

December 8, 1995

Mr. D. N. Morey
Vice President - Farley Project
Southern Nuclear Operating
Company, Inc.
Post Office Box 1295
Birmingham, Alabama 35201-1295

SUBJECT: ISSUANCE OF AMENDMENTS - JOSEPH M. FARLEY NUCLEAR PLANT,
UNITS 1 AND 2 (TAC NOS. M93781 and M93782)

Dear Mr. Morey:

The Nuclear Regulatory Commission has issued the enclosed Amendment No. 118 to Facility Operating License No. NPF-2 and Amendment No. 109 to Facility Operating License No. NPF-8 for the Joseph M. Farley Nuclear Plant, Units 1 and 2. The amendments change the Technical Specifications (TS) in response to your submittal dated September 26, 1995.

The amendments change the containment air lock door seal leakage rate from no detectable leakage to less than or equal to 0.01 L_a when the gap between the door seals is pressurized to greater than or equal to 10 psig for a period of not less than 15 minutes.

A copy of the related Safety Evaluation is also enclosed. A Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

Original signed by:
Byron L. Siegel, Senior Project Manager
Project Directorate II-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket Nos. 50-348
and 50-364

Enclosures:

1. Amendment No. 118 to NPF-2
2. Amendment No. 109 to NPF-8
3. Safety Evaluation

cc w/enclosures:

See next page

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UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

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Sincerely,

A handwritten signature in cursive script that reads "Byron L. Siegel".

Byron L. Siegel, Senior Project Manager
Project Directorate II-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

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See next page

Mr. D. N. Morey
Southern Nuclear Operating
Company, Inc.

Joseph M. Farley Nuclear Plant

cc:

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SOUTHERN NUCLEAR OPERATING COMPANY, INC.

ALABAMA POWER COMPANY

DOCKET NO. 50-348

JOSEPH M. FARLEY NUCLEAR PLANT, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 118
License No. NPF-2

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Southern Nuclear Operating Company, Inc. (Southern Nuclear), dated September 26, 1995, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications, as indicated in the attachment to this license amendment; and paragraph 2.C.(2) of Facility Operating License No. NPF-2 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 118, are hereby incorporated in the license. Southern Nuclear shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Herbert N. Berkow, Director
Project Directorate II-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: December 8, 1995

ATTACHMENT TO LICENSE AMENDMENT NO. 118
TO FACILITY OPERATING LICENSE NO. NPF-2
DOCKET NO. 50-348

Replace the following page of the Appendix A Technical Specifications with the enclosed page. The revised areas are indicated by marginal lines.

Remove Page

3/4 6-5

Insert Page

3/4 6-5

CONTAINMENT SYSTEMS

SURVEILLANCE REQUIREMENTS

- 4.6.1.3 Each containment air lock shall be demonstrated OPERABLE:
- a. After each opening, except when the air lock is being used for multiple entries, then at least once per 72 hours, by verifying the leakage rate is less than or equal to 0.01 L_a when the volume between the door seals is pressurized to greater than or equal to 10 psig for at least 15 minutes.
 - b. By conducting an overall air lock leakage test at not less than P_a (48 psig) and by verifying the over all airlock leakage rate is within its 0
 1. At least once per six months, and
 2. Prior to establishing CONTAINMENT INTEGRITY if opened when CONTAINMENT INTEGRITY was not required when maintenance has been performed on the air lock that could affect the air lock sealing capability,* and
 - c. At least once per six months by verifying that only one door in each air lock can be opened at a time.

The provisions of Specification 4.0.2 are not applicable.

* Exemption to Appendix "J" of 10 CFR 50.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SOUTHERN NUCLEAR OPERATING COMPANY, INC.

ALABAMA POWER COMPANY

DOCKET NO. 50-364

JOSEPH M. FARLEY NUCLEAR PLANT, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 109
License No. NPF-8

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Southern Nuclear Operating Company, Inc. (Southern Nuclear), dated September 26, 1995, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications, as indicated in the attachment to this license amendment; and paragraph 2.C.(2) of Facility Operating License No. NPF-8 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 109, are hereby incorporated in the license. Southern Nuclear shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Herbert N. Berkow, Director
Project Directorate II-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: December 8, 1995

ATTACHMENT TO LICENSE AMENDMENT NO.109
TO FACILITY OPERATING LICENSE NO. NPF-8
DOCKET NO. 50-364

Replace the following page of the Appendix A Technical Specifications with the enclosed page. The revised areas are indicated by marginal lines.

