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OCT 01 1981

Docket No. 50-364

Mr. F. L. Clayton  
 Senior Vice President  
 Alabama Power Company  
 Post Office Box 2641  
 Birmingham, Alabama 35291

Dear Mr. Clayton:

The Commission has issued the enclosed Amendment No. 2 to Facility Operating License No. NPF-8 for the Joseph M. Farley Nuclear Plant, Unit No. 2. The amendment consists of changes to the Technical Specifications in response to your application transmitted by telecopy request dated September 28, 1981.

This amendment modifies the telephone authorization provided September 27 and 28, 1981 (confirmed by letter dated September 29, 1981) which allowed continued operation until October 2, 1981. This amendment allows additional time to complete the repairs.

The amendment allows Unit 2 to continue operation for a total of seventeen days while repairs are accomplished on diesel generator 2c. During this time, Unit 1 is shutdown for major turbine generator overhaul and diesel generator 1c is also being repaired.

Copies of the Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

Original signed by:  
 S. A. Varga

Steven A. Varga, Chief  
 Operating Reactors Branch #1  
 Division of Licensing



Enclosures:

1. Amendment No. 2 to NPF-8
2. Safety Evaluation
3. Notice of Issuance

cc/w/enclosures:  
 See next page

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Mr. F. L. Clayton  
Alabama Power Company

cc: Mr. W. O. Whitt  
Executive Vice President  
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Chairman  
Houston County Commission  
Dothan, Alabama 36301

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George S. Houston Memorial Library  
212 W. Burdeshaw Street  
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Resident Inspector  
U. S. Nuclear Regulatory Commission  
Post Office Box 24-Route 2  
Columbia, Alabama 36319

State Department of Public Health  
ATTN: State Health Officer  
State Office Building  
Montgomery, Alabama 36104

Regional Radiation Representatives  
EPA Region IV  
345 Courtland Street, N.E.  
Atlanta, Georgia 30308



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

ALABAMA POWER COMPANY

DOCKET NO. 50-364

JOSEPH M. FARLEY NUCLEAR PLANT, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 8  
License No. NPF-8

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The request for amendment by Alabama Power Company (the licensee) dated September 28, 1981, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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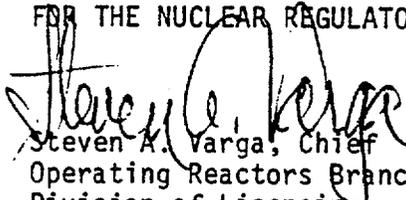
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-8 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 8, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment was effective September 27, 1981.

FOR THE NUCLEAR REGULATORY COMMISSION

  
Steven A. Varga, Chief  
Operating Reactors Branch #1  
Division of Licensing

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: October 1, 1981

ATTACHMENT TO LICENSE AMENDMENT  
AMENDMENT NO. 8 TO FACILITY OPERATING LICENSE NO. NPF-8  
DOCKET NO. 50-364

Revise Appendix A as follows:

<u>Remove Pages</u>	<u>Insert Pages</u>
3/4 8-1	3/4 8-1
3/4 8-2	3/4 8-2
3/4 8-3	3/4 8-3
3/4 8-4	3/4 8-4

### 3/4.8.1 A.C. SOURCES

#### OPERATING

#### LIMITING CONDITION FOR OPERATION

3.8.1.1 As a minimum, the following A.C. electrical power sources shall be OPERABLE:

- a. Two physically independent circuits between the offsite transmission network to the switchyard and two physically independent circuits from the switchyard to the onsite Class 1E distribution system, and
- b. Two separate and independent diesel generator sets (Set A: DG 1-2A and DG-1C, Set B: DG-2B and DG-2C) each with:
  1. Separate day tanks containing a minimum volume of 900 gallons of fuel for the 4075 kw diesel generators and 700 gallons of fuel for the 2850 kw diesel generator.
  2. A separate fuel transfer pump for each diesel.
- c. A fuel storage system consisting of four, independent storage tanks each containing a minimum of 25,000 gallons of fuel.\*

APPLICABILITY: MODES 1, 2, 3 and 4.

