

JUN 19 1981

Docket No. 50-369

Mr. William O. Parker, Jr.
Vice President, Steam Production
Duke Power Company
P.O. Box 2178
422 South Church Street
Charlotte, North Carolina 28242

Dear Mr. Parker:

SUBJECT: ISSUANCE OF AMENDMENT NO. 1 TO FACILITY OPERATING LICENSE NPF-9 -
MCGUIRE NUCLEAR STATION, UNIT 1

The Nuclear Regulatory Commission has issued Amendment No. 1 to Facility Operating License NPF-9 in accordance with your letter, dated June 17, 1981. A copy of this amendment is enclosed. The amendment revises certain pages of the Appendix A Technical Specifications to perform testing requirements of the diesel generator and to correct a listing of fire detection instrumentation.

Also enclosed is a copy of our Safety Evaluation supporting this change and the public notice concerning the issuance of this amendment. The notice has been forwarded to the Office of the Federal Register for publication.

Sincerely,

for *R. S. Gabbour*
Elinor G. Adensam, Acting Chief
Licensing Branch No. 4
Division of Licensing

Enclosures:

1. Amendment No. 1 to Facility Operating License NPF-9 w/Tech. Spec. Appendix A page changes
2. Safety Evaluation
3. Federal Register Notice

8106300014

E.K. subject comments on "Amendment" & "S.E."

CPZ

OFFICE	DL:LB#A/NA	DL:LB.#4	OELD	DL:LB.#4			
SURNAME	MRusbrook	RBirkel/hmc	EKetchen	EAdensam			
DATE	6/19/81	6/19/81	6/19/81	6/19/81			

Yellow

AMENDMENT NO. 1 TO
FACILITY OPERATING LICENSE NPE 9 - MCGUIRE NUCLEAR STATION, UNIT NO. 1

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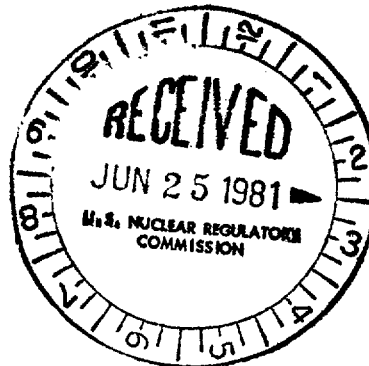
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McGuire

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ATTN: Ms. Elizabeth V. Jankus
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DUKE POWER COMPANY

DOCKET NO. 50-369

MCGUIRE NUCLEAR STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Facility Operating License No. NPF-9
Amendment No. 1

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Duke Power Company (Licensee), dated June 17, 1981, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the license, as amended, the provisions of the Act, and the regulations of the Commission;
 - C. There is reasonable assurance (1) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
 - F. Prior public notice of this amendment was not required since it does not involve a significant hazards consideration.

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OFFICE ▶
SURNAME ▶
DATE ▶

DUKE POWER COMPANY

DOCKET NO. 50-369

MCGUIRE NUCLEAR STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Facility Operating License No. NPF-9
Amendment No. 1

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 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
 - F. Prior public notice of this amendment was not required since it does not involve a significant hazards consideration nor ~~amendment of a license~~

*Omit; this is amendment. § 2.106(a)(2)
References 10 CFR 50, 52 which reference
§ 103 of AEA. This is 50, 52 which reference*

~~of the type described in 10 CFR Section 2.106(a)(2).~~

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2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment. Facility Operating License No. NPF-9 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 1 are hereby incorporated into this license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

K. Jabbo
for Elinor G. Adensam, Acting Chief
Licensing Branch No. 4
Division of Licensing

Attachment:
Page changes to Technical Specifications, Appendix A

Date of Issuance: JUN 19 1981

Text - subject to compliance with comments with interlinked

OFFICE	LA:LB #4:DL	DL:LB #4	DST/LEGB	OELD	DL:LB #4		
SURNAME	MR. <i>MR</i> <i>Brook</i>	RBirkel/hmc	W. Hatmore	E. Ketchen	E. Adensam		
DATE	6/19/81	6/18/81	6/ /81	6/19/81	6/19/81		

ATTACHMENT TO LICENSE AMENDMENT NO. 1

FACILITY OPERATING LICENSE NO. NPF-9

DOCKET NO. 50-369

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change.

