

October 17, 1997

Mr. D. N. Morey  
Vice President - Farley Project  
Southern Nuclear Operating  
Company, Inc.  
Post Office Box 1295  
Birmingham, Alabama 35201-1295

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SUBJECT: ISSUANCE OF AMENDMENTS - JOSEPH M. FARLEY NUCLEAR PLANT,  
UNITS 1 AND 2 (TAC NOS. M98269 AND M98270)

Dear Mr. Morey:

The Nuclear Regulatory Commission has issued the enclosed Amendment No. 130 to Facility Operating License No. NPF-2 and Amendment No. 123 to Facility Operating License No. NPF-8 for the Joseph M. Farley Nuclear Plant, Units 1 and 2. The amendments change the Technical Specifications (TS) in response to your submittal dated March 7, 1997, titled, "Containment Isolation Valves Surveillance Requirements."

The amendments change the TS for both Farley units to allow operability testing for certain containment isolation valves during defueled status.

A copy of the related Safety Evaluation is also enclosed. A Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

ORIGINAL SIGNED BY:

Jacob I. Zimmerman, Project Manager  
Project Directorate II-2  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Docket Nos. 50-348 and 50-364

Enclosures:

1. Amendment No. 130 to NPF-2
2. Amendment No. 123 to NPF-8
3. Safety Evaluation

cc w/encls: See next page

DOCUMENT NAME: G:\FARLEY\M99269.AMD

ADPR / TSB
W. BECKNER
10/15/97
YES / NO

OFFICE	PDII-2/PM	PDII-2/RM	PDII-2/LA	DSSA/SCSB	OGC	PDII-2/D
NAME	W. GLEAVES	J. ZIMMERMAN	L. BERRY	C. BERLINGER	e ma co	H. BERKOW
DATE	9/19/97	9/19/97	9/17/97	9/24/97	9/29/97	10/16/97
COPY	YES NO	YES NO	YES NO	YES NO	YES NO	YES NO

OFFICIAL RECORD COPY



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PDR ADOCK 05000348  
P PDR

NRG FILE CENTER COPY



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

October 17, 1997

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Southern Nuclear Operating  
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The amendments change the TS for both Farley units to allow operability testing for certain containment isolation valves during defueled status.

A copy of the related Safety Evaluation is also enclosed. A Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

A handwritten signature in black ink, appearing to read "Jacob I. Zimmerman".

Jacob I. Zimmerman, Project Manager  
Project Directorate II-2  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Docket Nos. 50-348 and 50-364

Enclosures:

1. Amendment No.130to NPF-2
2. Amendment No.123to NPF-8
3. Safety Evaluation

cc w/encls: See next page

Joseph M. Farley Nuclear Plant

cc:

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7388 N. State Highway 95  
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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

SOUTHERN NUCLEAR OPERATING COMPANY, INC.

ALABAMA POWER COMPANY

DOCKET NO. 50-348

JOSEPH M. FARLEY NUCLEAR PLANT, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 130  
License No. NPF-2

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Southern Nuclear Operating Company, Inc. (Southern Nuclear), dated March 7, 1997, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application; the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications, as indicated in the attachment to this license amendment; and paragraph 2.C.(2) of Facility Operating License No. NPF-2 is hereby amended to read as follows:

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P PDR

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 130 , are hereby incorporated in the license. Southern Nuclear shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Herbert N. Berkow, Director  
Project Directorate II-2  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: October 17, 1997

ATTACHMENT TO LICENSE AMENDMENT NO.130

TO FACILITY OPERATING LICENSE NO. NPF-2

DOCKET NO. 50-348

Replace the following page of the Appendix A Technical Specifications with the enclosed page. The revised area is indicated by marginal lines.

Remove

Page 3/4 6-15

Insert

Page 3/4 6-15

CONTAINMENT SYSTEMS

SURVEILLANCE REQUIREMENTS (Continued)

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4.6.3.2 Each isolation valve specified in Table 3.6-1 shall be demonstrated OPERABLE during the COLD SHUTDOWN or REFUELING MODES or defueled at least once per 18 months by:

- a. Verifying that on a Phase A containment isolation test signal, each Phase A isolation valve actuates to its isolation position.
- b. Verifying that on a Phase B containment isolation test signal, each Phase B isolation valve actuates to its isolation position.

4.6.3.3 The isolation time of each power operated or automatic valve of Table 3.6-1 shall be determined to be within its limit when tested pursuant to Specification 4.0.5.

4.6.3.4 The containment purge isolation valves shall be demonstrated OPERABLE prior to startup after each COLD SHUTDOWN if not performed in the previous 3 months by verifying that when the measured leakage rate is added to the leakage rates for all other Type B and C penetrations, the combined leakage rate is less than or equal to 0.60L. In addition, the leakage rate for the containment purge isolation valves shall be compared to the previously measured leakage rate (for the containment purge isolation valves) to detect excess valve degradation.

An engineering evaluation shall be performed to determine what corrective action, if any, is necessary.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

SOUTHERN NUCLEAR OPERATING COMPANY, INC.

