
M320	Component Cooling Water from Reactor Vessel Support Coolers and RCP Coolers	Auxiliary Building	Type C**
—	Flued Head to Guard Pipe Welds on all Hot Penetrations	Atmosphere, or Auxiliary Building, or Turbine Building	*
	Equipment Hatch	Atmosphere	Type C

*Pursuant to Specification 4.6.1.2.e.

**This penetration is exempted from the Type C leak rate test requirements of specification 4.6.1.2d for the period November 1 through November 30, 1982.

CONTAINMENT SYSTEMS

CONTAINMENT AIR LOCKS

LIMITING CONDITION FOR OPERATION

3.6.1.3 Each containment air lock shall be OPERABLE with:

- a. Both doors closed except when the air lock is being used for normal transit entry and exit through the containment, then at least one air lock door shall be closed, and
- b. An overall air lock leakage rate of less than or equal to $0.05 L_a$ at P_a , 14.8 psig.

APPLICABILITY: MODES 1, 2, 3 and 4.

ACTION:

- a. With one containment air lock door inoperable:
 1. Maintain at least the OPERABLE air lock door closed and either restore the inoperable air lock door to OPERABLE status within 24 hours or lock the OPERABLE air lock door closed.
 2. Operation may then continue until performance of the next required overall air lock leakage test provided that the OPERABLE air lock door is verified to be locked closed at least once per 31 days.
 3. Otherwise, be in at least HOT STANDBY within the next six hours and in COLD SHUTDOWN within the following 30 hours.
 4. The provisions of Specification 3.0.4 are not applicable.
- b. With the containment air lock inoperable, except as the result of an inoperable air lock door, maintain at least one air lock door closed; restore the inoperable air lock to OPERABLE status within 24 hours or be in at least HOT STANDBY within the next six hours and in COLD SHUTDOWN within the following 30 hours.

SURVEILLANCE REQUIREMENTS

4.6.1.3 Each containment air lock shall be demonstrated OPERABLE:

SAFETY EVALUATION REPORT
PROPOSED CHANGE TO TECHNICAL SPECIFICATIONS
AND EXEMPTION REQUEST CONCERNING
APPENDIX J CONTAINMENT LEAKAGE TESTING
MCGUIRE NUCLEAR STATION, UNIT 1

Background

By letter dated October 21, 1982, the licensee proposed a change to the facility Technical Specifications concerning containment leakage testing. Since the proposed change does not comply with the requirements of Appendix J to 10 CFR Part 50, the licensee also requested an exemption from certain requirements of Appendix J.

Technical Specification 4.6.1.2d of the McGuire plant requires that Type C penetration tests "...be conducted at intervals no greater than 24 months". This is based on 10 CFR 50, Appendix J, III.D.3, which requires that Type C penetration tests "...be performed during each reactor shutdown for refueling but in no case at intervals greater than 2 years."

McGuire, Unit 1 had previously been scheduled for shutdown in October 1982 to install a design modification on the Model D Westinghouse steam generators. Recent delays in delivery of the special tooling required for this modification have resulted in a delay of the scheduled shutdown until November 1982. Therefore, the licensee is proposing a change to the Technical Specifications (and thus also requesting an exemption from the requirements of Appendix J) to allow penetration number M320 to be leak tested no later than November 30, 1982. This would allow a 1-month extension of the test interval from 24 months to 25 months.

For several penetrations, the unit needs to be in cold shutdown during performance of the Type C leakage tests. These penetrations were last tested in 1980 and, therefore, are required by Appendix J to be tested in 1982. Previous shutdowns for steam generator eddy current inspections provided an opportunity to test penetrations which were tested mid-year 1980. At the time of the last shutdown in July 1982, a review was performed by the licensee of all surveillance requiring a unit shutdown to ensure the unit could operate until the next shutdown, which at that time was scheduled for September 1982. Penetration M320 was not leak tested during this July outage since it was believed the next shutdown in 1982 would occur no later than the expiration of the interval for testing of this penetration. Other penetrations with earlier test dates were tested during this outage.

EVALUATION

Penetration M320 is the component cooling return line for components in the reactor building, including the reactor coolant pump motor coolers. Testing of this penetration requires isolating and draining this portion of the line, and thus requires isolation of component cooling flow to the affected components. Therefore, Unit 1 would have to be shut down and cooled down in order to switch to residual heat removal, which would allow isolation of component cooling to the

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reactor coolant pump motor coolers. The licensee's current estimate for shut-down, cooldown, penetration testing, and heatup is approximately 72 hours. More importantly, the reactor coolant system would be subjected to a thermal cycle, only for the purpose of obtaining test results a few weeks earlier than otherwise would be available.

