

June 17, 1994

Docket Nos. 50-348  
and 50-364

Mr. D. N. Morey, Vice President  
Southern Nuclear Operating Co., Inc.  
Post Office Box 1295  
Birmingham, Alabama 35201-1295

Dear Mr. Morey:

SUBJECT: CORRECTION TO THE AMENDMENT NOS. 107 FOR OPERATING LICENSE NO. NPF-2  
AND AMENDMENT NO. 99 OPERATING LICENSE NO. NPF-8 FOR THE JOSEPH M.  
FARLEY NUCLEAR PLANT, UNITS 1 AND 2 (TAC NOS. 88188 AND 88189)

On May 16, 1994, the Nuclear Regulatory Commission issued Amendment No. 107 to Facility Operating License Nos. NPF-2 and Amendment No. 99 to NPF-8 for the Joseph M. Farley Nuclear Plant, Units 1 and 2. The amendments changed the surveillance test intervals and allowed outage times for the reactor trip system and engineered safety feature actuation system instrumentation Technical Specifications (TS) in response to your October 14, 1993 application.

At the time Amendment Nos. 107 and 99 were issued, Amendment Nos. 104 and 97, which changed the TS as a result of a plant modification that eliminated the steam/feedwater flow mismatch reactor trip, had been already issued. Due to the timing of the issuance of the two amendments, Pages 3/4 3-13 in the amendments were inadvertently issued without including the changes that occurred in Amendment Nos. 104 and 99. A corrected Page 3/4 3-13 for both units is enclosed.

We regret any inconvenience this may have caused.

Sincerely,

ORIGINAL SIGNED BY:

Byron L. Siegel, Senior Project Manager  
Project Directorate II-1  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Enclosure:  
TS pages 3/4 3-13

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cc w/enclosure:  
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OFFICE	LA:PD2-1	PM:PD2-1	PD:PD2-1	
NAME	PAnderson	BSiegel	WBateman	
DATE	06/17/94	06/17/94	06/17/94	

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cc: Farley Service List



UNITED STATES  
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

June 17, 1994

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Post Office Box 1295  
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We regret any inconvenience this may have caused.

Sincerely,

A handwritten signature in cursive script that reads "Byron L. Siegel".

Byron L. Siegel, Senior Project Manager  
Project Directorate II-1  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Enclosure:  
TS pages 3/4 3-13

cc w/enclosure:  
See next page

Mr. D. N. Morey  
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Company, Inc.

Joseph M. Farley Nuclear Plant

cc:

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Executive Vice President  
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P.O. Box 1295  
Birmingham, Alabama 35201

Resident Inspector  
U.S. Nuclear Regulatory Commission  
7388 N. State Highway 95  
Columbia, Alabama 36319

TABLE 4.3-1 (Continued)

REACTOR TRIP SYSTEM INSTRUMENTATION SURVEILLANCE REQUIREMENTS

<u>FUNCTIONAL UNIT</u>	<u>CHANNEL CHECK</u>	<u>CHANNEL CALIBRATION</u>	<u>CHANNEL FUNCTIONAL TEST</u>	<u>MODES IN WHICH SURVEILLANCE REQUIRED</u>
13. Steam Generator Water Level--Low-Low	S	R	Q	1, 2
14. Deleted				
15. Undervoltage - Reactor Coolant Pumps	N.A.	R	Q	1
16. Underfrequency - Reactor Coolant Pumps	N.A.	R	Q	1
17. Turbine Trip A. Low Auto Stop Oil Pressure B. Turbine Throttle Valve Closure	N.A. N.A.	R R	S/U(9)(10) S/U(9)(10)	N.A. N.A.
18. Safety Injection Input from ESF	N.A.	N.A.	R(4)	1, 2
19. Reactor Coolant Pump Breaker Position Trip	N.A.	N.A.	R	1
20. Reactor Trip System Interlocks	N.A.	R	S/U(8)	1
21. Reactor Trip Breaker	N.A.	N.A.	M(5)(14)(15), S/U(1)(14)(15)	1, 2, 3*, 4*, 5*
22. Automatic Trip Logic	N.A.	N.A.	M(5)	1, 2, 3*, 4*, 5*
23. Reactor Trip Bypass Breaker	N.A.	N.A.	(13),R(11)	1, 2, 3*, 4*, 5*

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FARLEY-UNIT 1

3/4 3-13

Corrected by letter dated June 17, 1994

AMENDMENT NO.  
26, 67, 104, 107

TABLE 4.3-1 (Continued)

## REACTOR TRIP SYSTEM INSTRUMENTATION SURVEILLANCE REQUIREMENTS

FUNCTIONAL UNIT	CHANNEL CHECK	CHANNEL CALIBRATION	CHANNEL FUNCTIONAL TEST	MODES IN WHICH SURVEILLANCE REQUIRED
13. Steam Generator Water Level--Low-Low	S	R	Q	1, 2
14. Steam/Feedwater Flow Mismatch and Low Steam Generator Water Level*	S	R	Q	1, 2
15. Undervoltage - Reactor Coolant Pumps	N.A.	R	Q	1
16. Underfrequency - Reactor Coolant Pumps	N.A.	R	Q	1
17. Turbine Trip				
A. Low Auto Stop Oil Pressure	N.A.	R	S/U(9)(10)	N.A.
B. Turbine Throttle Valve Closure	N.A.	R	S/U(9)(10)	N.A.
18. Safety Injection Input from ESF	N.A.	N.A.	R(4)	1, 2
19. Reactor Coolant Pump Breaker Position Trip	N.A.	N.A.	R	1
20. Reactor Trip System Interlocks	N.A.	R	S/U(8)	1
21. Reactor Trip Breaker	N.A.	N.A.	M(5)(14)(15), S/U(1)(14)(15)	1, 2, 3*, 4*, 5*
22. Automatic Trip Logic	N.A.	N.A.	M(5)	1, 2, 3*, 4*, 5*
23. Reactor Trip Bypass Breaker	N.A.	N.A.	(13),R(11)	1, 2, 3*, 4*, 5*

\* Deletion of this functional trip shall be implemented prior to startup from refueling outage 12.

FARLEY-UNIT 2

3/4 3-13

Corrected by letter dated June 17, 1994

AMENDMENT NO.  
89, 97, 99