

Mr. D. N. Morey
Vice President - Farley Project
Southern Nuclear Operating
Company, Inc.
Post Office Box 1295
Birmingham, Alabama 35201-1295

May 10, 1999

SUBJECT: ISSUANCE OF AMENDMENTS - JOSEPH M. FARLEY NUCLEAR PLANT,
UNITS 1 AND 2 (TAC NOS. MA5119 AND MA5120)

Dear Mr. Morey:

The Nuclear Regulatory Commission (NRC) has issued the enclosed Amendment No. 142 to Facility Operating License No. NPF-2 and Amendment No. 134 to Facility Operating License No. NPF-8 for the Joseph M. Farley Nuclear Plant (Farley), Units 1 and 2. The amendments change the Technical Specifications (TS) in response to your submittal dated April 2, 1999.

The amendment changes TS 3/4.4.9, "Specific Activity," and the associated bases to increase the limit associated with dose equivalent iodine-131. The steady-state dose equivalent iodine-131 limit would be increased from 0.15 μ Curie/gram to 0.3 μ Curie/gram and the transient limit for 80 percent to 100 percent power provided by Technical Specification Figure 3.4-1 will increase from 9 μ Curie/gram to 18 μ Curie/gram with a corresponding increase in the 0 percent to 80 percent power limits.

A copy of the related Safety Evaluation is also enclosed. A Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

Original signed by:
Jacob I. Zimmerman, Project Manager, Section 1
Project Directorate II
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

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PDR ADOCK 05000348
P PDR

Docket Nos. 50-348 and 50-364

Enclosures:

1. Amendment No. 142 to NPF-2
2. Amendment No. 134 to NPF-8
3. Safety Evaluation

cc w/encls: See next page

DISTRIBUTION: JZimmerman CHawes
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OFFICE	PDII-1/PM	PDII-1/LA	PSAB:BC*	EMCB/BC*	OGC*	PDII-1/SC	PDII/D
NAME	J.Zimmerman:cn	CHawes	RBarrett	WBateman	MTobler	REmch	HBERKOW
DATE	5/14/99	5/14/99	4/13/99	4/15/99	1/ /99	5/5/99	5/10/99
COPY	<input checked="" type="checkbox"/> YES <input type="checkbox"/> NO	YES NO	YES NO	YES NO	YES NO	YES NO	YES <input checked="" type="checkbox"/> NO

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UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

May 10, 1999

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Vice President - Farley Project
Southern Nuclear Operating
Company, Inc.
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A copy of the related Safety Evaluation is also enclosed. A Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

A handwritten signature in cursive script, reading "Jacob I. Zimmerman", is written over a horizontal line.

Jacob I. Zimmerman, Project Manager, Section 1
Project Directorate II
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket Nos. 50-348 and 50-364

Enclosures:

1. Amendment No. 142 to NPF-2
2. Amendment No. 134 to NPF-8
3. Safety Evaluation

cc w/encls: See next page

Joseph M. Farley Nuclear Plant

cc:

Mr. L. M. Stinson
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Chairman
Houston County Commission
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Dothan, Alabama 36302

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U.S. Nuclear Regulatory Commission
7388 N. State Highway 95
Columbia, Alabama 36319



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SOUTHERN NUCLEAR OPERATING COMPANY, INC.

ALABAMA POWER COMPANY

DOCKET NO. 50-348

JOSEPH M. FARLEY NUCLEAR PLANT, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 142
License No. NPF-2

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Southern Nuclear Operating Company, Inc. (Southern Nuclear), dated April 2, 1999, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications, as indicated in the attachment to this license amendment; and paragraph 2.C.(2) of Facility Operating License No. NPF-2 is hereby amended to read as follows:

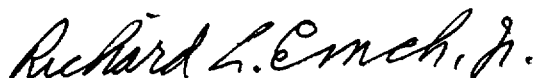
9905140028 990510
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P PDR

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 142 , are hereby incorporated in the license. Southern Nuclear shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Richard L. Emch, Jr., Chief, Section 1
Project Directorate II
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: May 10, 1999

ATTACHMENT TO LICENSE AMENDMENT NO. 142

TO FACILITY OPERATING LICENSE NO. NPF-2

DOCKET NO. 50-348

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages. The revised areas are indicated by marginal lines.