#### ACTION:

- a. With an offsite circuit inoperable, demonstrate the OPERABILITY of the remaining A.C. sources by performing Surveillance Requirements 4.8.1.1.1.a and 4.8.1.1.2.a.4 within one hour and at least once per 8\*\*\* hours thereafter; restore at least two offsite circuits to OPERABLE status within 72\*\* hours or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.
- b. With one diesel generator set inoperable, demonstrate the operability of the remaining A.C. sources by performing Surveillance Requirements 4.8.1.1.1.a and 4.8.1.1.2.a.4 within one hour and at least once per 8 hours thereafter. Restore both diesel generator sets to OPERABLE status within 72\*\* hours or comply with the following:
  - 1) Be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

\*One inoperable fuel storage tank is equivalent to one inoperable diesel generator set.

\*\*One time only exception for repair of Diesel 2C - the 72 hour action statement for operability of Diesel 2C may be extended to a period of 17 days provided Diesel 2C is returned to OPERABLE status as soon as maintenance is completed. The provisions of specification 3.0.4 are not applicable for this one time change.

\*\*\*One time only exception during repair of Diesel 2C - the 8 hour interval test is extended to 72 hours.

- 2) One diesel generator set may be made inoperable for up to 14 days to perform scheduled maintenance and testing on diesel generators 1C (or 2C) provided all the following are satisfied:
- a) Unit 1 is in MODE 5 or 6 and appropriate technical specifications covering the diesel generator sets are satisfied.
  - b) The remaining Unit 2 diesel generators 1-2A, 2B, 1C (or 2C) are OPERABLE.
  - c) The service water system is recirculated to the pond and surveillance requirement 4.7.6.2.1 is verified prior to removing 1C (or 2C) from service and once per 8 hours thereafter.
  - d) Diesel Generator 1C (or 2C) is returned to OPERABLE status as soon as maintenance is completed.

Otherwise, be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

- c. With one offsite circuit and one diesel generator set of the above required A.C. electrical power sources inoperable, demonstrate the OPERABILITY of the remaining A.C. sources by performing Surveillance Requirements 4.8.1.1.1.a and 4.8.1.1.2.a.4 within one hour and at least once per 8 hours thereafter; restore at least one of the inoperable sources to OPERABLE status within 12 hours or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours. Restore at least two offsite circuits and both diesel generator sets to OPERABLE status within 72\*hours from the time of initial loss or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.
- d. With two of the above required offsite A.C. circuits inoperable, demonstrate the OPERABILITY of both diesel generator sets by performing Surveillance Requirement 4.8.1.1.2.a.4 within one hour and at least once per 8 hours thereafter, unless the diesel generators are already operating; restore at least one of the inoperable offsite sources to OPERABLE status within 24 hours or be in at least HOT STANDBY within the next 6 hours. With only one offsite source restored, restore both offsite circuits to OPERABLE status within 72 hours from time of initial loss or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

\*One time only exception for repair of Diesel 2C - the 72 hour action statement for operability of Diesel 2C may be extended to a period of 17 days provided Diesel 2C is returned to OPERABLE status as soon as maintenance is completed. The provisions of specification 3.0.4 are not applicable for this one time change.

\*\*One time only exception during repair of Diesel 2C - the 3 hour interval test is extended to 72 hours.

## ELECTRICAL POWER SYSTEM

### ACTION: (Continued)

- e. With both of the above required diesel generator sets inoperable, demonstrate the OPERABILITY of two offsite A.C. circuits by performing Surveillance Requirement 4.8.1.1.a within 1 hour and at least once per 8 hours\* thereafter; restore at least one of the inoperable diesel generator sets to OPERABLE status within 2 hours\*\*\* or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours. Restore both diesel generator sets to OPERABLE status within 72\* hours from time of initial loss or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

### SURVEILLANCE REQUIREMENTS

4.2.1.1.1 Each of the above required independent circuits between the offsite transmission network and the onsite Class 1E distribution system shall be:

- a. Determined OPERABLE at least once per 7 days by verifying correct breaker alignments, indicated power availability, and
- b. Demonstrated OPERABLE at least once per 18 months during shutdown by transferring unit power supply from the normal circuit to the alternate circuit.

