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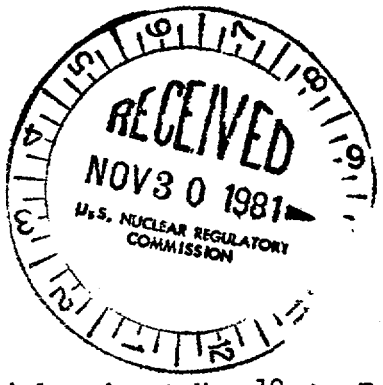
Docket No. 50-364



Mr. F. L. Clayton
Senior Vice President
Alabama Power Company
Post Office Box 2641
Birmingham, Alabama 35291

Dear Mr. Clayton:

The Commission has issued the enclosed Amendment No. 10 to Facility Operating License No. NPF-8 for the Joseph M. Farley Nuclear Plant, Unit No. 2. The amendment consists of changes to the Technical Specifications in response to your application transmitted by letter dated October 8, 1981.

The amendment modifies Technical Specifications related to primary containment leakage testing. In connection with this action, the Commission has granted a one-time exemption to 10 CFR 50.46, Appendix J. The action approves a request for an extension to the 24-month test interval for certain Type B and C containment penetration tests until plant conditions permit, but no later than the first refueling outage. We made minor changes to the proposed Technical Specifications to which your staff agreed.

We have reviewed the proposed testing schedule which will test the majority of containment penetrations as plant conditions permit within the 24-month test interval. The remaining penetrations will be tested if plant conditions permit, but in no case shall any penetration test interval extend beyond the first refueling outage. We find that granting the proposed exemption from requirements of Appendix J is authorized by law and will not endanger life or property or the common defense and security and is otherwise in the public interest.

Copies of the Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

Robert A. Purple for
Darrell G. Eisenhut, Director
Division of Licensing

Enclosures:
1. Amendment No. 10 to NPF-8
2. Safety Evaluation
3. Notice of Issuance

cc w/enclosures:
See next page

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no legal objection to and of notice only - see attached note re SE

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SURNAME	CParrish CP	EReeves:ds	SVarga	TNovak	DSwanson	DEisenhut	
DATE	11/16/81	11/16/81	11/16/81	11/17/81	11/14/81	11/18/81	

Mr. F. L. Clayton, Jr.
Alabama Power Company

cc: Mr. W. O. Whitt
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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

ALABAMA POWER COMPANY

DOCKET NO. 50-364

JOSEPH M. FARLEY NUCLEAR PLANT, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 10
License No. NPF-8

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. Pursuant to 10 CFR 50.12 of the Commission's Regulations, the Commission has authorized an exemption from the requirements of Appendix J to 10 CFR Part 50.

   B. The application for amendment by Alabama Power Company (the licensee) dated October 8, 1981, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   C. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   D. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   E. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   F. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-8 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 10, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Marshall Grotenhuis

Marshall Grotenhuis, Acting Chief
Operating Reactors Branch #1
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: November 19, 1981

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 10 TO FACILITY OPERATING LICENSE NO. NPF-8

DOCKET NO. 50-364

Revise Appendix A as follows:

Remove Page	Insert Page
3/4 6-3	3/4 6-3

SURVEILLANCE REQUIREMENTS (Continued)

b. If any periodic Type A test fails to meet either 0.75 L_a or 0.75 L_t, the test schedule for subsequent Type A tests shall be reviewed and approved by the Commission. If two consecutive Type A tests fail to meet either 0.75 L_a or 0.75 L_t, a Type A test shall be performed at least every 18 months until two consecutive Type A tests meet either 0.75 L_a or 0.75 L_t at which time the above test schedule may be resumed.

c. The accuracy of each Type A test shall be verified by a supplemental test which:

1. Confirms the accuracy of the Type A test by verifying that the difference between supplemental and Type A test data is within 0.25 L_a, or 0.25 L_t.

2. Has a duration sufficient to establish accurately the change in leakage rate between the Type A test and the supplemental test.

3. Requires the quantity of gas injected into the containment or bled from the containment during the supplemental test to be equivalent to at least 25 percent of the total measured leakage at P_a (48 psig) or P_t (24 psig.)

d. Type B and C tests shall be conducted with gas at P_a (48 psig) at intervals no greater than 24*months except for tests involving air locks.

e. Air locks shall be tested and demonstrated OPERABLE per Surveillance Requirement 4.6.1.3.

f. All test leakage rates shall be calculated using observed data converted to absolute values. Error analyses shall be performed to select a balanced integrated leakage measurement system.

g. The provisions of Specification 4.0.2 are not applicable.

*This is a one-time Exemption to 10 CFR 50.46 Appendix J. The 24 month interval may be extended during the first fuel cycle to allow individual penetrations to be tested as plant conditions permit, but in no case shall any individual test interval extend beyond the first refueling outage. If plant conditions support earlier testing or if incidents occur which could jeopardize the leak tight integrity of a penetration, the testing will be performed at that time.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO EXEMPTION AND AMENDMENT NO. 10 TO FACILITY OPERATING

LICENSE NO. NPF-8

ALABAMA POWER COMPANY

JOSEPH M. FARLEY NUCLEAR PLANT, UNIT NO. 2

DOCKET NO. 50-364

Introduction

Alabama Power Company (APCo), the licensee, requested by letter dated October 8, 1981 a one-time exemption from the requirements of Sections III.D.2.(a) and III.D.3 of 10 CFR Part 50, Appendix J, Primary Reactor Containment Leakage Testing for Water-Cooled Power Reactors. Technical Specification 4.6.1.2.d agrees with Appendix J and requires local leak rate tests (Type B and C) to be conducted during each reactor shutdown for refueling but in no case at intervals greater than every two years.