Remove Pages

3/4 4-23
3/4 4-24
3/4 4-25
3/4 4-26
B 3/4 4-5

Insert Pages

3/4 4-23
3/4 4-24
3/4 4-25
3/4 4-26
B 3/4 4-5

REACTOR COOLANT SYSTEM

3/4.4.9 SPECIFIC ACTIVITY

LIMITING CONDITION FOR OPERATION

3.4.9 The specific activity of the primary coolant shall be limited to:

- a. Less than or equal to 0.30 microCurie per gram DOSE EQUIVALENT I-131;
- b. Less than or equal to $100/\bar{E}$ microCurie per gram.

APPLICABILITY: MODES 1, 2, 3, 4, and 5

ACTION:

MODES 1, 2, and 3*:

- a. With the specific activity of the primary coolant greater than 0.30 microCurie per gram DOSE EQUIVALENT I-131 for more than 48 hours during one continuous time interval or exceeding the limit line shown on Figure 3.4-1, be in at least HOT STANDBY with T_{avg} less than 500°F within 6 hours.
- b. With the specific activity of the primary coolant greater than $100/\bar{E}$ microCurie per gram, be in at least HOT STANDBY with T_{avg} less than 500°F within 6 hours.

* With T_{avg} greater than or equal to 500°F.

REACTOR COOLANT SYSTEM

ACTION: (Continued)

MODES 1, 2, 3, 4, and 5:

- a. With the specific activity of the primary coolant greater than 0.30 microCurie per gram DOSE EQUIVALENT I-131 or greater than $100/\bar{E}$ microCuries per gram, perform the sampling and analysis requirements of item 4a of Table 4.4-4 until the specific activity of the primary coolant is restored to within its limits.

SURVEILLANCE REQUIREMENTS

4.4.9 The specific activity of the primary coolant shall be determined to be within the limits by performance of the sampling and analysis program of Table 4.4-4.

TABLE 4.4-4

PRIMARY COOLANT SPECIFIC ACTIVITY SAMPLE
AND ANALYSIS PROGRAM

<u>TYPE OF MEASUREMENT AND ANALYSIS</u>	<u>SAMPLE AND ANALYSIS FREQUENCY</u>	<u>MODES IN WHICH SAMPLE AND ANALYSIS REQUIRED</u>
1. Gross Activity Determination	At least once per 72 hours	1, 2, 3, 4
2. Isotopic Analysis for DOSE EQUIVALENT I-131 Concentration	1 per 14 days	1
3. Radiochemical for \bar{E} Determination	1 per 6 months*	1
4. Isotopic Analysis for Iodine Including I-131, I-133, and I-135	a) Once per 4 hours, whenever the specific activity exceeds 0.30 $\mu\text{Ci/gram DOSE EQUIVALENT}$ I-131 or $100/\bar{E}$ $\mu\text{Ci/gram}$, and b) One sample between 2 and 6 hours following a THERMAL POWER change exceeding 15 percent of the RATED THERMAL POWER within a one hour period.	1#, 2#, 3#, 4#, 5# 1, 2, 3

Until the specific activity of the primary coolant system is restored within its limits.

* Sample to be taken after a minimum of 2 EFPD and 20 days of POWER OPERATION have elapsed since reactor was last subcritical for 48 hours or longer.

