Distribution for Amendment No. 1 to Facility Operating License NPF-8

June 18, 1981

Docket File 50-364
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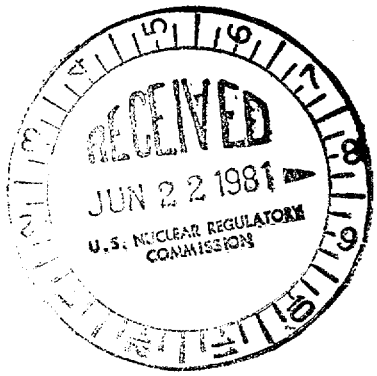

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OPY

Docket No.: 50-364

JUN 15 1981

Mr. F. L. Clayton, Jr.
Senior Vice President
Alabama Power Company
Post Office Box 2641
Birmingham, Alabama 35291

Dear Mr. Clayton:

Subject: Issuance of Amendment No. 1 to Facility Operating License NPF-8 - Joseph M. Farley Nuclear Plant, Unit 2

The Nuclear Regulatory Commission has issued Amendment No. 1 to Facility Operating License No. NPF-8 in response to your application dated May 5, 1981.

A copy of this amendment is enclosed. The amendment revises Appendix A Technical Specifications Surveillance Requirements 4.8.1.1.2.C.4.b) and 4.8.1.1.2.C.6.b) and corrects typographical errors as stated in the enclosed page changes.

This amendment was effective May 6, 1981. Telephone authorization was given for this amendment on May 6, 1981 and was confirmed by letter dated May 6, 1981.

A copy of the amendment with Appendix A Technical Specification page changes, the supporting Safety Evaluation and a related notice, which has been forwarded to the Office of the Federal Register for publication are enclosed.

Sincerely,

/S/

B. J. Youngblood, Chief
Licensing Branch No. 1
Division of Licensing

Enclosures:
1. Amendment No. 1 to NPF-8 w/Appendix A Technical Specification page changes
2. Safety Evaluation
3. Federal Register Notice

cc: See next page

P

8106240326

no legal objection

OFFICE	DL:LB#1	DL:LB#1	OELD	DL:LB#1			
SURNAME	MRushbrook/lg	JThoma	S. Tuby	BJYoungblood			
DATE	6/2/81	6/2/81	6/9/81	6/15/81			

NRC FORM 318 (10/80) NRCM 0240 OFFICIAL RECORD COPY * USGPO: 1980—329-824

Mr. F. L. Clayton, Jr., Senior Vice President
Alabama Power Company
Post Office Box 2641
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JUN 15 1981

cc: Mr. W. O. Whitt
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JUN 15 1981

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State Health Officer
State Dept. of Public Health
State Office Building
Montgomery, Alabama 36104

Honorable A. A. Middleton
Chairman
Houston County Commission
Dothan, Alabama 36301

U.S. Environmental Protection Agency
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Atlanta, Georgia 30308

Attorney General
State Capitol
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ALABAMA POWER COMPANY

DOCKET NO. 50-364

JOSEPH M. FARLEY NUCLEAR PLANT, UNIT 2

FACILITY OPERATING LICENSE

Amendment No. 1
License No. NPF-8

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Alabama Power Company (the licensee), dated May 5, 1981, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the license, as amended, the provisions of the Act, and the regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

8106240327

OFFICE							
SURNAME							
DATE							

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment. Facility Operating License No. NPF-8 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 1, are hereby incorporated into this license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment was effective May 6, 1981.

FOR THE NUCLEAR REGULATORY COMMISSION

/S/

B. J. Youngblood, Chief
Licensing Branch No. 1
Division of Licensing

Date of Issuance: JUN 15 1981

Enclosure:
Revised pages to Appendix A
Technical Specifications

*SEE PREVIOUS YELLOW FOR CONCURRENCE.

OFFICE	DL:LB#1*	DL:LB#1*	OELD	DL:LB#1			
SURNAME	MRushbrook/lgJThoma		STreby	BJYoungblood			
DATE	6/8/81	6/9/81	6/9/81	6/15/81			

F. Prior public notice of this amendment was not required since it does not involve a significant hazards consideration nor amendment of a license of the type described in 10 CFR Section 2.106 (a)(2).

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment. Facility Operating License No. NPF-8 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 1, are hereby incorporated into this license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment was effective May 6, 1981.

FOR THE NUCLEAR REGULATORY COMMISSION

B. J. Youngblood, Chief
Licensing Branch No. 1
Division of Licensing

Date of Issuance:

Enclosure:
Revised pages to Appendix A
Technical Specifications

OFFICE	DL:LB#1	DL:LB#1	OELD	DL:LB#1			
SURNAME	MRushbrook/lg	JThoma	STreby	BJYoungblood			
DATE	6/2/81	6/2/81	6/3/81	6/ /81			

ATTACHMENT TO LICENSE AMENDMENT NO. 1

FACILITY OPERATING LICENSE NPF-8

DOCKET NO. 50-364

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change.

PAGE	REVISIONS
3/4	8-4
3/4	8-5

PAGE CHANGES FOR TYPOGRAPHICAL ERRORS

XII	
3/4	3-39
3/4	3-60a
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LIMITING CONDITIONS FOR OPERATION AND SURVEILLANCE REQUIREMENTS

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BASES

TABLE 3.3-6

RADIATION MONITORING INSTRUMENTATION

	INSTRUMENT	MINIMUM CHANNELS OPERABLE	APPLICABLE MODES	ALARM/TRIP SETPOINT	MEASUREMENT RANGE	ACTION
1.	AREA MONITORS					
	a. Fuel Storage Pool Area (R-5)	1	(a)	$\leq$ 15 mR/hr	10^{-1}-10^{4} mR/hr	23
	b. Containment Area (R-27A&B)	2	1,2,3,4	N/A	1 - 10^{7} R/hr	27a
2.	PROCESS MONITORS					
	a. Fuel Storage Pool Area Gaseous Activity- Ventilation System Isolation (R-25A&B)	1	(b)	$\leq$ 8.73 x 10^{-3} µCi/cc(c)	10-10^{6} cpm	25
	b. Containment					
	i. Gaseous Activity-					
	a) Purge & Exhaust Isolation (R-24A&B)	1	1,2,3 (d)	$\leq$ 2.27 x 10^{-2} µCi/cc(c)	10-10^{6} cpm	26
			4,5,6 (d)	$\leq$ 2.27 x 10^{-2} µCi/cc(c)		26
			1,2,3,4,5,6 (e)	$\leq$ 4.54 x 10^{-3} µCi/cc(c)		26
			1,2,3,4,5,6 (f)	$\leq$ 2.27 x 10^{-3} µCi/cc(c)		26
	b) RCS Leakage Detection (R-12)	1	1,2,3 & 4	N/A	10-10^{6} cpm	24
	ii. Particulate Activity RCS Leakage Detection (R-11)	1	1,2,3 & 4	N/A	10-10^{6} cpm	24
	c. Control Room Isolation (R35A&B)	1	1,2,3 & 4 and during movement of irradiated fuel or movement of loads over irradiated fuel	$\leq$ 800 cpm	10-10^{6} cpm	27

TABLE 3.3-12
FIRE DETECTION INSTRUMENTATION
(Continued)

Room/ Fire Zone	Description	Elevation	Total Smoke Detectors	Minimum of Operable Smoke Detectors
Auxiliary Building				
2401	Control Room	155'-0"	4	2
2452	Storage Area	155'0"	7	4
2462	Non-Radioactive Vent Equip. Rm.	155'-0"	5	3
2466	West Cable Chase	155'-0"	7	4
2471	Control Rm. (Instrument Racks)	155'-0"	12	6
2500	West Cable Chase	168'-2"	7	4
Containment*				
55	Containment Coolers	155'-9"	12/Fan	6/Fan
55	Containment	155'-0"	14	7
Service Water Intake Structure				
72 A	Pump Room Area	188'-9"	12	6
72 A	Strainer Bay	167'-0"	12	6
72 B	Switchgear Room - Train B	188'-9"	2	1
72 C	Foyer - Train B	188'-9"	1	1
72 D	Foyer - Train A	188'-9"	1	1
72 E	Switchgear Room - Train A	188'-9"	2	1
73	Battery Room - Train B	188'-9"	1	1
74	Battery Room - Train A	188'-9"	1	1
Diesel Generator Building				
56 A	Switchgear Room - Train A	155'-0"	12	6
56 B	Foyer	155'-0"	4	2
56 C	Switchgear Room - Train B	155'-0"	12	6
71	Hallway	155'-0"	9	5
Diesel Generator Building (Heat Detectors)				
57	Diesel Driven Generator 2C	155'-0"	5	3
59	Diesel Driven Generator 2B	155'-0"	5	3
60	Diesel Driven Generator 1C	155'-0"	5	3
61	Diesel Driven Generator 1-2A	155'-0"	5	3
62	Day Tank Room	155'-0"	1	1
64	Day Tank Room	155'-0"	1	1
65	Day Tank Room	155'-0"	1	1
66	Day Tank Room	155'-0"	1	1

*The Fire Detection instruments located within the Containment are not required to be OPERABLE during the performance of Type A Containment Leakage Rate Tests.