Pages

3/4 3-54

3/4 3-55

3/4 8-5

TABLE 3.3-11 (Continued)

FIRE DETECTION INSTRUMENTS

Fire Zone	Description	Location	MINIMUM INSTRUMENTS OPERABLE ² :	
			Smoke	Fixed Temperature/ Rate of Rise
79	Safety Inj. Pump 1B	GG-54 EL. 716	2	2
80	Aisle/Cables	GG-55 EL. 716	12	12
91	Corridor/Cables	EE-53 EL. 733	4	4
92	Corridor/Cables	JJ-51 EL. 733	6	6
93	Corridor/Cables	NN-52 EL. 733	11	11
94	Aisle/Cables	JJ-55 EL. 733	8	8
95	600V MCC IEMXB-IEMXB3	FF-55 EL. 733	1	1
96	Cable Tray Access	EE-55 EL. 733	1	1
103	Corridor/Cables	MM-51 EL. 750	6	6
104	Hatch Area Cables	LL-53 EL. 750	7	7
106	600V MCC IEMXA	FF-54 EL. 750	2	2
108	Aisle/Cables	JJ-55 EL. 750	13	13
115	Corridor/Cables	JJ-54 EL. 767	13	13
116	HVAC Equip. Area/Cables	NN-52 EL. 767	7	7
130	Cables/Oil Fuel Pool	PP-52 EL. 750	4	4
134	RB Pipe Corridor	215° - 270°	5	0
135	RB Pipe Corridor	270° - 315°	5	0
136	RB Pipe Corridor	315° - 0°	6	0
137	RB Pipe Corridor	0° - 44°	4	0
138	RB Pipe Corridor	44° - 90°	4	0
139	RB Pipe Corridor	90° - 126°	4	0
140	RB Pipe Corridor	126° - 173°	7	0
141	RB Below Oper. Floor	329° - 349°	7	0
142	RB Below Oper. Floor	13° - 29°	4	0
143	RB Below Oper. Floor	34° - 51°	3	0
144	RB Below Oper. Floor	51° - 124°	13	0
145	RB Below Oper. Floor	124° - 143°	3	0
146	RB Below Oper. Floor	143° - 167°	8	0
147	RB Below Oper. Floor	RCP - 1A	4	1

TABLE 3.3-11 (Continued)
FIRE DETECTION INSTRUMENTS

Fire Zone	Description	Location	MINIMUM INSTRUMENTS OPERABLE*	
			Smoke	Fixed Temperature/ Rate of Rise
148	RB Below Oper. Floor	RCP - 1B	1	1
149	RB Below Oper. Floor	RCP - 1C	3	1
150	RB Below Oper. Floor	RCP - 1D	4	1
151	RB Below Oper. Floor	VP Filter Bed	2	2
152	RB Below Oper. Floor	VP Filter Bed	2	2
153	RB Annulus	293° - 331°	10	0
154	RB Annulus	324° - 0°	4	0
155	RB Annulus	0° - 50°	5	0
156	RB Annulus	50° - 80°	4	0
157	RB Annulus	80° - 123°	24	0
158	RB Annulus	123° - 165°	22	0
159	RB Annulus	333° - 16°	13	0
160	RB Annulus	16° - 54°	23	0
161	RB Annulus	122° - 180°	16	0
162	RB Annulus	180° - 256°	13	0
127	Fuel Pool Area	MM-50, EL. 731+6	10	10

*The fire detection instruments located within the containment are not required to be operable during the performance of Type A Containment Leakage Rate Tests.

ELECTRICAL POWER SYSTEMS

SURVEILLANCE REQUIREMENTS (Continued)

8. Verifying the diesel generator operates for at least 24 hours. During the first 2 hours of this test, the diesel generator shall be loaded to greater than or equal to 4400 kw and during the remaining 22 hours of this test, the diesel generator shall be loaded to greater than or equal to 4000 kw. The generator voltage and frequency shall be at least 4160 volts and 57 Hz within 11 seconds after the start signal. The steady state generator voltage and frequency shall be maintained within 4160 ± 420 volts and 60 ± 1.2 Hz during this test. Within 5 minutes after completing this 24-hour test, perform Specification 4.8.1.1.2.d.7.b.*
9. Verifying that the auto-connected loads to each diesel generator do not exceed the 2-hour rating of 4400 kw.
10. Verifying the diesel generator's capability to:
 - a) Synchronize with the offsite power source while the generator is loaded with its emergency loads upon a simulated restoration of offsite power,
 - b) Transfer its loads to the offsite power source, and
 - c) Be restored to its standby status.
11. Verifying that with the diesel generator operating in a test mode (connected to its bus), a simulated safety injection signal overrides the test mode by (1) returning the diesel generator to standby operation and (2) automatically energizing the emergency loads with offsite power.
12. Verifying that the fuel transfer pump transfers fuel from each fuel storage tank to the day tank of each diesel via the installed cross-connection lines.
13. Verifying that the load sequencing times are within the tolerances shown in Table 4.8-2.
14. Verifying that the following diesel generator lockout features prevent diesel generator starting only when required:
 - a) turning gear engaged,
 - b) emergency stop.