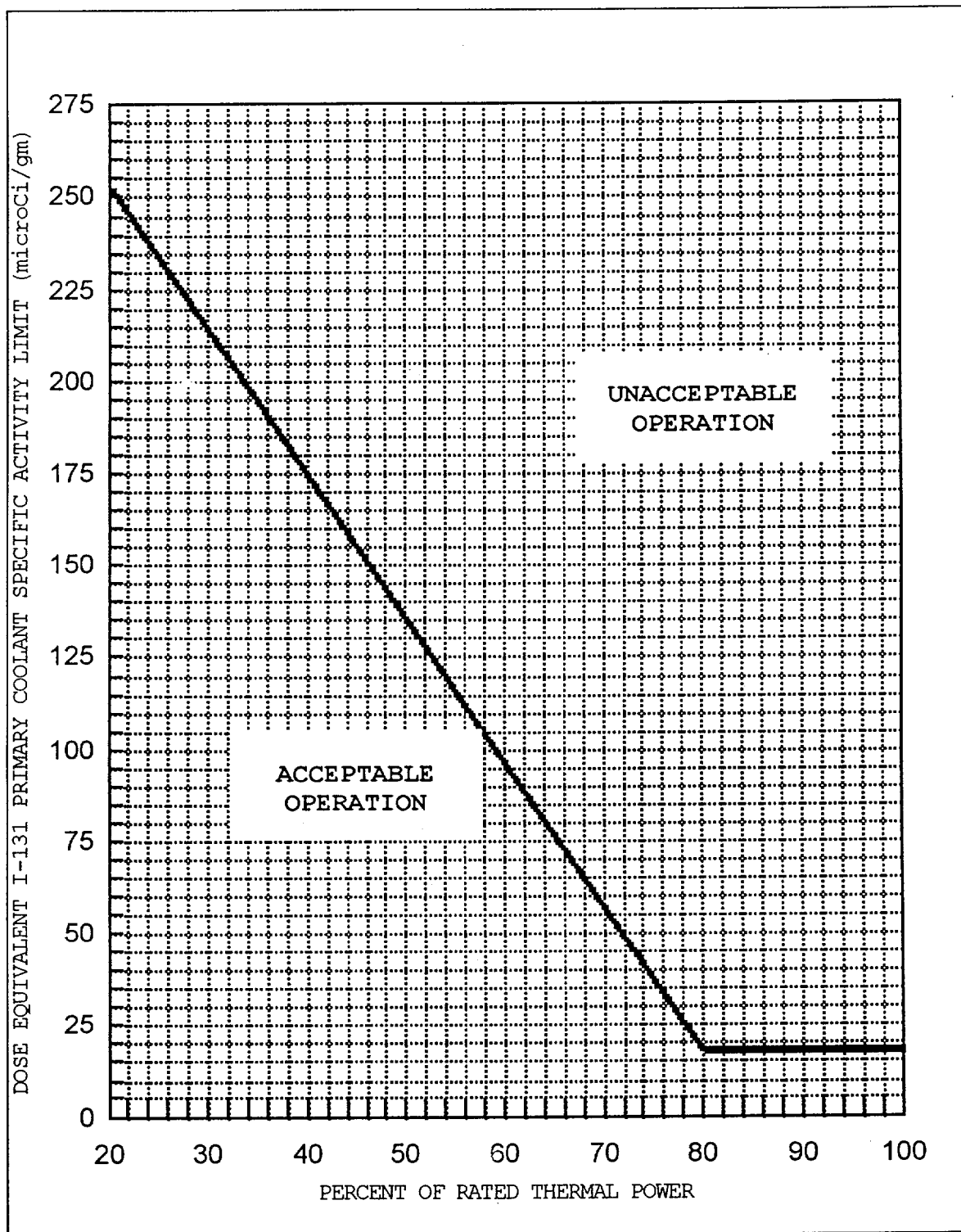


FIGURE 3.4-1

DOSE EQUIVALENT I-131 Primary Coolant Specific Activity Limit Versus Percent of RATED THERMAL POWER with the Primary Coolant Specific Activity > 0.30 μ Ci/gram Dose Equivalent I-131

REACTOR COOLANT SYSTEM

BASES

3/4.4.8 CHEMISTRY

The limitations on Reactor Coolant System chemistry ensure that corrosion of the Reactor Coolant System is minimized and reduces the potential for Reactor Coolant System leakage or failure due to stress corrosion. Maintaining the chemistry within the Steady State Limits provides adequate corrosion protection to ensure the structural integrity of the Reactor Coolant System over the life of the plant. The associated effects of exceeding the oxygen, chloride, and fluoride limits are time and temperature dependent. Corrosion studies show that operation may be continued with contaminant concentration levels in excess of the Steady State Limits, up to the Transient Limits, for the specified limited time intervals without having a significant effect on the structural integrity of the Reactor Coolant System. The time interval permitting continued operation within the restrictions of the Transient Limits provides time for taking corrective actions to restore the contaminant concentrations to within the Steady State Limits.