9. Preservice Inspection means an inspection of the full length of each tube in each steam generator performed by eddy current techniques prior to service to establish a baseline condition of the tubing. This inspection shall be performed after the field hydrostatic test and prior to initial POWER OPERATION using the equipment and techniques expected to be used during subsequent inservice inspections.

b. The steam generator shall be determined OPERABLE after completing the corresponding actions (plug all tubes exceeding the plugging limit and all tubes containing through-wall cracks) required by Table 4.4-2.

4.4.6.5 Reports

a. Following each inservice inspection of steam generator tubes, the number of tubes plugged in each steam generator shall be reported to the Commission within 15 days.

b. The complete results of the steam generator tube inservice inspection shall be submitted to the Commission in a Special Report pursuant to Specification 6.9.2 within 12 months following the completion of the inspection. This Special Report shall include:

1. Number and extent of tubes inspected.

2. Location and percent of wall-thickness penetration for each indication of an imperfection.

3. Identification of tubes plugged.

c. Results of steam generator tube inspections which fall into Category C-3 and require prompt notification of the Commission shall be reported pursuant to Specification 6.9.1 prior to resumption of plant operation. The written followup of this report shall provide a description of investigations conducted to determine cause of the tube degradation and corrective measures taken to prevent recurrence.

TABLE 4.4-1

MINIMUM NUMBER OF STEAM GENERATORS TO BE INSPECTED DURING INSERVICE INSPECTION

No. of Steam Generators per Unit	Three
First Inservice Inspection	Two
Second & Subsequent Inservice Inspections	One*

*The other steam generator not inspected during the first inservice inspection shall be reinspected. The third and subsequent inspections may be limited to one steam generator on a rotating schedule encompassing 3 N % of the tubes (where N is the number of steam generators in the plant) if the results of the first or previous inspections indicate that all steam generators are performing in a like manner. Note that under some circumstances, the operating conditions in one or more steam generators may be found to be more severe than those in other steam generators. Under such circumstances the same sequence shall be modified to inspect the most severe conditions.

REACTOR COOLANT SYSTEM

OPERATIONAL LEAKAGE

LIMITING CONDITION FOR OPERATION

3.4.7.2 Reactor Coolant System leakage shall be limited to:

a. No PRESSURE BOUNDARY LEAKAGE,

b. 1 GPM UNIDENTIFIED LEAKAGE,

c. 1 GPM total primary-to-secondary leakage through all steam generators and 500 gallons per day through any one steam generator,

d. 10 GPM IDENTIFIED LEAKAGE from the Reactor Coolant System, and

e. 31 GPM CONTROLLED LEAKAGE at a Reactor Coolant System pressure of 2235 ± 20 psig.

f. 1 GPM leakage from any Reactor Coolant System Pressure Isolation Valve specified in Table 3.4-1 at a Reactor Coolant System pressure of 2235 = 20 psig.

APPLICABILITY: MODES 1, 2, 3 and 4

ACTION:

a. With any PRESSURE BOUNDARY LEAKAGE, be in at least HOT STANDBY within 6 hours and in COLD SHUTDOWN within the following 30 hours.

b. With any Reactor Coolant System leakage greater than any one of the above limits, excluding PRESSURE BOUNDARY LEAKAGE, reduce the leakage rate to within limits within 4 hours or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

c. With any Reactor Coolant System Pressure Isolation Valve leakage greater that the above limit, isolate the high pressure portion of the affected system from the low pressure portion within 4 hours by use of at least two closed manual or deactivated automatic valves, or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

SURVEILLANCE REQUIREMENTS

4.4.7.2.1 Reactor Coolant System leakages shall be demonstrated to be within each of the above limits by;

a. Monitoring the containment atmosphere particulate radioactivity monitor at least once per 12 hours.

b. Monitoring the containment air cooler condensate level system or containment atmosphere gaseous radioactivity monitor at least once per 12 hours.

SURVEILLANCE REQUIREMENTS (Continued)

c. Measurement of the CONTROLLED LEAKAGE from the reactor coolant pump seals at least once per 31 days when the Reactor Coolant System pressure is 2235 ± 20 psig with the modulating valve fully open. The provisions of Specification 4.0.4 are not applicable for entry into MODE 3 or 4.

d. Performance of a Reactor Coolant System water inventory balance at least once per 72 hours.

e. Monitoring the reactor head flange leakoff system at least once per 24 hours.

4.4.7.2.2 Each Reactor Coolant System Pressure Isolation Valve specified in Table 3.4-1 shall be demonstrated OPERABLE pursuant to Specification 4.0.5 except that in lieu of any leakage testing required by Specification 4.0.5, each valve should be demonstrated OPERABLE by verifying leakage to be within its limit.

a. Every refueling outage during startup.

b. Prior to returning the valve to service following maintenance, repair or replacement work on the valve affecting the seating capability of the valve.

c. Following valve actuation due to automatic or manual action or flow through the valve for valves identified in Table 3.4-1 by an asterisk.

d. The provisions of Specification 4.0.4 are not applicable for entry into MODE 3 or 4.

SURVEILLANCE REQUIREMENTS

4.4.10.3.1 Each RHR relief valve shall be demonstrated OPERABLE by:

a. Verifying the RHR relief valve isolation valves (8701a, 8701b, 8702a and 8702b) are open at least once per 72 hours when the RHR relief valve is being used for overpressure protection.

b. Testing in pursuant to Specification 4.0.5.

c. Verification of the RHR relief valve setpoint, of at least one RHR relief valve, at least once per 18 months on a rotating basis.

4.4.10.3.2 The RCS vent shall be verified to be open at least once per 12 hours* when the vent is being used for overpressure protection.

*Except when the vent pathway is provided with a valve which is locked, sealed, or otherwise secured in the open position, then verify these valves open at least once per 31 days.

3/4.4.11 STRUCTURAL INTEGRITY

ASME CODE CLASS 1, 2 and 3 COMPONENTS

LIMITING CONDITION FOR OPERATION

3.4.11 The structural integrity of ASME Code Class 1, 2 and 3 components shall be maintained in accordance with Specification 4.4.11.

APPLICABILITY: All MODES

ACTION:

a. With the structural integrity of any ASME Code Class 1 component(s) not conforming to the above requirements, restore the structural integrity of the affected component(s) to within its limit or isolate the affected component(s) prior to increasing the Reactor Coolant System temperature more than 50°F above the minimum temperature required by NDT considerations.

b. With the structural integrity of any ASME Code Class 2 component(s) not conforming to the above requirements, restore the structural integrity of the affected component(s) to within its limit or isolate the affected component(s) prior to increasing the Reactor Coolant System temperature above 200°F.

c. With the structural integrity of any ASME Code Class 3 component(s) not conforming to the above requirements, restore the structural integrity of the affected component(s) to within its limit or isolate the affected component(s) from service.

d. The provisions of Specification 3.0.4 are not applicable.

SURVEILLANCE REQUIREMENTS

4.4.11.1 The structural integrity of ASME Code Class 1, 2 and 3 components shall be demonstrated;

a. Per the requirements of Specification 4.0.5 and

b. By the augmented program specified in Specifications 4.4.11.2 and 4.4.11.3

TABLE 3.7-3
STEAM LINE VALVES PER LOOP

VALVE NUMBER	LIFT SETTING (± 1%)*	ORIFICE SIZE (SQ. IN.)
a. Q2N11VO - 10A, 11A, 12A	1075 psig	16
b. Q2N11VO - 10B, 11B, 12B	1088 psig	16
c. Q2N11VO - 10C, 11C, 12C	1102 psig	16
d. Q2N11VO - 10D, 11D, 12D	1115 psig	16
e. Q2N11VO - 10E, 11E, 12E	1129 psig	16

*The lift setting pressure shall correspond to ambient conditions of the valve at nominal operating temperature and pressure.

AUXILIARY FEEDWATER SYSTEM

LIMITING CONDITION FOR OPERATION

3.7.1.2 At least three independent steam generator auxiliary feedwater pumps and associated manual actuation switches in the control room and flow paths shall be OPERABLE with:

a. Two auxiliary feedwater pumps, each capable of being powered from separate emergency busses, and

b. One auxiliary feedwater pump capable of being powered from an OPERABLE steam supply system.

APPLICABILITY: MODES 1, 2 and 3.

ACTION:

a. With one auxiliary feedwater pump inoperable, restore the required auxiliary feedwater pumps to OPERABLE status within 72 hours or be in at least HOT STANDBY within the next 6 hours and in HOT SHUTDOWN within the following 6 hours.

b. With two auxiliary feedwater pumps inoperable be in at least HOT STANDBY within 6 hours and in HOT SHUTDOWN within the following 6 hours.

c. With three auxiliary feedwater pumps inoperable, immediately initiate corrective action to restore at least one auxiliary feedwater pump to OPERABLE status as soon as possible.