4.8.1.1.2 Each diesel generator shall be demonstrated OPERABLE:

- a. In accordance with the frequency specified in Table 4.8-1 on a STAGGERED TEST BASIS by:
  1. Verifying the fuel level in the day tank,
  2. Verifying the fuel level in the fuel storage tanks;
  3. Verifying the fuel transfer pump can be started and transfers fuel from the storage system to the day tank,
  4. Verifying the diesel starts from ambient condition and accelerates to at least 900 rpm, for the 2850 kw generator and 514 rpm for the 4075 kw generators, in less than or equal to 10 seconds. The generator voltage and frequency shall be  $\geq 3952$  volts and  $\geq 57$  Hz within 10 seconds after the start signal.
  5. Verifying the generator is synchronized, loaded to greater than or equal to its continuous rating, and operates for greater than or equal 60 minutes,

\* - One time only exception for repair of Diesel 2C - 72 hour action statement for operability of Diesel 2C may be extended to a period of 17 days provided Diesel 2C is returned to OPERABLE status as soon as maintenance is completed. The provisions of specification 3.0.4 are not applicable for this one time change.

\*\* - One time only exception during repair of Diesel 2C - the 8 hour interval test is extended to 72 hours.

\*\*\* - One time only exception for repair of Diesel 1C during the current refueling outage of Unit 1 - the 2 hour action statement for operability of the diesel set which contains Diesel 1C may be extended to a period of 8 days (coincident with the return to operable status of Diesel 1C as provided in Section 3.8.1.1.b.2) of Unit 2 Technical Specifications.) The provisions of specification 3.0.4 are not applicable for this one time change.

## ELECTRICAL POWER SYSTEMS

### SURVEILLANCE REQUIREMENTS (Continued)

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6. Verifying the diesel generator is aligned to provide standby power to the associated emergency busses.
- b. At least once per 92 days by verifying that a sample of diesel fuel from the fuel storage tank obtained in accordance with ASTM-D270-65, is within the acceptable limits specified in Table 1 of ASTM D975-74 when checked for viscosity, water and sediment.
- c. At least once per 18 months by:
  1. Subjecting the diesel to an inspection in accordance with procedures prepared in conjunction with its manufacturer's recommendations for this class of standby service,
  2. Verifying the capability to reject a load of greater than or equal to the largest single load associated with that diesel generator, while maintaining voltage between 3120 and 4910 volts and speed less than or equal to 75% of the difference between nominal speed and the overspeed trip setpoint and verifying recovery to  $4160 \pm 420$  volts and  $60 \pm 1.2$  Hz within 2 seconds.
  3. Verifying the generator capability to reject a load equal to its continuous rating without tripping. The generator voltage shall not exceed 120% during and following the load rejection.
  4. Simulating a loss of offsite power by itself, and:
    - a) Verifying de-energization of the emergency busses and load shedding from the emergency busses.
    - b) Verifying the diesel starts on the auto-start signal, energizes the emergency busses with permanently connected loads within 10 seconds, energizes the auto-connected shutdown loads through the load sequencer and operates for greater than or equal to 5 minutes while its generator is loaded with the shutdown loads. After energization of all loads, the steady state voltage and frequency of the emergency busses shall be maintained at  $4160 \pm 420$  volts and  $60 \pm 1.2$  Hz during this test.
  5. Verifying that on an Safety Injection test signal (without loss of offsite power) the diesel generator starts on the auto-start signal and operates on standby for greater than or equal to 5 minutes. The generator voltage and frequency shall be  $\geq 3952$  volts and  $> 57$  Hz within 10 seconds after the auto-start signal; the steady state generator voltage and frequency shall be maintained between  $4160 \pm 420$  volts and  $60 \pm 1.2$  Hz during this test.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
RELATED TO AMENDMENT NO. 8 TO FACILITY OPERATING LICENSE NO. NPF-8  
ALABAMA POWER COMPANY  
JOSEPH M. FARLEY NUCLEAR PLANT, UNIT 2  
DOCKET NO. 50-364

Introduction

The two unit Farley plant has five diesel generators (DG). Units 1 and 2 have dedicated DG-1B and DG-2B, respectively. The remaining DG-1C, DG-2C and DG 1-2A are swing diesels capable of serving either unit.

Unit 1 is currently shutdown due to repairs of the main turbine generator. Unit 1 is not expected to be ready to resume power operations until January, 1982.