The surveillance requirements provide adequate assurance that concentrations in excess of the limits will be detected in sufficient time to take corrective action.

3/4.4.9 SPECIFIC ACTIVITY

The limitations on the specific activity of the primary coolant ensure that the resulting 2 hour doses at the site boundary will not exceed an appropriately small fraction of Part 100 limits in the event of primary-to-secondary leakage as a result of a steamline break.

The ACTION statement permitting POWER OPERATION to continue for limited time periods with the primary coolant's specific activity greater than 0.30 microCuries/gram DOSE EQUIVALENT I-131, but within the allowable limit shown on Figure 3.4-1, accommodates possible iodine spiking phenomenon which may occur following changes in THERMAL POWER.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SOUTHERN NUCLEAR OPERATING COMPANY, INC.

ALABAMA POWER COMPANY

DOCKET NO. 50-364

JOSEPH M. FARLEY NUCLEAR PLANT, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 134
License No. NPF-8

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Southern Nuclear Operating Company, Inc. (Southern Nuclear), dated April 2, 1999, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications, as indicated in the attachment to this license amendment; and paragraph 2.C.(2) of Facility Operating License No. NPF-8 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 134, are hereby incorporated in the license. Southern Nuclear shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Richard L. Emch, Jr., Chief, Section 1
Project Directorate II
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: May 10, 1999

ATTACHMENT TO LICENSE AMENDMENT NO. 134

TO FACILITY OPERATING LICENSE NO. NPF-8

DOCKET NO. 50-364

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages. The revised areas are indicated by marginal lines.

Remove

3/4 4-23
3/4 4-24
3/4 4-25
3/4 4-26
B 3/4 4-5

Insert

3/4 4-23
3/4 4-24
3/4 4-25
3/4 4-26
B 3/4 4-5

REACTOR COOLANT SYSTEM

3/4.4.9 . SPECIFIC ACTIVITY

LIMITING CONDITION FOR OPERATION

3.4.9 The specific activity of the primary coolant shall be limited to:

- a. Less than or equal to 0.30 microCurie per gram DOSE EQUIVALENT I-131;
- b. Less than or equal to $100/\bar{E}$ microCurie per gram.

APPLICABILITY: MODES 1, 2, 3, 4 and 5

ACTION:

MODES 1, 2 and 3*:

- a. With the specific activity of the primary coolant greater than 0.30 microCurie per gram DOSE EQUIVALENT I-131 for more than 48 hours during one continuous time interval or exceeding the limit shown on Figure 3.4-1, be in at least HOT STANDBY with T_{avg} less than 500°F within 6 hours.
- b. With the specific activity of the primary coolant greater than $100/\bar{E}$ microCurie per gram, be in at least HOT STANDBY with T_{avg} less than 500°F within 6 hours.

* With T_{avg} greater than or equal to 500°F.

REACTOR COOLANT SYSTEM

ACTION: (Continued)

MODES 1, 2, 3, 4 and 5:

- a. With the specific activity of the primary coolant greater than 0.30 microCurie per gram DOSE EQUIVALENT I-131 or greater than 100/E microCuries per gram, perform the sampling and analysis requirements of item 4a of Table 4.4-4 until the specific activity of the primary coolant is restored to within its limits.

SURVEILLANCE REQUIREMENTS

4.4.9 The specific activity of the primary coolant shall be determined to be within the limits by performance of the sampling and analysis program of Table 4.4-4.

PRIMARY COOLANT SPECIFIC ACTIVITY SAMPLE
AND ANALYSIS PROGRAM

[illegible]

Until the specific activity of the primary coolant system is restored within its limits.

* Sample to be taken after a minimum of 2 EFPD and 20 days of POWER OPERATION have elapsed since reactor was last subcritical for 48 hours or longer.