SURVEILLANCE REQUIREMENTS

4.7.1.2.1 Each motor-driven and the turbine-driven auxiliary feedwater pump shall be demonstrated OPERABLE pursuant to Specification 4.0.5. For the turbine-driven pump, the provisions of Specification 4.0.4 are not applicable for entry into MODE 3.

4.7.1.2.2 Each auxiliary feedwater pump shall be demonstrated OPERABLE:

a. At least once per 31 days by:

1. Verifying that each non-automatic valve in the flow path that is not locked, sealed, or otherwise secured in position, is in its correct position.

3/4.7.5 RIVER WATER SYSTEM

LIMITING CONDITION FOR OPERATION

3.7.5 At least two independent river water loops shall be OPERABLE with at least two river water pumps per loop.

APPLICABILITY: MODES 1, 2, 3 and 4.

ACTION:

With only one river water loop OPERABLE, restore at least two loops to OPERABLE status within 72 hours or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

SURVEILLANCE REQUIREMENTS

4.7.5 Each river water loop shall be demonstrated OPERABLE:

a. At least once per 31 days by verifying that each valve (manual, power operated or automatic), in the flow path, servicing safety related equipment that is not locked, sealed, or otherwise secured in position, is in its correct position.

b. At least once per 18 months during shutdown, by:

1. Verifying that each automatic valve servicing safety related equipment actuates to its correct position on a low pond level signal.

2. Verifying that the buried piping is leak tight by a visual inspection of the ground area.

PLANT SYSTEMS

3/4.7.6 ULTIMATE HEAT SINK

RIVER

LIMITING CONDITION FOR OPERATION

3.7.6.1 The ultimate heat sink (river) shall be OPERABLE with a minimum water level at or above 70'-0" Mean Sea Level, USGS datum, and a maximum water level at or below 127' Mean Sea Level.

APPLICABILITY: MODES 1, 2, 3 and 4.

ACTION:

With the requirements of the above specification not satisfied, be in at least HOT STANDBY within 6 hours and in COLD SHUTDOWN within the following 30 hours.

SURVEILLANCE REQUIREMENTS

4.7.6.1 The ultimate heat sink shall be determined OPERABLE at least once per 24 hours by verifying the water level to be within limits.

SURVEILLANCE REQUIREMENTS (Continued)

1. With a half-life greater than 30 days (excluding Hydrogen 3), and

2. In any form other than gas.

b. Stored sources not in use - Each sealed source and fission detector shall be tested prior to use or transfer to another licensee unless tested within the previous six months. Sealed sources and fission detectors transferred without a certificate indicating the last test date shall be tested prior to being placed into use.

c. Startup sources and fission detectors - Each sealed startup source and fission detector shall be tested within 31 days prior to being subjected to core flux or installed in the core and following repair or maintenance to the source.

4.7.10.3 Reports - A report shall be prepared and submitted to the Commission on an annual basis if sealed source or fission detector leakage tests reveal the presence of greater than or equal to 0.005 microcuries of removable contamination.

PLANT SYSTEMS

3/4.7.11 FIRE SUPPRESSION SYSTEMS

FIRE SUPPRESSION WATER SYSTEM

LIMITING CONDITION FOR OPERATION

3.7.11.1 The fire suppression water system shall be OPERABLE with:

a. Two high pressure pumps, each with a capacity of 2500 gpm, with their discharge aligned to the fire suppression header,

b. Separate water supplies, each with a minimum contained volume of 250,000 gallons, and

c. An OPERABLE flow path capable of taking suction from each tank and transferring the water through distribution piping with OPERABLE sectionalizing control or isolation valves to the yard hydrant curb valves, the last valve ahead of the water flow alarm device on each sprinkler or hose standpipe, and the last valve ahead of the deluge valve on each deluge or spray system required to be OPERABLE per Specifications 3.7.11.2, 3.7.11.4 and 3.7.11.5.

APPLICABILITY: At all times.

ACTION:

a. With one of the above required pumps and/or water supplies inoperable, restore the inoperable equipment to OPERABLE status within 7 days or, in lieu of any other report required by Specification 6.9.1, prepare and submit a Special Report to the Commission pursuant to Specification 6.9.2 within the next 30 days outlining the plans and procedures to be used to provide for the loss of redundancy in this system. The provisions of Specifications 3.0.3 and 3.0.4 are not applicable.

b. With the fire suppression water system otherwise inoperable:

1. Establish a backup fire suppression water system within 24 hours, and

2. In lieu of any other report required by Specification 6.9.1, submit a Special Report in accordance with Specification 6.9.2:

a) By telephone within 24 hours,

b) Confirmed by telegraph, mailgram or facsimile transmission no later than the first working day following the event, and

c) In writing within 14 days following the event, outlining the action taken, the cause of the inoperability and the plans and schedule for restoring the system to OPERABLE status.

ACTION: (Continued)

e. With both of the above required diesel generator sets inoperable, demonstrate the OPERABILITY of two offsite A.C. circuits by performing Surveillance Requirement 4.8.1.1.1.a within 1 hour and at least once per 8 hours thereafter; restore at least one of the inoperable diesel generator sets to OPERABLE status within 2 hours or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours. Restore both diesel generator sets to OPERABLE status within 72 hours from time of initial loss or be in least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

SURVEILLANCE REQUIREMENTS

4.8.1.1.1 Each of the above required independent circuits between the offsite transmission network and the onsite Class 1E distribution system shall be:

a. Determined OPERABLE at least once per 7 days by verifying correct breaker alignments, indicated power availability, and

b. Demonstrated OPERABLE at least once per 18 months during shutdown by transferring unit power supply from the normal circuit to the alternate circuit.

4.8.1.1.2 Each diesel generator shall be demonstrated OPERABLE:

a. In accordance with the frequency specified in Table 4.8-1 on a STAGGERED TEST BASIS by:

1. Verifying the fuel level in the day tank,

2. Verifying the fuel level in the fuel storage tanks,

3. Verifying the fuel transfer pump can be started and transfers fuel from the storage system to the day tank,

4. Verifying the diesel starts from ambient condition and accelerates to at least 900 rpm, for the 2850 kw generator and 514 rpm for the 4075 kw generators, in less than or equal to 10 seconds. The generator voltage and frequency shall be $\geq$ 3952 volts and $\geq$ 57 Hz within 10 seconds after the start signal.

5. Verifying the generator is synchronized, loaded to greater than or equal to its continuous rating, and operates for greater than or equal to 60 minutes,

SURVEILLANCE REQUIREMENTS (Continued)

6. Verifying the diesel generator is aligned to provide standby power to the associated emergency busses.

b. At least once per 92 days by verifying that a sample of diesel fuel from the fuel storage tank obtained in accordance with ASTM-D270-65 is within the acceptable limits specified in Table 1 of ASTM-D975-74 when checked for viscosity, water and sediment.

c. At least once per 18 months by:

1. Subjecting the diesel to an inspection in accordance with procedures prepared in conjunction with its manufacturer's recommendations for this class of standby service,

2. Verifying the capability to reject a load of greater than or equal to the largest single load associated with that diesel generator, while maintaining voltage between 3120 and 4910 volts and speed less than or equal to 75% of the difference between nominal speed and the overspeed trip setpoint and verifying recovery to 4160 ± 420 volts and 60 ± 1.2 Hz within 2 seconds.

3. Verifying the generator capability to reject a load equal to its continuous rating without tripping. The generator voltage shall not exceed 120% during and following the load rejection.

4. Simulating a loss of offsite power by itself, and:

a) Verifying de-energization of the emergency busses and load shedding from the emergency busses.

b) Verifying the diesel starts on the auto-start signal, energizes the emergency busses with permanently connected loads within 10 seconds,* energizes the auto-connected shutdown loads through the load sequencer and operates for greater than or equal to 5 minutes while its generator is loaded with the shutdown loads. After energization of all loads, the steady state voltage and frequency of the emergency busses shall be maintained at 4160 $\pm$ 420 volts and 60 $\pm$ 1.2 Hz during this test.

5. Verifying that on an Safety Injection test signal (without loss of offsite power) the diesel generator starts on the auto-start signal and operates on standby for greater than or equal to 5 minutes. The generator voltage and frequency shall be $\geq$ 3952 volts and $\geq$ 57 Hz within 10 seconds after the auto-start signal; the steady state generator voltage and frequency shall be maintained between 4160 ± 420 volts and 60 ± 1.2 Hz during this test.

*Energization of the Unit 2 emergency bus for diesel 2C is achieved within 24 seconds.

SURVEILLANCE REQUIREMENTS (Continued)

6. Simulating a loss of offsite power in conjunction with a Safety Injection test signal, and

   a) Verifying de-energization of the emergency busses and load shedding from the emergency busses.

   b) Verifying the diesel starts from ambient condition on the auto-start signal, energizes the emergency busses with permanently connected loads within 10 seconds,** energizes the auto-connected emergency (accident) loads through the load sequencer and operates for greater than or equal to 5 minutes while its generator is loaded with the emergency loads. After energization, the steady state voltage and frequency of the emergency busses shall be maintained at 4160 ± 420 volts and 60 ± 1.2 Hz during this test.

   c) Verifying that all automatic diesel generator trips, except engine overspeed and generator differential and low lube oil pressure, are automatically bypassed upon loss of voltage on the emergency bus and/or a safety injection test signal.