The licensee states, and we agree, that acceptance of a small increase in the surveillance interval when compared with the unnecessary thermal cycle of cooldown and heatup represents responsible and prudent concern for the safety of plant operations. The next outage planned to begin in mid-November allows more than sufficient time to complete this penetration test and all other required surveillance for 1982.

The component cooling system penetration M320 (component cooling water from reactor vessel support coolers and RCP coolers) was previously tested in May 1979 and October 1980. The results from these prior tests were 40 sccm and 45 sccm, respectively. The total allowable leakage for bypass penetrations is specified as 0.07 La or 6343 sccm at 14.8 psig. The average maximum allowable leakage for each of the 69 penetrations would thus be 92 sccm. Therefore, penetration M320 has previously exhibited leakage rates less than half the maximum allowable for individual penetrations. The latest results for leakage rates of all penetrations show a current total of 2187 sccm which is well below the allowable limit of 6343 sccm.

The past history shows the leak rate of the penetration to be well within the allowable limit and there is no reason to assume that penetration M320 will degrade beyond the allowable limit by November 30, 1982.

CONCLUSION

Based on our review of the licensee's submittal, we conclude that this limited extension in the surveillance period, as proposed, does not represent a significant increase in the risk to the health and safety of the public, and is acceptable. We also conclude that a temporary exemption to the requirements of Appendix J, to permit the proposed change to the Technical Specifications, is justified and should be granted.

ENVIRONMENTAL CONSIDERATION

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR Section 51.5(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

CONCLUSION

We have concluded, based on the consideration discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of accidents previously considered, does not create the possibility of an accident of a type different from any evaluated previously and does not

involve a significant decrease in a safety margin, the amendment does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Date: October 29, 1982

Principal Contributors: J. Pulsipher, CSB
R. Birkel, LB #4

UNITED STATES NUCLEAR REGULATORY COMMISSIONDOCKET NO. 50-369DUKE POWER COMPANYNOTICE OF ISSUANCE OF AMENDMENTFACILITY OPERATING LICENSE NO. NPF-9

The U.S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 18 to Facility Operating License No. NPF-9, issued to Duke Power Company (licensee) for the McGuire Nuclear Station, Unit 1 (the facility) located in Mecklenburg County, North Carolina. The amendment is effective as of its date of issuance.

The amendment permits a one-month extension of the time interval between containment penetration type C leak rate tests in order that it coincide with a scheduled shutdown. In connection with this action, the Commission has granted an exemption to 10 CFR 50, Appendix J, "Primary Reactor Containment Leakage Testing For Water-Cooled Power Reactors."

Issuance of this amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations. The Commission has made appropriate findings as required by the Act and the Commission's regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment was not required since the amendment does not involve a significant hazards consideration.

The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement, or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of this

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For further details with respect to this action, see (1) Duke Power Company letter dated October 21, 1982, (2) Amendment No. 18 to Facility Operating License No. NPF-9 and (3) the Commission's related Safety Evaluation.

These items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D. C., and the Atkins Library, University of North Carolina, Charlotte (UNCC Station), North Carolina 28223. A copy of these items may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 29th day of October 1982.

FOR THE NUCLEAR REGULATORY COMMISSION

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Elinor G. Adensam, Chief
Licensing Branch No. 4
Division of Licensing, NRR

No legal objection
to Form of FR

OFFICE	LA:DL:LB #4	DL:LB #4	OELD	DL:LB #4			
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October 29, 1982

Docket No. 50-369

MEMORANDUM FOR: Thomas M. Novak, Assistant Director
for Licensing
Division of Licensing

THRU: Elinor G. Adensam, Chief
Licensing Branch No. 4
Division of Licensing

FROM: Ralph A. Birkel, Project Manager
Licensing Branch No. 4
Division of Licensing

SUBJECT: ISSUANCE OF AMENDMENT NO. 18 TO FACILITY OPERATING
LICENSE NPF-9 McGUIRE NUCLEAR STATION, UNIT 1

There is no known public correspondence or irreversible impact associated
with the issuance of the subject amendment.

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Ralph A. Birkel, Project Manager
Licensing Branch No. 4
Division of Licensing

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DATE	10/28/82	10/28/82	10/29/82				

AMENDMENT NO. 18 TO FACILITY OPERATING LICENSE NPF-9 - McGUIRE NUCLEAR STATION, UNIT 1

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Meeting Title: Status of Gimmes Investigation
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MEETING DATE:

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