*This test shall be performed at the first refueling outage and at least once per 18 months thereafter. This is a one time change to plant operations prior to initial criticality.

1000 1 1 80

SAFETY EVALUATION BY THE
OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 1
TO LICENSE NPF-9
DUKE POWER COMPANY

Introduction

By letter dated June 17, 1981 the licensee requested certain revisions to the McGuire Nuclear Station Unit 1 Technical Specifications, Appendix A, to License NPF-9. These revisions are:

1. Revision to a diesel generator testing requirements.
2. Correction to listing of fire detection instrumentation.

The McGuire Unit 1 Technical Specifications were issued on June 12, 1981 as an integral part of the facility operating license and are standard Westinghouse-PWR technical specifications reflecting plant specific design.

Evaluation

The McGuire Nuclear Station utilizes the standard Westinghouse-PWR technical specifications and as such have been developed to reflect the McGuire design features.

With respect to item (1), the requirement (T.S.4.8.1.1.2.d.8) to perform a test loading of the diesel generator with design accident loads within 5 minutes after completion of a 24 hour test run is a requirement reflecting the staff position regarding diesel generator testing of Regulatory Guide 1.108, "Periodic Testing of Diesel Generator Units as Onsite Electric Power Systems at Nuclear Power Plants". The diesel generator should be so tested during the plant preoperational test program and at least once every 18 months. A successful test loading of both McGuire Unit 1 diesel generators with accident loads was performed during preoperational testing. The technical specification requirement has been revised to reflect the staff testing requirements.

Item (2) relates to fire detection instruments that are required to be operable in each fire zone. The number and location of smoke detectors and fixed temperature/rate of rise detectors is governed by criteria and requirements of National Fire Protection Association Code 72E-1974. Only one detector is required for each unobstructed fire zone. Additional detectors have been added to compensate for obstructions caused by the amount of cable located in each fire zone. We conclude that the proposed revision is in compliance with the foregoing and is acceptable.

810630 002*

Environmental Consideration

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR Section 51.5(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the amendment does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

DATE: JUN 19 1981

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NO. 50-369

DUKE POWER COMPANY

NOTICE OF ISSUANCE OF AMENDMENT TO

FACILITY OPERATING LICENSE NO. NPF-9

The U.S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 1 to Facility Operating License No. NPF-9. This amendment was issued to Duke Power (licensee) for diesel generator testing requirements and a correction to listing of fire detection instrumentation. The McGuire Nuclear Station, Unit 1 is located near Charlotte, North Carolina in Mecklenburg County. This amendment is effective as of its date of issuance.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's requirements. The Commission has made appropriate findings as required by the Act and the Commission's regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment was not required since the amendment does not involve a significant hazards considerations.

The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR Section 51.5(d)(4) an environmental impact statement, or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of this amendment.

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OFFICE ▶
SURNAME ▶
DATE ▶

For further details with respect to this action, see (1) the application for amendment, dated June 17, 1981; (2) Facility Operating License NPF-9, dated June 12, 1981; and (3) the Commission's related Safety Evaluation. All of these documents are available for public inspection at the Commission's Public Document Room, located at 1717 H Street, N.W., Washington, D. C. 20555 and at the Atkins Library, University of North Carolina, Charlotte (UNCC Station) North Carolina 28223 or may be requested by writing to U.S. Nuclear Regulatory Commission, Washington, D. C. 20555 Attention: Director, Technical Information and Document Control.

Dated at Bethesda, Maryland, this _____ day of June 1981.

FOR THE NUCLEAR REGULATORY COMMISSION

15/
K. Jabbour for
Elinor G. Adensam, Acting Chief
Licensing Branch No. 4
Division of Licensing

*OK-subj. to
comments on
AMENDMENT &
Safety Evaluation*

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DATE ▶	6/19/81	6/18/81	6/19/81	6/19/81			