7. Verifying the diesel generator operates for at least 24 hours. During the first 2 hours of this test, the diesel generator shall be loaded to greater than or equal to 4474 kw for the 4075 kw diesels and 3250 for the 2850 diesels and during the remaining 22 hours of this test, the diesel generator shall be loaded to greater than or equal to 4075 kw for the 4075 kw diesels and 2850 kw for the 2850 kw diesels. Immediately after completing this 24 hour test, perform Specification 4.8.1.1.2.c.4. The generator voltage and frequency shall be ≧ 3952 volts and ≧ 57 Hz within 10 seconds after the start signal; the generator voltage and frequency shall be maintained between 3120 and 4910 volts and 57 and 61.2 Hz during this test.*

8. Verifying that the auto-connected loads to each diesel generator do not exceed the 2000 hour rating of 4353 kw for the 4075 kw generator and 3100 kw for the 2850 kw generator.

*This surveillance is not required for MODE 3 or 4. This is a one time change to plant operations prior to initial criticality.

**Energization of the Unit 2 emergency bus for diesel 2C is achieved within 24 seconds.

SURVEILLANCE REQUIREMENTS (Continued)

10. Verifying that with the diesel generator operating in a test mode (connected to its bus), a simulated safety injection signal overrides the test mode by returning the diesel generator to standby operation.

11. Verifying that the automatic load sequence timer is OPERABLE with each load sequence time within ± 10% of its required value or 0.5 seconds whichever is greater.

12. Verifying that the following diesel generator lockout features prevent diesel generator starting only when required:

    a) Oil Temperature High (OTH)

    b) Coolant Temperature High (CTH)

    c) Coolant Pressure Low (CPL)

    d) Crankcase Pressure High (CCPH)

d. At least once per 10 years or after any modifications which could affect diesel generator interdependence by starting the diesel generators simultaneously, and verifying that the diesel generators accelerate to at least 900 rpm, for the 2850 kw generator and 514 rpm for the 4075 kw generator, in less than or equal to 10 seconds.

4.8.1.1.3 Reports - All diesel generator failures, valid or non-valid, shall be reported to the Commission pursuant to Specification 6.9.1. Reports of diesel generator failures shall include the information recommended in Regulatory Position C.3.b of Regulatory Guide 1.108, Revision 1, August 1977. If the number of failures in the last 100 valid tests (on a per diesel type basis) is greater than or equal to 7, the report shall be supplemented to include the additional information recommended in Regulatory Position C.3.b of Regulatory Guide 1.108, Revision 1, August 1977.

SURVEILLANCE REQUIREMENTS (Continued)

(c) For each circuit breaker found inoperable during these functional tests, an additional representative sample of at least one of the circuit breakers of the inoperable type shall also be functionally tested until no more failures are found or all circuit breakers of that type have been functionally tested.

2. By selecting and functionally testing a representative sample of at least 10% of each type of lower voltage circuit breakers. Circuit breakers selected for functional testing shall be selected on a rotating basis. The functional test shall consist of injecting a current input at the specified setpoint to each selected circuit breaker and verifying that each circuit breaker functions as designed. For each circuit breaker found inoperable during these functional tests, an additional representative sample of at least 10% of all the circuit breakers of the inoperable type shall also be functionally tested until no more failures are found or all circuit breakers of that type have been functionally tested.

3. By selecting and functionally testing a representative sample of each type of fuse on a rotating basis. Each representative sample of fuses shall include at least 10% of all fuses of that type. The functional test shall consist of a non-destructive resistance measurement test which demonstrates that the fuse meets its manufacturer's design criteria. For each fuse found inoperable during these functional tests, an additional representative sample of at least 10% of all fuses of that type shall be functionally tested until no more failures are found or all fuses of that type have been functionally tested.

b. At least once per 60 months by subjecting each circuit breaker to an inspection and preventive maintenance in accordance with procedures prepared in conjunction with its manufacturer's recommendations.

4.8.3.2 Power sources feeding circuits indicated by an asterisk (*) in Table 3.8-1 shall be verified to be interrupted by an open breaker or removed fuse at least once every 31 days.

TABLE 3.8-1

CONTAINMENT PENETRATION CONDUCTOR
OVERCURRENT PROTECTIVE DEVICES

Device Number and Location	Trip Setpoint (Ampere)	Response Time (Seconds)	System Powered
1. 4160VAC Switchgears			
DA04	9600	.01-.08	Reactor Coolant Pump 2A
DB03	9600	.01-.08	Reactor Coolant Pump 2B
DC03	9600	.01-.08	Reactor Coolant Pump 2C
2. 600VAC Load Centers			
EA04*,#	1000 ± 10%	.07-.18	Reactor Polar Crane
EA10	1800 ± 10%	.01-.05	Containment Cooler 2A (Normal)
EB05	1800 ± 10%	.01-.05	Containment Cooler 2B (Normal)
EB06	1800 ± 10%	.01-.05	Containment Cooler 2C (Normal)
EC12	1800 ± 10%	.01-.05	Containment Cooler 2D (Normal)
ED11	1200 ± 10%	.01-.05	CRDM Cooler Fan 2A
ED15	1500 ± 10%	.01-.05	Containment Cooler 2A (Emergency)
ED16	1500 ± 10%	.01-.05	Containment Cooler 2B (Emergency)
EE08	1500 + 10%	.01-.05	Containment Cooler 2C (Emergency)
EE13	1200 ± 10%	.01-.05	CRDM Cooler Fan 2B
EE16	1500 ± 10%	.01-.05	Containment Cooler 2D (Emergency)
3. 600VAC Motor Control Centers			
FA-F7	210 (INST)	.01-.016	Reactor Cavity Clg Fan Mtr. 2A
FA-I6	10 (INST)	.01-.016	Reactor Cavity Cool Fan Mov
FA-I5	105 (INST)	.01-.016	CTMT Post LOCA Air Mixing Fan 2D
FA-J5	105 (INST)	.01-.016	CTMT Post LOCA Air Mixing Fan 2C
FB-I3	105 (INST)	.01-.016	CTMT Post LOCA Air Mixing Fan 2B
FB-I4	105 (INST)	.01-.016	CTMT Post LOCA Air Mixing Fan 2A
FD-H7R*,#	400-700	.01-.016	CTMT G1B Crane (N2T31K005-N)
FB-J7	210 (INST)	.01-.016	Reactor Cavity Clg Fan Mtr. 2A
FB-A4R*,#	400-700	.01-.016	CTMT Elevator No. 3 Controller
FC-P3	105 (INST)	.01-.016	CTMT Dome Recirc. Fan 2A
FC-P4	105 (INST)	.01-.016	CTMT Dome Recirc. Fan 2B
FC-I2*,#	400-700	.01-.016	RCP Motor Space Heaters
FC-I4	125 (INST)	.01-.016	RCP BRG Oil Lift Pump
FC-N3	125 (INST)	.01-.016	RCP BRG Oil Lift Pump
FC-J3*	210 (INST)	.01-.016	Refueling Wtr Surface Supply
FC-M5*	630 (INST)	.01-.016	Refueling Wtr Surface Exhaust
FC-J5	330 (INST)	.01-.016	CTMT Pre-Access Fan Motors
FC-N4*,#	400-700	.01-.016	RCP Mtr Space Heaters 2C

TABLE 3.8-1 (Continued)

