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AUG 17 1982

Docket No. 50-364

Mr. F. L. Clayton  
Senior Vice President  
Alabama Power Company  
Post Office Box 2641  
Birmingham, Alabama 35291

Dear Mr. Clayton:

The Commission has issued the enclosed Amendment No. 15 to Facility Operating License No. NPF-8 for the Joseph M. Farley Nuclear Plant, Unit No. 2. The amendment consists of changes to the Technical Specifications in response to your telecopy request transmitted by letter dated April 28, 1982, as confirmed by letter of the same date. This amendment was authorized by telephone on April 28, 1982 and was confirmed by our letter dated April 30, 1982.

The amendment modifies the Technical Specifications to extend a six hour time limit to nine hours for one time only to allow time to complete repairs and retests of the rod control system.

Copies of the Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

Original signed by:

Edward A. Reeves, Project Manager  
Operating Reactors Branch No. 1  
Division of Licensing

Enclosures:

- 1. Amendment No. 15 to NPF-8
- 2. Safety Evaluation
- 3. Notice of Issuance

cc w/enclosures:  
See next page

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*revised objection  
DF to or note*

OFFICE	ORB 1	ORB 1	ORB 1	AD-OR	OELD		
SURNAME	CParrish	EReeves/rs	SVarga	GLainas	J. S. NIXON		
DATE	8/6/82	8/6/82	8/9/82	8/9/82	8/11/82		

Mr. F. L. Clayton  
Alabama Power Company

cc: Mr. W. O. Whitt  
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U. S. Nuclear Regulatory Commission  
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Atlanta, Georgia 30303



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

ALABAMA POWER COMPANY

DOCKET NO. 50-364

JOSEPH M. FARLEY NUCLEAR PLANT, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 15  
License No. NPF-8

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The request for amendment by Alabama Power Company (the licensee) dated April 28, 1982, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

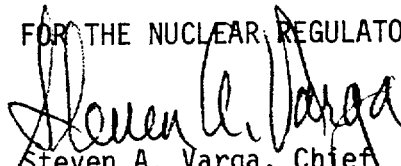
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-8 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 15, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective on April 28, 1982 for one time only.

FOR THE NUCLEAR REGULATORY COMMISSION



Steven A. Varga, Chief  
Operating Reactors Branch No. 1  
Division of Licensing

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance:

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 15 TO FACILITY OPERATING LICENSE NO. NPF-8

DOCKET NO. 50-364

Revise Appendix A as follows:

Remove Pages

3/4 1-14

Insert Pages

3/4 1-14

## REACTIVITY CONTROL SYSTEMS

### 3/4.1.3 MOVABLE CONTROL ASSEMBLIES

#### GROUP HEIGHT

#### LIMITING CONDITION FOR OPERATION

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3.1.3.1 All full length (shutdown and control) rods shall be OPERABLE and positioned within  $\pm 12$  steps (indicated position) of their group step counter demand position.

APPLICABILITY: MODES 1\* and 2\*.

#### ACTION:

- a. With one or more full length rods inoperable due to being immovable as a result of excessive friction or mechanical interference or known to be untrippable, determine that the SHUTDOWN MARGIN requirement of Specification 3.1.1.1 is satisfied within 1 hour and be in HOT STANDBY within 6 hours.
- b. With more than one full length rod inoperable or misaligned from the group step counter demand position by more than  $\pm 12$  steps (indicated position), be in HOT STANDBY within 6# hours.
- c. With one full length rod inoperable due to causes other than addressed by ACTION a, above, or misaligned from its group step counter demand height by more than  $\pm 12$  steps (indicated position), POWER OPERATION may continue provided that within one hour either:
  1. The rod is restored to OPERABLE status within the above alignment requirements, or
  2. The remainder of the rods in the group with the inoperable rod are aligned to within  $\pm 12$  steps of the inoperable rod within one hour while maintaining the rod sequence and insertion limits of Figures 3.1-1 and 3.1-2; the THERMAL POWER level shall be restricted pursuant to Specification 3.1.3.6 during subsequent operation, or
  3. The rod is declared inoperable and the SHUTDOWN MARGIN requirement of Specification 3.1.1.1 is satisfied. POWER OPERATION may then continue provided that:
    - a) A reevaluation of each accident analysis of Table 3.1-1 is performed within 5 days; this reevaluation shall confirm that the previously analyzed results of these accidents remain valid for the duration of operation under these conditions.

\*See Special Test Exceptions 3.10.2 and 3.10.3.

#One-time only exception allowing time for being in HOT STANDBY to be extended from 6 to 9 hours for completion of surveillance testing on April 28, 1982.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
RELATED TO AMENDMENT NO. 15 TO FACILITY OPERATING LICENSE NO. NPF-8

ALABAMA POWER COMPANY

JOSEPH M. FARLEY NUCLEAR PLANT, UNIT NO. 2

DOCKET NO. 50-364

Introduction

By letter dated April 28, 1982, which confirmed an Alabama Power Company (APCo) telecopy request of the same day, APCo requested a one-time Technical Specification change to License No. NPF-8 for Farley Unit 2. The change would allow an additional three hours outage time for certain rod control system position indicators.

