

November 9, 2001

Mr. Craig G. Anderson  
Vice President Operations, ANO  
Entergy Operations, Inc.  
1448 S. R. 333  
Russellville, AR 72801

SUBJECT: REQUEST FOR ADDITIONAL INFORMATION ON REACTOR COOLANT  
PUMP (RCP) FLYWHEEL INSPECTION INTERVAL APPLICATION -  
ARKANSAS NUCLEAR ONE, UNIT 2 (ANO-2) (TAC NO. MB2552)

Dear Mr. Anderson:

By application dated July 31, 2001, you proposed to revise the ANO-2 RCP flywheel inspection interval. The Nuclear Regulatory Commission (NRC) staff has reviewed the information provided in the application. In order for the NRC staff to continue its review, the NRC staff requires a response to the enclosed request for additional information (RAI).

The contents of this RAI have been discussed with Mr. Steve Bennett of your staff, and a response time frame of the end of November 2001, was agreed to. The NRC staff appreciates your efforts in regard to this matter.

If for any reason this date becomes unreasonable, please contact me at your earliest opportunity.

Sincerely,

**/RA/**

Thomas W. Alexion, Project Manager, Section 1  
Project Directorate IV  
Division of Licensing Project Management  
Office of Nuclear Reactor Regulation

Docket No. 50-368

Enclosure: RAI

cc: See next page

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Accession No.: ML013130320

OFFICE	PDIV-1/PM	PDIV-1/LA	EMCB	EMCB/SC	PDIV-1/SC
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DATE	11/08/01	11/8/01	11/07/01	11/07/01	11/9/01

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Request for Additional Information  
Reactor Coolant Pump (RCP) Flywheel Inspection Interval Application  
Arkansas Nuclear One, Unit 2

1. The licensee did not provide any basis for using the nil-ductility transition temperature (NDT) as the reference temperature ( $RT_{NDT}$ ) for the RCP flywheel material. Based on the two Charpy test data of 63 ft-lbs. and 105 ft-lbs. performed at 150°F, the NRC staff used the American Society of Mechanical Engineers (ASME) Code procedure and determined that the  $RT_{NDT}$  value of the RCP flywheel material is 90°F instead of 10°F as stated in the submittal, and the fracture toughness ( $K_{IC}$ ) is 50 ksi square-root-inch instead of 180 ksi square-root-inch as stated in the submittal. Provide additional information to support your  $RT_{NDT}$  value of 10°F. If you plan to propose an alternative methodology in determining the  $K_{IC}$  value to replace the use of the  $K_{IC}$  versus (T- $RT_{NDT}$ ) curve in the ASME Code, you need to submit a relief request.

Arkansas Nuclear One

cc:

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