CONTAINMENT PENETRATION CONDUCTOR
OVERCURRENT PROTECTIVE DEVICES

Device Number and Location	Trip Setpoint (Ampere)	Response Time (Seconds)	System Powered
FC-J6L*,#	600-1000	.01-.016	Receptacles Term. Box 2A
FC-J6R*	400-700	.01-.016	Reactor Cavity Filter Pmp Recep.
FC-S3L*	400-700	.01-.016	Up Ending Frame Winch Mtr
FC-M4L*,#	400-700	.01-.016	CTMT GIB Crane (N2T31K004-N)
FC-N2L*,#	400-700	.01-.016	Reactor Cavity Manipulator Crane
FC-S3R*,#	400-700	.01-.016	CTMT GIB Crane (N2T31K006-N)
FC-C5R*,#	400-700	.01-.016	SW Port Drain Pump Recept.
FD-D6	125 (INST)	.01-.016	RCP BRG Oil Lift Pump
FD-E3	105 (INST)	.01-.016	CTMT Dome Recirculation Fan 2D
FD-B4	330 (INST)	.01-.016	Reactor Cool Drain Tank Pump 2B
FD-C2	125 (INST)	.01-.016	CTMT Sump Pump Mtr 2A
FD-D2	125 (INST)	.01-.016	CTMT Sump Pump Mtr 2B
FD-C4	330 (INST)	.01-.016	CTMT Pre-Access Fan Mtrs
FD-G3*,#	400-700	.01-.016	RCP Mtr Space Heaters 2B
FD-A3L*,#	600-1000	.01-.016	Receptacles Term Box 2C
FE-A6	330 (INST)	.01-.016	Reactor Cool Drain Tank Pump 2A
FE-A4L*,#	600-1000	.01-.016	Receptacles Term Box 2B
FE-C3L	600-1000	.01-.016	Incore Det. Drive & Cont. Pnl.
FE-H5	105 (INST)	.01-.016	CTMT Dome Recirc. Fan 2C
FU-G2	29 (INST)	.01-.016	RHR Pumps Inlet Mov
FU-H2	12 (INST)	.01-.016	CTMT Air Cooling Fan Mov
FU-H3	12 (INST)	.01-.016	CTMT Air Cooling Fan Mov
FU-H4	10 (INST)	.01-.016	RCP Motor Cooler Disch Mov
FU-J4	10 (INST)	.01-.016	CTMT To Atmos Diff Press Mov
FU-K4	22 (INST)	.01-.016	Pressurizer to Relief Tank MOV
FU-K6	10 (INST)	.01-.016	CTMT Cooler Disch MOV
FU-L4	12 (INST)	.01-.016	Post ACDT Air Sampler From CTMT Mov
FU-L5	12 (INST)	.01-.016	Post ACDT Air Sampler From CTMT Mov
FU-T4	22 (INST)	.01-.016	RCP Seal Water Return Isol.
FU-M4	12 (INST)	.01-.016	Post ACDT Air Sampler Return Mov
FU-R5	16 (INST)	.01-.016	CTMT Air Sampler Mov.
FU-V4	12 (INST)	.01-.016	CRDM Cool Fan Dampers Mov.
FU-W2	16 (INST)	.01-.016	CTMT Air Cooler Disch Mov.
FU-W4	29 (INST)	.01-.016	Reactor Cavity H_2 Dilution Fan VLV 2A

TABLE 3.8-1 (Continued)

CONTAINMENT PENETRATION CONDUCTOR
OVERCURRENT PROTECTIVE DEVICES

Device Number and Location	Trip Setpoint (Ampere)	Response Time (Seconds)	System Powered
FU-T5	45 (INST)	.01-.016	RHR System Inlet Isol. Vlv.
FU-Z2	400 (INST)	.01-.016	Accumulator 2A Disch. Vlv.
FU-Z3	400 (INST)	.01-.016	Accumulator 2C Disch. Vlv.
FV-M3	12 (INST)	.01-.016	Post ACDT Air Sampler Return Mov.
FV-H4	12 (INST)	.01-.016	Post ACDT Air Sampler From CTMT Mov
FV-H5	12 (INST)	.01-.016	Post ACDT Air Sampler From CTMT Mov
FV-Y4	12 (INST)	.01-.016	Post LOCA CTMT Vent Mov
FV-Y5	29 (INST)	.01-.016	Instr. Air Line Disch. Mov.
FV-I2	12 (INST)	.01-.016	CRDM Cool Fan Damper
FV-J4	16 (INST)	.01-.016	CTMT Air Cooler Disch. Mov.
FV-J5	16 (INST)	.01-.016	CTMT Air Cooler Disch. Mov.
FV-N2	29 (INST)	.01-.016	Reac. Cavity H_2 Dilution Fan Vlv. 2B
FV-S2	400 (INST)	.01-.016	Accumulator 2B Disch. Vlv.
FV-V3	45 (INST)	.01-.016	RHR System Inlet Isol. Vlv.
FV-W4	22 (INST)	.01-.016	Pressurizer to Relief Isol. Mov.
FV-V2	45 (INST)	.01-.016	RHR System Outlet Isol. Vlv.
FV-C3	22 (INST)	.01-.016	RCP CCW Return from Oil Cool
FV-F2	12 (INST)	.01-.016	CTMT Air Cooling Fan Mov.
FV-F3	12 (INST)	.01-.016	CTMT Air Cooling Fan MOV.
4. 208 VAC Motor Control Centers			
HA-03	105 (INST)	.01-.016	Reac. Cavity H_2 Dilution Fan 2A
HB-N7	150 (INST)	.01-.016	Reac. Cavity H_2 Dilution Fan 2B
5. 600VAC Pressurizer Distr. Pnl 2A			
BKR #1	750-1600	.01-.016	Pressurizer Htr Group 2A
BKR #2	750-1600	.01-.016	Terminal Box N2TB010
BKR #3	750-1600	.01-.016	
BKR #4	750-1600	.01-.016	
BKR #5	750-1600	.01-.016	

TABLE 3.8-1 (Continued)

CONTAINMENT PENETRATION CONDUCTOR
OVERCURRENT PROTECTIVE DEVICES

	Device Number and Location	Trip Setpoint (Ampere)	Response Time (Seconds)	System Powered
6.	600VAC Pressurizer Distr. Pnl 2B			
	BKR #1	750-1600	.01.-016	Pressurizer Htr Group 2B
	BKR #2	750-1600	.01.-016	Terminal Box N2TB011
	BKR #3	750-1600	.01.-016	
	BKR #4	750-1600	.01.-016	
	BKR #5	750-1600	.01.-016	
7.	600VAC Pressurizer Htr. Distr. Pnl 2C Circuit Bkrs.			
	BKR #1	750-1600	.01-.016	Pressurizer Htr Group 2C
	BKR #2	750-1600	.01-.016	Terminal Box N2TB008
	BKR #3	750-1600	.01-.016	
	BKR #4	750-1600	.01-.016	
	BKR #4	750-1600	.01-.016	
	BKR #5	750-1600	.01-.016	
	BKR #6	750-1600	.01-.016	
	BKR #7	750-1600	.01-.016	
8.	600VAC Pressurizer Htr. Distr. Pnl 2D Circuit Bkrs.			
	BKR #1	750-1600	.01-.016	Pressurizer Htr Group 2D
	BKR #2	750-1600	.01-.016	Terminal Box N2TB007
	BKR #3	750-1600	.01-.016	
	BKR #4	750-1600	.01-.016	
9.	600VAC Pressurizer HTR. Distr. Pnl 2E Circuit Bkrs.			
	BKR #1	750-1600	.01-.016	Pressurizer Htr Group 2E
	BKR #2	750-1600	.01-.016	Terminal Box N2TB009
	BKR #3	750-1600	.01-.016	
	BKR #4	750-1600	.01-.016	
	BKR #5	750-1600	.01-.016	

TABLE 3.8-1 (Continued)

CONTAINMENT PENETRATION CONDUCTOR
OVERCURRENT PROTECTIVE DEVICES

	Device Number and Location	Trip Setpoint (Ampere)	Response Time (Seconds)	System Powered
10.	125VDC Sol. Vlvs. Power Fuses			
	FB1 (TC-05A)	3	N.A.	PRZR PWR Relief VLV (445A) 2A
	FB1 (TC-25A)	3	N.A.	PRZR PWR Relief VLV (444B) 2B
	FB5 (TC-06A)	3	N.A.	Letdown Line Isol. Vlv. (459)
	FB6 (TC-06A)	3	N.A.	Letdown Line Isol. Vlv. (460)
	FB5 (TC-05A)	3	N.A.	Reac. Cool Drn Tank Pump Disc Vlv (1003A)
	FB6 (TC-05A)	3	N.A.	Reac. Cool Drn Tank Vent Isol Vlv (7126)
	FB3 (TC-06A)	3	N.A.	RMW to RCP Standpipe Fill (8168C)
	FB4 (TC-25B)	3	N.A.	PRZR Rel. Tnk to RMW Isol. Vlv. (8030)
	FB5 (TC-25B)	3	N.A.	PRZR Rel. Tnk Drn to WPS Drn Tnk (8031)
	FB6 (TC-27B)	3	N.A.	Excess Letdown Isol. Vlv. (8153)
	FB5 (TC-27B)	3	N.A.	RCS PRZR Aux. Spray Vlv. (8145)
	FB7 (TC-27B)	3	N.A.	Excess Letdown Isol. Vlv. (8154)
	FB2 (TC-27B)	3	N.A.	RMW to RCP Standpipe Fill (8168A)
	FB3 (TC-27B)	3	N.A.	RMW to RCP Standpipe Fill (8168B)
	FB2 (TC-08A)	3	N.A.	Accum. N_2 Supply & Vent Isol (8875A)
	FB6 (TC-08A)	3	N.A.	Accum. N_2 Supply & Vent Isol (8875C)
	FB4 (TC-08A)	3	N.A.	Accum. Line Test Isol. (8877A)
	FB8 (TC-08A)	3	N.A.	Accum. Line Test Isol. (8877C)
	FB3 (TC-08A)	3	N.A.	Accum. Fill Line Isol. (8878A)
	FB7 (TC-08A)	3	N.A.	Accum Fill Line Isol. (8878C)
	FB5 (TC-08A)	3	N.A.	Accum. Inject Test Line Isol. (8879A)
	FB9 (TC-08A)	3	N.A.	Accum. Inject. Test Line Isol. (8879C)
	FB2 (TC-29B)	3	N.A.	Accum. N_2 Supply & Vent Isol (8875B)

TABLE 3.8-1 (Continued)