Background

Technical Specification 3.1.3.1 requires that full length (shutdown and control) rods be operable and positioned within + 12 steps (indicated position) of their group step counter demand position. Action Statement b. requires the plant to be in HOT STANDBY within 6 hours if Technical Specification 3.1.3.1 is not met. During surveillance tests late on the night of April 26, 1982 electrical circuitry problems arose which precluded insertion of the Group 1 rods of Control Banks A and C. Troubleshooting and retesting would be required to resolve the problem.

Alabama Power Company (APCo) personnel notified the NRC staff by telephone of the problem during the early morning of April 28, 1982. Subsequently, by telecopy letter dated April 28, 1982, APCo requested a one-time extension of the Limiting Condition of Operation 3.1.3.1 Action Statement b. from six hours to nine hours to allow enough time to continue troubleshooting of the electrical circuitry, make repairs and retest the system.

Discussion and Evaluation

The NRC staff evaluated the information provided by APCo and determined the following:

1. Our confidence was high that the control rods could be tripped, if required, either automatically or manually. This confidence was based on APCo's report that only the electrical circuitry for the manual stepping of rods in the inward direction was affected. The slave cyclor step counter card was later found to be the defective component, thus, reactor trip capability existed as we analyzed previously.

2. The rods being tested are normally in the fully withdrawn position and do not affect core power distribution. The only time the system would be needed would be for a controlled manual shutdown by rod insertion. Such action was not planned at this time.
3. Troubleshooting had been underway for several hours when it was found that more time than six hours allowed by TS's would be required to repair and retest. The problem had been pinpointed to only Group 1 rods of three rod groups (Shutdown A, Control Banks A and B) which failed to move inward upon demand.
4. We had confidence that repairs and retests would be completed within a maximum of three hours beyond the 6-hour limit. For the short period of time involved, we considered that the risk to the health and safety of the public was unaffected.

Based on the considerations noted above, and since additional shutdown capability existed by boron dilution, we conclude that there was no undue risk to the health and safety of the public by this action. Our action at this time is only for record purposes to document in the license the authorization previously given verbally on April 28, 1982 and documented by letter dated April 30, 1982.

#### Environmental Consideration

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

#### Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of an accident previously evaluated, does not create the possibility of an accident of a type different from any evaluated previously, and does not involve a significant reduction in a margin of safety, the amendment does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Date: August 17, 1982

Principal Contributor:  
E. A. Reeves



UNITED STATES NUCLEAR REGULATORY COMMISSIONDOCKET NO. 50-364ALABAMA POWER COMPANYNOTICE OF ISSUANCE OF AMENDMENT TO FACILITY  
OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 15 to Facility Operating License No. NPF-8 issued to Alabama Power Company (the licensee), which revised Technical Specifications for operation of the Joseph M. Farley Nuclear Plant, Unit No. 2 (the facility) located in Houston County, Alabama. The amendment was effective on April 28, 1982 for one-time only.

The amendment modifies the Technical Specifications for one time to extend a six hour time limit to nine hours to allow time to complete repairs and retests of the rod control system. The amendment was authorized on an expedited basis to maintain the plant at steady condition and avoid a shutdown transient shown by our evaluation to be unnecessary but required by the Technical Specifications unless amended.

The request for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment was not required since this amendment does not involve a significant hazards consideration.

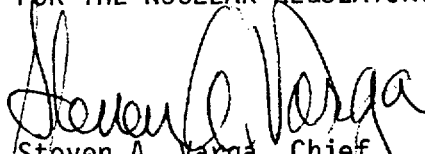
- 2 -

The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of this amendment.

For further details with respect to this action, see (1) the request for amendment dated April 28, 1982, (2) the Commission's letter dated April 30, 1982, (3) Amendment No. 15 to License No. NPF-8, and (4) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N. W., Washington, D. C. and at the George S. Houston Memorial Library, 212 W. Burdeshaw Street, Dothan, Alabama 36303. A copy of items (2), (3) and (4) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 17th day of August 1982.

FOR THE NUCLEAR REGULATORY COMMISSION



Steven A. Varga, Chief  
Operating Reactors Branch No. 1  
Division of Licensing

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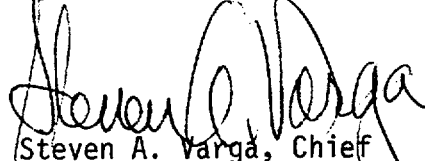
- 2 -

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Dated at Bethesda, Maryland, this 17th day of August 1982.

FOR THE NUCLEAR REGULATORY COMMISSION



Steven A. Varga, Chief  
Operating Reactors Branch No. 1  
Division of Licensing