NRC's REVISED REACTOR OVERSIGHT PROCESS

The new process takes into account improvements in the performance of the nuclear industry over the past 25 years and improved approaches of inspecting and assessing safety performance at NRC licensed plants.

The new process monitors licensee performance in three broad areas (called strategic performance areas):

Reactor Safety: avoiding accidents and reducing the consequences of accidents if they occur.

Radiation Safety: protecting plant employees and the public during routine operations.

Safeguards: Protecting the plant against sabotage or other security threats.

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REACTOR SAFETY

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Initiating Events Mitigating Systems Barrier Integrity Emergency Preparedness

TWO ASSESSMENT PROCESSES

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Performance Indicators

Inspection

REPORT 2000-009

"During 2R11 the inspectors observed portions of nondestructive examination (NDE) activities performed on the Unit 2 steam generators and the safety injection system. Additionally, the inspectors reviewed several procedures and reports associated with eddy current inspection of the Salem steam generators, including the degradation assessment report, a signal/noise ratio study, and eddy current examination procedures. Finally, the inspectors reviewed several ASME Section XI code repair/replacement packages on Units 1 and 2, as well as radiographs on the following Unit 1 components: S1-VEN-2-1-A, S1-VEN-2-2-A, and S1-14SW57-TD-3813-1. The steam generator review included: (1) interviewing PSEG Nuclear and contract eddy current NDE personnel to assess their knowledge of steam generator degradation phenomena, (2) reviewing samples of eddy current inspection data obtained from several tubes, and (3) verifying that approved probes were used to acquire eddy current inspection data. The inspectors reviewed data from the following tubes:

STEAM GENERATOR	ROW	COLUMN
21	29	71
21	22	30
22	22	30
22	42	65
23	2	3
23	3	29
23	2	45

Regarding the NDE performed on the safety injection system, the inspectors verified that the technicians used proper technique when performing the examinations, reported indications when necessary, and calibrated the ultrasonic testing instruments in accordance with PSEG Nuclear procedures. The inspectors observed both ultrasonic and liquid penetrant NDE on weld 6-SJ-1222-8. PSEG Nuclear did not perform any non-code repairs on safety-related equipment at Units 1 and 2 over the last twelve months that required NRC relief from the ASME code. Therefore, non-code repair activities were not reviewed during this inspection. Finally, several evaluations associated with findings from the NDE program were reviewed."

Documents Reviewed

Steam Generator Tubing Degradation Assessment EPRI Appendix H Technique Qualification S2.SG-ST.RCE-0001(Q), Rev 3 Salem Unit 2 Steam Generator Eddy Current Examination