Unit 2 is currently operating at 100% power. Technical Specifications (TS) 3.8.1.1.b for Unit 2 state that two separate and independent diesel generator sets must be operable (Set A: DG 1-2A and DG-1C, Set B: DG-2B and DG-2C). The associated Action statements require that:

- 3.8.1.1.b) With one DG set inoperable, restore both sets to operable status within 72 hours.
- 3.8.1.1.b.2) With Unit 1 in mode 5 or 6, either DG-1C or DG-2C may be inoperable for up to 14 days to perform scheduled maintenance. (The primary purpose of these DG is to provide power for the train A and train B river water systems respectively).
- 3.8.1.1.c) With both DG sets inoperable, restore both sets to operable status within 2 hours.

If any of the above three Action statements cannot be met, the Unit 2 facility must be placed in the Hot Standby condition within 6 hours.

On September 27, 1981, Unit 2 was in the fifth day of Action statement 3.8.1.1.b.2. With Unit 1 shutdown in Mode 5, DG-1C was inoperable due to scheduled maintenance. While attempting to perform routine surveillance testing on DG-2C, the engine tripped on high crankcase pressure after being successfully started and loaded for thirty minutes. DG-2C was immediately declared inoperable. Preliminary investigations indicated a crack in the liner seal between the jacket water system and the air intake system which allowed water to enter one of the cylinders.

Since DG-1C and DG-2C are now both inoperable, Unit 2 entered Action statement 3.8.1.1.e which calls for plant shutdown within two hours. Based on the remaining DG capacity to provide power to at least one safety related train for each unit, including the river water system, under all postulated accident conditions, NRC management gave verbal agreement to extend TS Action statement 3.8.1.1.e for 33 hours to get through the current weekend and provide the NRC staff one complete working day to further analyze the situation.

On September 28, 1981 the licensee stated that DG-2C needed to be disassembled and repaired. Their preliminary investigation indicated a need to replace the liner, o-rings, gaskets, pistons, wrist pins and bushings for all the affected cylinders. In addition, all the cylinder liners would be evaluated and probably replaced. The licensee has estimated that this work will take approximately 17 days to accomplish.

With Unit 1 shutdown for repairs, the licensee believes there is a critical need for the operation of Unit 2 for the Southern Company Power Pool. Therefore, in order to prevent shutting down Unit 2, the licensee has requested the following, one time only extensions to the Farley TS to allow for the repair of DG-2C:

- 1) TS 3.8.1.1.b needs to be extended to allow one DG set to remain inoperable up to 17 days (an increase from 72 hours).
- 2) TS 3.8.1.1.e needs to be extended to allow both DG sets to remain inoperable up to eight days (an increase from 2 hours). (On September 28 the licensee was in day 6 of Action statement 3.8.1.1.b.2. By extending this TS for eight days, the licensee is assured of having the full 14 days originally allowed for the scheduled maintenance of DG-1C).
- 3) TS 3.8.1.1 also states that if any of the DG sets or offsite power sources is inoperable, the remaining AC sources must be tested in order to show operability every eight hours. Due to the expected time required to repair DG-2C, over 100 starts would be required from the remaining DG. Since the DG manufacturer recommends against this testing frequency, the licensee proposes to extend the frequency from 8 to 72 hours in order to avoid any potential, accelerated wear.

#### Discussion and Evaluation Availability of Safety Trains

The two DG currently out of service are the smaller (2850 Kw) units. The remaining three DG have a larger (4075 Kw) capacity. The NRC staff has examined the DG schematic arrangement and has discussed the load sharing capability of the remaining DG with the licensee. The licensee has shown that for all combinations of loss of offsite power with and without a coincident LOCA at one of the units, there will be at least

one train of safety related equipment available at each unit. In addition, the staff has investigated the possibility of each unit experiencing a single failure of a remaining diesel generator coincident with loss of offsite power to both units and a LOCA occurring at one unit. But even under these postulated conditions, the flexibility of the diesels circuitry results in one safety train being available on each unit to supply power to the required loads.

The primary purpose of the two DG out of service is to supply power to the river water pumps. The river water pumps supply water to the pond. The safety related service water system takes suction from the pond. The pond has a 30 day capacity according to FSAR analysis. Service water exiting from Unit 2 can either go directly back to the river or be recirculated back to the pond.

The licensee has stated that under worst case conditions (i.e., loss of pond dam coincident with plant LOCA), only two river water pumps would be necessary per unit. With the three remaining diesel generators, there will be the capability to automatically run six river water pumps. Operator action will not be necessary to actuate the river water system.