The licensee's request is prompted by a delay in the initial plant startup after the initial fuel load which has resulted in the need to perform the first periodic Type B and C leak rate tests before the first refueling outage which is scheduled for late 1982. To comply with the regulation and Technical Specification 4.6.1.2.d, the first periodic local leak rate tests (Type B and C) must be performed between November 1981 and June 1982. Since the preservice tests demonstrated extremely low leakage rates (less than 5 percent of allowable), APCo proposed a plan to complete most of the tests without plant shutdown to cold conditions. Further, APCo proposed a one-time change to Technical Specification 4.6.1.2.d which we modified slightly. APCo staff agreed to our change.

Discussion and Evaluation

The APCo plan proposed that of the 100 penetrations to be leak tested, 53 can be tested while the unit is at power; therefore, 53 penetrations will be tested within the prescribed two-year interval. Testing of 25 of the remaining penetrations would require that the unit be at cold shutdown conditions. APCo proposes to leak test the latter 47 penetrations as plant conditions permit but no later than the first refueling outage which is scheduled for the end of 1982.

Within the next year, we consider it very likely that plant conditions will permit leak testing of some of the 47 penetrations. We consider that any remaining penetration that is not tested until the first refueling outage is likely to maintain its leak tight integrity past the two year interval. This is because the pre-operational Type B and C tests demonstrated extremely low leakage rates. Moreover, incidents which could cause abnormal degradation to the integrity of the penetrations have not occurred. Based on the results of the pre-operational local leak rate tests, the total leakage of these remaining 47 penetrations represents only 2.3 percent of the maximum allowable leakage (0.6 La).

Penetrations which could provide a direct leakage path to the environment, e.g., personnel access hatches, containment purge lines, equipment hatch, and fuel transfer tube, have been or will be tested (due to test provisions which allow testing while at power) on a schedule consistent with the provisions of the current Technical Specifications. All the other penetrations do not constitute direct leakage paths from the containment. A pipe failure in conjunction with valve degradation would be necessary for any post accident release to occur outside of containment. Furthermore, the penetration room filtration system maintains the auxiliary building rooms adjacent to the containment at a slightly negative pressure during accident conditions. The filtration system would recirculate and filter any such releases prior to exhausting to the environment. Thus, the negative pressure ensures inleakage into the penetration rooms and filtration of any containment penetration leakage. The functional capability of this filtration system is periodically verified in accordance with the surveillance requirements of the Technical Specifications, Section 4.7.8.

Summary

Based on the foregoing discussion, it is the NRC staff judgment that APCo's proposal for performing the local (Type B and C) leak rate tests is acceptable and presents no undue risk to the health and safety of the public. In summary, the basis for our conclusion is:

(1) Penetrations which are potentially direct release paths to the environment are being tested in accordance with the provisions of the Technical Specifications;

(2) Other penetrations require multiple failures for post-accident containment releases to the auxiliary building;

(3) Such releases would be processed by the penetration filtration system;

(4) The 47 penetrations that cannot be tested while at power have leakage expectations that consistute a small fraction of the total allowable local leakage; and

(5) No appreciable degradation of the 47 penetrations is expected during the remainder of the first fuel cycle.

Since extending the time interval for Type B and C local leak rate tests on 47 penetrations does not present an undue risk to the health and safety of the public, we find the proposal acceptable. Thus, we grant a one-time Exemption to Appendix J and approve a change to Technical Specification 4.6.1.2.d to extend the test interval for these penetrations to no later than the first refueling outage. If plant conditions support earlier testing of the penetrations or if incidents occur which could jeopardize the leak tight integrity of a penetration testing will be performed at that time.

Environmental Consideration

We have determined that the Exemption and amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the Exemption and amendment involve an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.1(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with this action.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the Exemption, pursuant to 10 CFR 50.12 is authorized by law and will not endanger life or property or the common defense and security and is otherwise in the public interest, and the amendment do not involve a significant decrease in a safety margin, the Exemption and amendment do not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the granting of this Exemption and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: November 19, 1981

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NO. 50-364

ALABAMA POWER COMPANY

NOTICE OF ISSUANCE OF AMENDMENT TO FACILITY OPERATING LICENSE, AND

GRANTING OF EXEMPTION

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 10 to Facility Operating License No. NPF-8 issued to Alabama Power Company (the licensee), which revised Technical Specifications for operation of the Joseph M. Farley Nuclear Plant, Unit No. 2 (the facility) located in Houston County, Alabama. The amendment is effective as of the date of issuance.

The amendment modifies the Technical Specifications related to primary containment leakage testing. In connection with this action, the Commission has granted a one-time Exemption to 10 CFR 50.46, Appendix J. The action approves a request for an extension to the 24-month test interval for certain Type B and C containment penetration tests.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment was not required since this amendment does not involve a significant hazards consideration.

The Commission has determined that the issuance of this amendment will not result in any signficant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of this amendment.

The Commission has determined that, pursuant to 10 CFR 50.12, an exemption is authorized by law and will not endanger life or property or the common defense and security and is otherwise in the public interest. Therefore, the Commission has approved the exemption request identified above.

For further details with respect to this action, see (1) the application for amendment dated October 8, 1981, (2) Amendment No. 10 to License No. NPF-8, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D.C. and at the George S. Houston Memorial Library, 212 W. Burdeshaw Street, Dothan, Alabama 36303. A copy of items (2) and (3) may be obtained upon requested addressed to the U. S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 19th day of November, 1981.

FOR THE NUCLEAR REGULATORY COMMISSION

Marshall Grotenhuis

Marshall Grotenhuis, Acting Chief
Operating Reactors Branch #1
Division of Licensing