CONTAINMENT PENETRATION CONDUCTOR
OVERCURRENT PROTECTIVE DEVICES

Device Number and Location	Trip Setpoint (Ampere)	Response Time (Seconds)	System Powered
FB4 (TC-29B)	3	N.A.	Accum. Test Line Isol. (8877B)
FB3 (TC-29B)	3	N.A.	Accum. Fill Line Isol. (8878B)
FB5 (TC-29B)	3	N.A.	Accum. Inject. Test Line Isol (8879B)
FB1 (TC-06A)	3	N.A.	RCP Seal Leak Off Isol (8141A)
FB1 (TC-29B)	3	N.A.	Damper Sol. Vlvs. CTMT Purge Isol (3196)
FB9 (TC-06A)	3	N.A.	Letdown Orifice Isol. Vlv. (8149B)
FB2 (TC-06A)	3	N.A.	RCP Seal Leak Off Isol. (8141C)
FB1 (TC-27B)	3	N.A.	RCP Seal Leak Off Isol. (8141B)
FB1 (TC-06A)	3	N.A.	RCS Alternate Charging Line (8147)
FB1 (TC-27B)	3	N.A.	RCS Normal Charging Line (8146)
FB7 (TC-25B)	3	N.A.	Reac. Vessel Leak Off Isol. (8032)
FB2 (TC-25B)	3	N.A.	PRZR Rel. Tnk to RMW Supply Isol (8047)
FB8 (TC-27B)	3	N.A.	RCP Seals Wtr By-pass Isol. Vlv. (8142)
FB5 (TC-27B)	3	N.A.	Excess L'Down to VLT or RC DrnTk (8143)
FB10 (TC-06a)	3	N.A.	Letdown Orifice Isol. Vlv. (8149A)
FB8 (TC-06A)	3	N.A.	Letdown Orifice Isol. Vlv. (8149C)
FB1 (TC-08A)	3	N.A.	Accum. Test Line Isol. Vlv. (8871)
FB4 (TC-31B)	3	N.A.	RCP Comp. Cool (3184)
FB1 (TC-09A)	3	N.A.	Excess Letdown Hx Cool Disch (3443)
FAN (NBL2702B-B)	3	N.A.	SG2A Blowdown (3179C)
FAM (NBL2702B-B)	3	N.A.	SG2B Blowdown (3180C)
FAL (NBL2702B-B)	3	N.A.	SG2C Blowdown (3181C)
FAC (NBL2702B-B)	3	N.A.	SG2A Blowdown (3951B)
FAB (NBL2702B-B)	3	N.A.	SG2B Blowdown (3952B)
FAA (NBL2702B-B)	3	N.A.	SG2C Blowdown (3953B)
FAD (NBL2702B-B)	3	N.A.	CVCS Letdown Line Vlv. (3950B)
FD (NBL2702A-A)	3	N.A.	CVCS Letdown Line Vlv. (3950A)

TABLE 3.8-1 (Continued)

CONTAINMENT PENETRATION CONDUCTOR
OVERCURRENT PROTECTIVE DEVICES

Device Number and Location	Trip Setpoint (Ampere)	Response Time (Seconds)	System Powered
FC (NBL2702A-A)	3	N.A.	SG2A Blowdown (3951A)
FB (NBL2702A-A)	3	N.A.	SG2B Blowdown (3952A)
FA (NBL2702A-A)	3	N.A.	SG2C Blowdown (3953A)
FK (NBL2702A-A)	3	N.A.	SG2A Blowdown (7697A)
FJ (NBL2702A-A)	3	N.A.	SG2B Blowdown (7698A)
FH (NBL2707A-A)	3	N.A.	SG2C Blowdown (7699A)
FAK (NBL2702B-B)	3	N.A.	SG2A Blowdown (7697B)
FAJ (NBL2702B-B)	3	N.A.	SG2B Blowdown (7698B)
FAH (NBL2702B-B)	3	N.A.	SG2C Blowdown (7699B)
FAS (NBL2702B-B)	3	N.A.	Press. Stm Space Sampler (3880)
FAR (NBL2702B-B)	3	N.A.	Press. Liq Space Sampler (3881)
F/3766(NFSS2607A-A)	3	N.A.	Accumulator Sampler (3766)
F/3179A(NFSS2607A-A)	3	N.A.	SG2A Blowdown Sampler (3179A)
F/3179B(NFSS2607A-A)	3	N.A.	SG2A Blowdown Sampler (3179B)
F/3180A(NFSS2607A-A)	3	N.A.	SG2B Blowdown Sampler (3180A)
F/3180B(NFSS2607A-A)	3	N.A.	SG2B Blowdown Sampler (3180B)
F/3181A(NFSS2607A-A)	3	N.A.	SG2C Blowdown Sampler (3181A)
F/3181B(NFSS2607A-A)	3	N.A.	SG2C Blowdown Sampler (3181B)
F/3162(NFSS2607A-A)	3	N.A.	Accum. Tank 2A Sampler (3162)
F/3163(NFSS2607A-A)	3	N.A.	Accum. Tank 2B Sampler (3163)
F/3164(NFSS2607A-A)	3	N.A.	Accum. Tank 2C Sampler (3164)
F/3101(NFSS2607B-B)	3	N.A.	Reac.Coolant Hot Leg (3101)
F/3102(NFSS2607B-B)	3	N.A.	Reac.Coolant Hot Leg (3102)
FA(NGB25040)	3	N.A.	CTMT Sump Disch. (3376)
FA(NGB25Q4)	3	N.A.	CTMT Cooler Drains (3395A)
FD(NGB25Q4)	3	N.A.	CTMT Cooler Drains (3395B)
FC(NGB25Q4)	3	N.A.	CTMT Cooler Drains (3395C)
FB(NGB25Q4)	3	N.A.	CTMT Cooler Drains (3395D)
F/3103(NFSS2607B-B)	3	N.A.	PRZR Liquid Sampler (3103)
F/3104(NFSS2607B-B)	3	N.A.	PRZR Liquid Sampler (3104)
F/3765(NFSS2607B-B)	3	N.A.	Reac. Hot Leg Sampler (3765)
11. 120VAC Sol. Vlvs. Power Fuses			
FM(NGB2504M)	3	N.A.	Reac. Cavity Cooling (3999A)
FF(NGB25040)	3	N.A.	Reac. Cavity Cooling (3999B)

TABLE 3.8-1 (Continued)

CONTAINMENT PENETRATION CONDUCTOR
OVERCURRENT PROTECTIVE DEVICES

Device Number and Location	Trip Setpoint (Ampere)	Response Time (Seconds)	System Powered
12. Variable D.C. Voltage CRDM Fuses (125 V Max)			
FU13/A22	10	N.A.	Mechanism #1 Sta.Gripper Group A
FU14/A22	10	N.A.	Mechanism #2 Sta.Gripper Group A
FU15/A22	10	N.A.	Mechanism #3 Sta.Gripper Group A
FU16/A22	10	N.A.	Mechanism #4 Sta.Gripper Group A
FU25/A23	10	N.A.	Mechanism #1 Sta.Gripper Group B
FU26/A23	10	N.A.	Mechanism #2 Sta.Gripper Group B
FU27/A23	10	N.A.	Mechanism #3 Sta.Gripper Group B
FU28/A23	10	N.A.	Mechanism #4 Sta.Gripper Group B
FU41/A24	10	N.A.	Mechanism #1 Sta.Gripper Group C
FU42/A24	10	N.A.	Mechanism #2 Sta.Gripper Group C
FU43/A24	10	N.A.	Mechanism #3 Sta.Gripper Group C
FU44/A24	10	N.A.	Mechanism #4 Sta.Gripper Group C
FU1/A51	50	N.A.	Mechanism #1 Lift Group A
FU2/A51	50	N.A.	Mechanism #2 Lift Group A
FU1/A52	50	N.A.	Mechanism #3 Lift Group A
FU2/A52	50	N.A.	Mechanism #4 Lift Group A
FU1/A53	50	N.A.	Mechanism #1 Lift Group A
FU2/A53	50	N.A.	Mechanism #2 Lift Group B
FU1/A54	50	N.A.	Mechanism #3 Lift Group B
FU2/A54	50	N.A.	Mechanism #4 Lift Group B
FU1/A55	50	N.A.	Mechanism #1 Lift Group C
FU2/A55	50	N.A.	Mechanism #2 Lift Group C
FU1/A56	50	N.A.	Mechanism #3 Lift Group C
FU2/A56	50	N.A.	Mechanism #4 Lift Group C
FU21/A25	10	N.A.	Mechanism #1 Moving Gripper Group A
FU22/A25	10	N.A.	Mechanism #2 Moving Gripper Group A
FU23/A25	10	N.A.	Mechanism #3 Moving Gripper Group A

TABLE 3.8-1 (Continued)