ALABAMA POWER COMPANY

DOCKET NO. 50-364

JOSEPH M. FARLEY NUCLEAR PLANT, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 123  
License No. NPF-8

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Southern Nuclear Operating Company, Inc. (Southern Nuclear), dated March 7, 1997, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application; the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications, as indicated in the attachment to this license amendment; and paragraph 2.C.(2) of Facility Operating License No. NPF-8 is hereby amended to read as follows:

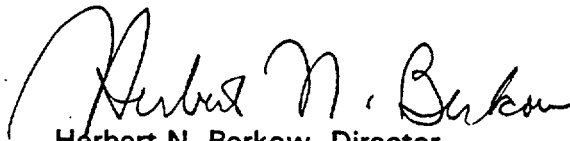


(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 123, are hereby incorporated in the license. Southern Nuclear shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Herbert N. Berkow, Director  
Project Directorate II-2  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: October 17, 1997

ATTACHMENT TO LICENSE AMENDMENT NO. 123

TO FACILITY OPERATING LICENSE NO. NPF-8

DOCKET NO. 50-364

Replace the following page of the Appendix A Technical Specifications with the enclosed page. The revised area is indicated by marginal lines.

Remove

Insert

Page 3/4 6-15

Page 3/4 6-15

## CONTAINMENT SYSTEMS

### SURVEILLANCE REQUIREMENTS (Continued)

---

4.6.3.2 Each isolation valve specified in Table 3.6-1 shall be demonstrated OPERABLE during the COLD SHUTDOWN or REFUELING MODES or defueled at least once per 18 months by:

- a. Verifying that on a Phase A containment isolation test signal, each Phase A isolation valve actuates to its isolation position.
- b. Verifying that on a Phase B containment isolation test signal, each Phase B isolation valve actuates to its isolation position.

4.6.3.3 The isolation time of each power operated or automatic valve of Table 3.6-1 shall be determined to be within its limit when tested pursuant to Specification 4.0.5.

4.6.3.4 The containment purge isolation valves shall be demonstrated OPERABLE prior to startup after each COLD SHUTDOWN if not performed in the previous 3 months by verifying that when the measured leakage rate is added to the leakage rates for all other Type B and C penetrations, the combined leakage rate is less than or equal to 0.60L. In addition, the leakage rate for the containment purge isolation valves shall be compared to the previously measured leakage rate (for the containment purge isolation valves) to detect excess valve degradation.

An engineering evaluation shall be performed to determine what corrective action, if any, is necessary.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
RELATED TO AMENDMENT NO. 130 TO FACILITY OPERATING LICENSE NO. NPF-2  
AND AMENDMENT NO. 123 TO FACILITY OPERATING LICENSE NO. NPF-8  
SOUTHERN NUCLEAR OPERATING COMPANY, INC., ET AL.  
JOSEPH M. FARLEY NUCLEAR PLANT, UNITS 1 AND 2  
DOCKET NOS. 50-348 AND 50-364

1.0 INTRODUCTION

By letter dated March 7, 1997, the Southern Nuclear Operating Company, Inc. (SNC) et al., submitted a request for changes to the Joseph M. Farley Nuclear Plant, Units 1 and 2, Technical Specifications (TS). The requested changes would revise the Surveillance Requirements (SR) associated with certain containment isolation valves (CIV). The proposed TS amendments add the defueled condition to operating modes listed in TS SR 4.6.3.2 where CIV operability testing is allowed.

2.0 EVALUATION

The proposed Farley TS amendments revise, for both Units 1 and 2, SR 4.6.3.2. The current SR 4.6.3.2 states:

- 4.6.3.2. Each isolation valve specified in Table 3.6-1 shall be demonstrated OPERABLE during the COLD SHUTDOWN or REFUELING MODE at least once per 18 months by:
- a. Verifying that on a Phase A containment isolation test signal, each Phase A isolation valve actuates to its isolation position.
  - b. Verifying that on a Phase B containment isolation test signal, each Phase B isolation valve actuates to its isolation position.

The proposed Farley TS amendments revise, for both Units 1 and 2, SR 4.6.3.2 to read:

- 4.6.3.2. Each isolation valve specified in Table 3.6-1 shall be demonstrated OPERABLE during the COLD SHUTDOWN or REFUELING MODES or defueled at least once per 18 months by:
- a. Verifying that on a Phase A containment isolation test signal, each Phase A isolation valve actuates to its isolation position.

- b. Verifying that on a Phase B containment isolation test signal, each Phase B isolation valve actuates to its isolation position.

Testing of the CIVs is intended to ensure that all CIVs are operable so the containment atmosphere can be isolated from the outside environment in the event of a release of radioactive material in the containment building. In this amendment request, the licensee proposes to add the defueled condition to the periods in which these valves can be tested. The defueled condition is an extension of the refueling mode. The condition exists when all reactor fuel is removed from the reactor core. In this state, containment integrity and the containment isolation functions are not necessary to limit the release of radioactive material in the containment atmosphere, since the reactor fuel would no longer be in the containment building.

The staff has reviewed the proposed TS amendments and concurs with SNC that the containment isolation function is not required to limit the release of radioactive material in the containment atmosphere postulated to occur during a design basis accident when the reactor is defueled. Therefore, the staff finds the proposed testing, in the defueled condition, of the containment isolation valves as specified in SRs 4.6.3.2(a) and 4.6.3.2(b) for both Farley Unit 1 and Unit 2 to be acceptable.

### **3.0 STATE CONSULTATION**

In accordance with the Commission's regulations, the State of Alabama official was notified of the proposed issuance of the amendments. The State official had no comments.

### **4.0 ENVIRONMENTAL CONSIDERATION**

The amendments change the surveillance requirements. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration, and there has been no public comment on such finding (62 FR 19834 dated April 23, 1997). Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

## 5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: W. Gleaves

Date: October 17, 1997