Remove Page

Insert Page

3/4 6-5

3/4 6-5

CONTAINMENT SYSTEMS

SURVEILLANCE REQUIREMENTS

- 4.6.1.3 Each containment air lock shall be demonstrated OPERABLE:
- a. After each opening, except when the air lock is being used for multiple entries, then at least once per 72 hours, by verifying the leakage rate is less than or equal to 0.01 L_a when the volume between the door seals is pressurized to greater than or equal to 10 psig for at least 15 minutes.
 - b. By conducting an overall air lock leakage test at not less than P_a (48 psig) and by verifying the over all airlock leakage rate is within its limit:
 1. At least once per six months, and
 2. Prior to establishing CONTAINMENT INTEGRITY if opened when CONTAINMENT INTEGRITY was not required when maintenance has been performed on the air lock that could affect the air lock sealing capability,* and
 - c. At least once per six months by verifying that only one door in each air lock can be opened at a time.

‡ The provisions of Specification 4.0.2 are not applicable.

* Exemption to Appendix "J" of 10 CFR 50.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 118 TO FACILITY OPERATING LICENSE NO. NPF-2
AND AMENDMENT NO. 109 TO FACILITY OPERATING LICENSE NO. NPF-8

SOUTHERN NUCLEAR OPERATING COMPANY, INC.

JOSEPH M. FARLEY NUCLEAR PLANT, UNITS 1 AND 2

DOCKET NOS. 50-348 AND 50-364

1.0 INTRODUCTION

By letter dated September 26, 1995, the Southern Nuclear Operating Company, Inc., et al. (the licensee), submitted a request for changes to the Joseph M. Farley Nuclear Plant, Units 1 and 2, Technical Specifications (TS). The proposed TS would change the containment air lock door seal leakage rate from no detectable leakage to less than or equal to 0.01 L_g when the gap between the door seals is pressurized to greater than or equal to 10 psig for a period of not less than 15 minutes.

2.0 EVALUATION

The licensee, in proposing to change the current acceptance criteria from "no detectable seal leakage by pressure decay when the volume between the door seals is pressurized to greater than or equal to 10 psig for at least 15 minutes" to a leakage rate of "less than or equal to 0.01 L_g when the gap between the door seals is pressurized to greater than or equal to 10 psig for a period of not less than 15 minutes," has concluded that no detectable leakage represents an impractical criteria to evaluate door seal performance. The licensee has determined that this change in seal leakage criteria will have no impact on the radiological consequences of the loss-of-coolant accident and the rod ejection accident, for which containment leakage is an assumption, because the plant safety analysis is based on a more conservative assumption that the containment leaks at its design leak rate of 0.15 percent per day for the first 24 hours and 0.075 percent per day thereafter for each of these accidents.

In accordance with the requirements of 10 CFR Part 50, Appendix J, Option A, Section III(D)(b)(iv), the licensee's proposed TS change is needed to provide complete documentation of the containment air lock acceptance criteria values. The overall leakage rate of the containment air lock doors has not been changed and remains at less than or equal to 0.05 L_g when tested at a pressure greater than or equal to P₀. The proposed leakage test for each set of door seals is consistent with the acceptance criteria stated in NUREG-1431, "Standard Technical Specifications - Westinghouse Plants" and comparable reduced pressure leak rates previously reviewed and approved by the staff for

other plants. The test pressure and duration (equal to or greater than 10 psig for at least 15 minutes) was previously approved by the staff in Supplement 4 of NUREG-0117, the Farley licensing Safety Evaluation Report.

Based on the above, the staff has concluded that the proposed TS change, which establishes an absolute value for the door seal leakage, constitutes performance of a reasonable test that assures the door seals will perform their design function during accident conditions, and is therefore acceptable.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the State of Alabama official was notified of the proposed issuance of the amendments. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

The amendments change the surveillance requirements. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration, and there has been no public comment on such finding (60 FR 56370 dated November 8, 1995). Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: Byron Siegel

Date: December 8, 1995

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