CONTAINMENT PENETRATION CONDUCTOR
OVERCURRENT PROTECTIVE DEVICES

Device Number and Location	Trip Setpoint (Ampere)	Response Time (Seconds)	System Powered
FU24/A25	10	N.A.	Mechanism #4 Moving Gripper Group A
FU33/A26	10	N.A.	Mechanism #1 Moving Gripper Group B
FU34/A26	10	N.A.	Mechanism #2 Moving Gripper Group B
FU35/A26	10	N.A.	Mechanism #3 Moving Gripper Group B
FU36/A26	10	N.A.	Mechanism #4 Moving Gripper Group B
FU49/A27	10	N.A.	Mechanism #1 Moving Gripper Group C
FU50/A27	10	N.A.	Mechanism #2 Moving Gripper Group C
FU51/A27	10	N.A.	Mechanism #3 Moving Gripper Group C
FU52/A27	10	N.A.	Mechanism #4 Moving Gripper Group C
13. 480/277 VAC Lighting Cables			
No Prot. Device	N.A.	N.A.	Lighting Panel 2R(N2T51L001A-N
No Prot. Device	N.A.	N.A.	Lighting Panel 2O(N2T51L001D-N
No Prot. Device	N.A.	N.A.	Lighting Panel 2Q(N2T51L001B-N
No Prot. Device	N.A.	N.A.	Lighting Panel 2P(N2T51L001C-N
No Prot. Device	N.A.	N.A.	Receptacle Panel 2F(N2T51L002A-N)
14. 480 VAC, 30, H_2 Recombiner Power Supply			
No Prot. Device	N.A.	N.A.	H_2 Recombiner HTRS (Q2E17G001A-A)
No Prot. Device	N.A.	N.A.	H_2 Recombiner HTRS (Q2E17G001B-B)

*Circuits are to be deenergized except where allowed by note #.

#May be energized under administrative controls when required for operational or maintenance activities.

TABLE 3.8-2 (Continued)

MOTOR OPERATED VALVES THERMAL OVERLOAD PROTECTION DEVICES*

VALVE NUMBER	FUNCTION	BYPASS DEVICE
MOV-8106	Charging Pump Mini Flow Isolation	No
MOV-8826A	Containment Spray Suction from Containment Sump	No
MOV-8826B	Containment Spray Suction from Containment Sump	No
MOV-8827A	Containment Spray Suction from Containment Sump	No
MOV-8827B	Containment Spray Suction from Containment Sump	No
MOV-8817A	Containment Spray Suction from RWST	No
MOV-8817B	Containment Spray Suction from RWST	No
MOV-8836A	Eductor Suction from Spray Additive Tank	No
MOV-8836B	Eductor Suction from Spray Additive Tank	No
MOV-8820A	Discharge to Spray Ring	No
MOV-8820B	Discharge to Spray Ring	No
MOV-8803A	BIT Inlet	No
MOV-8803B	BIT Inlet	No
MOV-8801A	BIT Outlet	No
MOV-8801B	BIT Outlet	No
MOV-8886	Charging Pump Discharge to Hot Leg	No
MOV-8884	Charging Pump Discharge to Hot Leg	No
MOV-8885	Charging Pump Discharge to Cold Leg	No
MOV-8808A	SIS Accumulator Outlet	No
MOV-8808B	SIS Accumulator Outlet	No
MOV-8808C	SIS Accumulator Outlet	No
MOV-8811A	RHR Suction from Containment Sump	No
MOV-8811B	RHR Suction from Containment Sump	No
MOV-8812A	RHR Suction from Containment Sump	No
MOV-8812B	RHR Suction from Containment Sump	No
MOV-8809A	RHR Suction from RWST	No
MOV-8809B	RHR Suction from RWST	No
MOV-8887A	RHR Discharge Crossconnect	No
MOV-8887B	RHR Discharge Crossconnect	No
FCV-602B	RHR Pump Mini Flow	No
FCV-602A	RHR Pump Mini Flow	No
MOV-8889	RHR Discharge to Hot Leg	No
MOV-8888A	RHR Discharge to Cold Leg	No
MOV-8888B	RHR Discharge to Cold Leg	No
MOV-8706A	RHR Discharge to Charging Pump Suction	No
MOV-8706B	RHR Discharge to Charging Pump Suction	No
MOV-8112	Seal Water Return Containment Isolation	No
MOV-8100	Seal Water Return Containment Isolation	No

TABLE 3.8-2 (Continued)

MOTOR OPERATED VALVES THERMAL OVERLOAD PROTECTION DEVICES*

VALVE NUMBER	FUNCTION	BYPASS DEVICE
QSP25V513	RW to Pond Isolation	No
QSP25V514	RW to Pond Isolation	No
QSP25V517	RW to Wet Pit Isolation	No
QSP25V518	RW to Wet Pit Isolation	No
MOV-3536	CTMT Air Purge Isolation	No
MOV-3530	Post LOCA Vent Isolation	No
MOV-3740	Post LOCA Vent Isolation	No
MOV-3528A	CTMT Air Sample Isolation	No
MOV-3528B	CTMT Air Sample Isolation	No
MOV-3528C	CTMT Air Sample Isolation	No
MOV-3528D	CTMT Air Sample Isolation	No
MOV-3739A	CTMT Air Sample Isolation	No
MOV-3739B	CTMT Air Sample Isolation	No
MOV-3745A	CTMT Air Sample Isolation	No
MOV-3745B	CTMT Air Sample Isolation	No
MOV-3835A	CTMT Air Sample Isolation	No
MOV-3835B	CTMT Air Sample Isolation	No
MOV-3362A	Pen Room Vent Damper	No
MOV-3362B	Pen Room Vent Damper	No
MOV-3361A	Pen Room Vent Damper	No
MOV-3361B	Pen Room Vent Damper	No
MOV-3406	Turbine Trip and Throttle	No
MOV-3232A	Feedwater Isolation	No
MOV-3232B	Feedwater Isolation	No
MOV-3232C	Feedwater Isolation	No
Q2P16V514	Turbine Building Isolation	No
Q2P16V515	Turbine Building Isolation	No
Q2P16V516	Turbine Building Isolation	No
Q2P16V517	Turbine Building Isolation	No
Q2P16V538	Pond Recirculation	No
Q2P16V539	Pond Recirculation	No
LCV-115C	Charging Pump Suction from VCT	No
LCV-115E	Charging Pump Suction from VCT	No
LCV-115B	Charging Pump Suction from RWST	No
LCV-115D	Charging Pump Suction from RWST	No
MOV-8131A	Charging Pump Suction Crossconnect	No
MOV-8131B	Charging Pump Suction Crossconnect	No
MOV-8130A	Charging Pump Suction Crossconnect	No
MOV-8130B	Charging Pump Suction Crossconnect	No
MOV-8132A	Charging Pump Discharge Crossconnect	No
MOV-8132B	Charging Pump Discharge Crossconnect	No
MOV-8133A	Charging Pump Discharge Crossconnect	No
MOV-8133B	Charging Pump Discharge Crossconnect	No
MOV-8107	Charging Line Isolation	No
MOV-8108	Charging Line Isolation	No
MOV-8109A	Charging Pump Mini Flow Isolation	No
MOV-8109B	Charging Pump Mini Flow Isolation	No
MOV-8109C	Charging Pump Mini Flow Isolation	No

*Licensee may delete valves from this table provided the thermal overload protection devices are permanently bypassed.

REFUELING OPERATIONS

3/4.9.13 STORAGE POOL VENTILATION (FUEL MOVEMENT)

LIMITING CONDITION FOR OPERATION

3.9.13 Two independent penetration room filtration systems (Specification 3.7.8) shall be OPERABLE and aligned to the spent fuel pool room:

APPLICABILITY: During crane operation with loads, over the fuel in the spent fuel pit and during fuel movement within the spent fuel pit.

ACTION

a. With one penetration room filtration system inoperable return both systems to OPERABLE status within 48 hours or suspend all movement of fuel and crane operation with loads over the spent fuel in the storage pool room.

b. The provisions of Specification 3.0.3 and 3.0.4 are not applicable.

SURVEILLANCE REQUIREMENTS

4.9.13.1 Two penetration room filtration systems shall be verified to be aligned to the spent fuel pool room within 12 hours prior to fuel handling or crane operations in the storage pool room and at least once per 24 hours thereafter until fuel movement operations in the storage pool room are suspended.

4.9.13.2 The penetration room filtration system shall be demonstrated OPERABLE per the requirements of Specification 4.7.8.

4.9.13.3 At least once per 18 months verify that the normal spent fuel pool system ventilation system will isolate upon receipt of either;

a. The spent fuel pool ventilation low differential pressure test signal, or

b. A spent fuel pool high radiation test signal.