#### Station Blackout

Station blackout is characterized by the loss of both offsite and emergency AC power for an extended period of time. Core melt can occur if the turbine-driven auxiliary feedwater system fails or if the reactor coolant pump seals fail subsequently because of lack of cooling if no corrective actions are taken.

In the staff's evaluation found in License Amendment No. 20 for Unit 1 and No. 2 for Unit 2, an estimate was made of the core melt probabilities assuming that diesel generator 1C would be inoperable for a total of 13 days. Since the licensee has requested a 17 day outage for diesel generator 2C, the probabilities can be assumed to be nearly identical to that previously reported and found to be acceptably low.

#### Diesel Testing

The reduced testing frequency requested is acceptable provided staggered testing of the four diesels is scheduled within the 72 hour time frame. The following actions are being taken during recovery from this event:

1. Plant procedures will assure operators are aware of the staggered diesel test frequency and proper bus loading procedures with diesel 2C out of commission.
2. Each Senior Reactor Operator will brief each oncoming shift and the Shift Technical Advisor.

### Summary

The licensee has shown that for all combinations of loss of offsite power with and without a coincident LOCA at one of the units, there will always be power available to run at least one of the redundant safety trains at each unit. Staff analyses also showed that the inclusion of single failures of a remaining diesel generator at either or both units would not change this result.

The probability of core melt during the one-time 17 day Technical Specification change is acceptably low. Therefore, the proposed Technical Specification change is acceptable on a one-time only basis.

Currently, the licensee, with a team of engineers and technicians from the DG vendor, is undertaking a thorough investigation/repair program of DG-2C. DG-1C is nearly ready for operation and it can provide additional margin if needed. We believe that the licensee has taken prompt and appropriate action to correct the DG failure.

Based on our review of the system and discussions with the licensee, we believe that the remaining DG will provide sufficient capacity to automatically operate all necessary safety related equipment following a postulated accident. Therefore, we conclude that the proposed TS change should be acceptable.

### Environmental Consideration

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement, or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

### Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the amendment does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Date: October 1, 1981

UNITED STATES NUCLEAR REGULATORY COMMISSIONDOCKET NO. 50-364ALABAMA POWER COMPANYNOTICE OF ISSUANCE OF AMENDMENT TO FACILITY  
OPERATING LICENSE

The U.S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 8 to Facility Operating License No. NPF-8 issued to Alabama Power Company (the licensee), which revised Technical Specifications for operation of the Joseph M. Farley Nuclear Plant, Unit No. 2 (the facility) located in Houston County, Alabama. The amendment was effective September 27, 1981.

The amendment was authorized by phone on September 27 and 28, 1981, and was confirmed by letter dated September 29, 1981. The amendment grants temporary relief from diesel generator operability and surveillance frequency requirements. The amendment allows Unit 2 to continue operation for a total of seventeen days while repairs are accomplished on diesel generator 2c. During this time, Unit 1 is shutdown for major turbine generator overhaul and diesel generator 1c is also being repaired. The amendment was issued on an expedited basis to maintain the plant at a steady-state condition and avoid a shutdown transient shown by our evaluation to be unnecessary but required by Technical Specifications unless amended.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made

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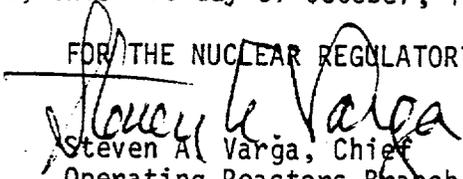
appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment was not required since this amendment does not involve a significant hazards consideration.

The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of this amendment.

For further details with respect to this action, see (1) the request for amendment dated September 28, 1981, (2) the Commission's letter to the licensee dated September 29, 1981, (3) Amendment No. 8 to License No. NPF-8, and (4) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D.C. and at the George S. Houston Memorial Library, 212 W. Burdeshaw Street, Dothan, Alabama 36303. A copy of items (2), (3) and (4) may be obtained upon request addressed to the U.S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 1st day of October, 1981.

FOR THE NUCLEAR REGULATORY COMMISSION

  
Steven A. Varga, Chief  
Operating Reactors Branch #1  
Division of Licensing