REFUELING OPERATIONS

3/4.9.14 CONTAINMENT PURGE EXHAUST FILTER

LIMITING CONDITION FOR OPERATION

3.9.14 The containment purge exhaust filter shall be OPERABLE and valve N1P13V293 closed.

APPLICABILITY: During CORE ALTERATIONS and Fuel Movement inside containment with any containment purge isolation valve open.*

ACTION: With the containment purge exhaust filter inoperable either:

1. Immediately close the 48 inch containment purge isolation valves (CBV-HV-3196, 3197, 3198A and 3198D) and the 18 inch containment mini-purge isolation valves (CBV-HV-2866A, 2866B, 2867A and 2867B), or

2. Cease all CORE ALTERATIONS and fuel movement.

SURVEILLANCE REQUIREMENTS

4.9.14 The above required containment purge exhaust filter shall be demonstrated OPERABLE:

a. At least once per 18 months or (1) after any structural maintenance on the HEPA filter or charcoal adsorber housings, or (2) following painting, fire or chemical release that could have contaminated the charcoal adsorbers or HEPA filter in any ventilation zone communicating with the system by:

*Not applicable during initial fuel loading.

j. Offsite releases of radioactive materials in liquid and gaseous effluents which exceed the limits of Specification 3.11.1.1 or 3.11.2.1.

k. Exceeding the limits in Specification 3.11.1.4 or 3.11.2.6 for the storage of radioactive materials in the listed tanks. The written follow-up report shall include a schedule and a description of activities planned and/or taken to reduce the contents to within the specified limits.

THIRTY-DAY WRITTEN REPORTS

6.9.1.13 The types of events listed below shall be the subject of written reports to the Director of the Regional Office within 30 days of occurrence of the event. The written report shall include, as a minimum, a completed copy of a licensee event report form. Information provided on the licensee event report form shall be supplemented, as needed, by additional narrative material to provide complete explanation of the circumstances surrounding the event.

a. Reactor protection system or engineered safety feature instrument settings which are found to be less conservative than those established by the Technical Specifications but which do not prevent the fulfillment of the functional requirements of affected systems.

b. Conditions leading to operation in a degraded mode permitted by a Limiting Condition for Operation or plant shutdown required by a Limiting Condition for Operation.

c. Observed inadequacies in the implementation of administrative or procedural controls which threaten to cause reduction of degree of redundancy provided in reactor protection systems or engineered safety feature systems.

d. Abnormal degradation of systems other than those specified in 6.9.1.12.c above designed to contain radioactive material resulting from the fission process.

e. An unplanned offsite release of 1) more than 1 curie of radioactive material in liquid effluents, 2) more than 150 curies of noble gas in gaseous effluents, or 3) more than 0.05 curies of radioiodine in gaseous effluents. The report of an unplanned offsite release of radioactive material shall include the following information:

1. A description of the event and equipment involved.
2. Cause(s) for the unplanned release.
3. Actions taken to prevent recurrence.
4. Consequences of the unplanned release.

f. Measured levels of radioactivity in an environmental sampling medium determined to exceed the reporting level values of Table 3.12-2 when average over any calendar quarter sampling period.

RADIAL PEAKING FACTOR LIMIT REPORT

6.9.1.14 The F_{xy} limit for Rated Thermal Power (F_{xy}^{RTP}) shall be provided to the Director of the Regional Office of Inspection and Enforcement, with a copy to the Director, Nuclear Reactor Regulation, Attention Chief of the Core Performance Branch, U.S. Nuclear Regulatory Commission, Washington, D.C. 20555 for all core planes containing bank "D" control rods and all unrodded core planes at least 60 days prior to cycle initial criticality. In the event that the limit would be submitted at some other time during core life, it will be submitted 60 days prior to the date the limit would become effective unless otherwise exempted by the Commission.

Any information needed to support F_{xy}^{RTP} will be by request from the NRC and need not be included in this report.

SPECIAL REPORTS

6.9.2 Special reports shall be submitted to the Director of the Office of Inspection and Enforcement Regional Office within the time period specified for each report.

6.10 RECORD RETENTION

In addition to the applicable record retention requirements of Title 10, Code of Federal Regulations, the following records shall be retained for at least the minimum period indicated.

6.10.1 The following records shall be retained for at least five years:

a. Records and logs of unit operation covering time interval at each power level.

b. Records and logs of principal maintenance activities, inspections, repair and replacement of principal items of equipment related to nuclear safety.

c. All REPORTABLE OCCURRENCES submitted to the Commission.

d. Records of surveillance activities, inspections and calibrations required by these Technical Specifications.

e. Records of changes made to the procedures required by Specification 6.8.1.

f. Records of radioactive shipments.

g. Records of sealed source and fission detector leak tests and results.

JUN 15 1981

SAFETY EVALUATION REPORT

JOSEPH M. FARLEY NUCLEAR PLANT, UNITS 1 & 2

DIESEL GENERATOR TECHNICAL SPECIFICATION CHANGE

Introduction

The primary purpose of the 2C diesel generator at the Farley Plant is to start the five river water pumps of the river water system's Train B. The attached table shows that emergency bus 1J of Unit 1 is energized 10 seconds following receipt of signal and river water pumps 4 and 5 are started at five second intervals.

Since diesel 2C is relatively small at 2850kw output, care is taken to prevent two river water pumps from starting simultaneously so that the diesel will not trip. A time delay was designed into the system so that emergency bus 2J of Unit 2 is not energized until approximately 21 seconds following receipt of signal. As shown in the attached table, the number 1, 2 and 3 river water pumps are started in sequence after the bus is energized.

Farley, Unit 2 utilizes the Standard Technical Specifications. One of the requirements is that each diesel generator shall start and energize its emergency buses within 10 seconds following receipt of signal. Although a time delay was attached to emergency bus 2J by design, diesel 2C does not meet this requirement.

By letter dated May 5, 1981, the licensee requested a Technical Specification change to allow diesel generator 2C 24 seconds to energize its emergency buses. All of the remaining diesels will meet the 10 second requirement.

Evaluation

The proposed Technical Specification change does not involve a safety consideration. The safety analysis involving the river water pumps has previously included the time delay. The purpose of the river water pumps is to maintain the service water pond. Pond water flows to the common service water suction pit, flows to the two units and then it is normally discharged directly to the river. Under transient conditions, the service water can be pumped directly back to either the pond or the service water suction pit thus reducing or eliminating any immediate need of the river water pumps. If, however, the seismically designed dam failed causing loss of the service water pond, the river water pumps would be relied upon to discharge directly into the service water suction pit.

If diesel generator 2C energized both emergency buses 1J and 2J ten seconds following receipt of signal, the automatic sequencing would attempt to start two river water pumps simultaneously. In order to avoid any potential of tripping the diesel, the liensee has the following time delay for bus 2J:

```
  1 second for intermediate relay actuation and breaker tip
  1 second for closing of the output breaker
 20 second time delay for relay actuation
+ 2 seconds tolerance for relay actuation
 24 seconds total time to energize bus 2J
```

All river water pumps from Train B are therefore postulated to be in operation 36 seconds following the initiating event. The river water pumps on the redundant Train A (whose emergency buses are all energized within 10 seconds) are all postulated to be in operation 32 seconds following the initiating event.

The proposed Technical Specification change has been made to accomodate the system design. There are no deficiencies in either the equipment or its design to prompt this change. There are no safety considerations involved because the plant's analysis has previously included the time delay.

Due to the above considerations, we find the proposed change to the plant's Technical Specifications to be acceptable.

Environmental Consideration

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the amendment does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered

by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

JUN 15 1981

Diesel Generator 2C

Loss of Offsite Power (LOSP)

Typical Sequence of Events

Time	Event
0	LOSP (Signal to start diesel 2C and trip breaker DG-13)
1 SEC	Bus 1J and 2J load shed and starts 20 second timer for Bus 2J
10 SEC	2C D/G energized Bus 1J and its permanent loads
15 SEC	RW 4 starts
20 SEC	RW 5 starts
21 SEC	2C D/G energizes Bus 2J and its permanent loads
26 SEC	RW 1 starts
31 SEC	RW 2 starts
36 SEC	RW 3 starts

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NO. 50-364

ALABAMA POWER COMPANY

NOTICE OF ISSUANCE OF AMENDMENT TO FACILITY OPERATING LICENSE NO. NPF-8

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 1 to Facility Operating License No. NPF-8. This amendment was issued to Alabama Power Company for the Joseph M. Farley Nuclear Plant, Unit 2 to correct typographical errors in the Technical Specifications and to allow a 24 second delay in energizing one emergency bus for one diesel generator.

The Joseph M. Farley Nuclear Plant is located in Houston County, Alabama. This amendment was effective on May 6, 1981.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's requirements. The Commission has made appropriate findings as required by the Act and the Commission's regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment was not required since the amendment does not involve a significant hazards considerations.

The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR Section 51.5(d)(4) an Environmental Impact Statement, or Negative Declaration and Environmental Impact Appraisal need not be prepared in connection with issuance of this amendment.

For further details with respect to this action, see (1) the application for amendment, dated May 5, 1981; (2) Amendment No. 1 to Facility Operating License NPF-8; and (3) the Commission's related Safety Evaluation. All of these

documents are available for public inspection at the Commission's Public Document Room, located at 1717 H Street, N. W., Washington, D. C. 20555 and at the G. S. Houston Memorial Library, 212 W. Burdeshaw Street, Dothan, Alabama 36303 or may be requested by writing to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555 Attention: Director, Technical Information and Document Control.

Dated at Bethesda, Maryland, this 15th day of June, 1981.

FOR THE NUCLEAR REGULATORY COMMISSION

/s/

B. J. Youngblood, Chief
Licensing Branch No. 1
Division of Licensing

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