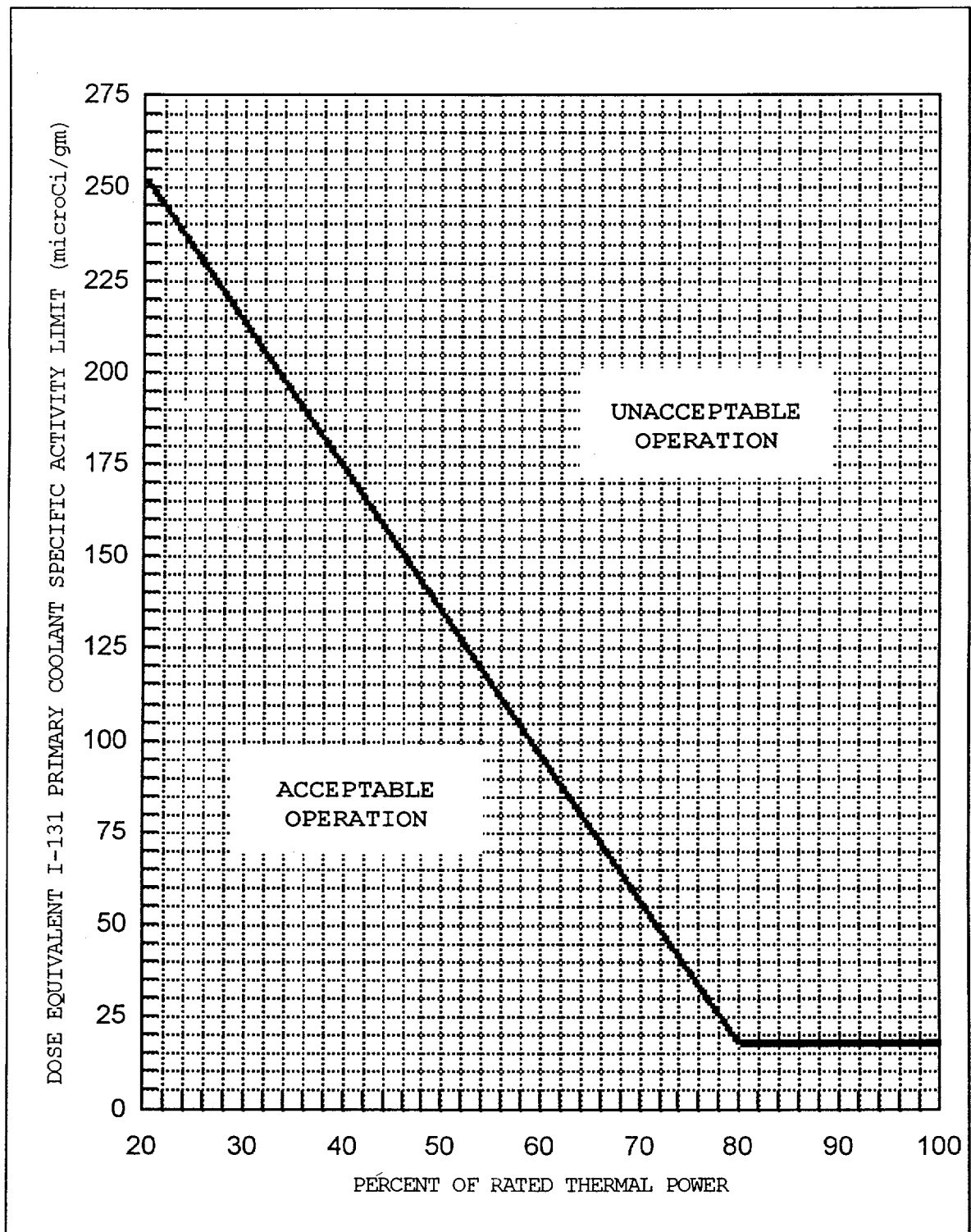


FIGURE 3.4-1

DOSE EQUIVALENT I-131 Primary Coolant Specific Activity Limit Versus
Percent of RATED THERMAL POWER with the Primary Coolant Specific
Activity > 0.30 $\mu\text{Ci}/\text{gram}$ Dose Equivalent I-131

REACTOR COOLANT SYSTEM

BASES

3/4.4.8 CHEMISTRY

The limitations on Reactor Coolant System chemistry ensure that corrosion of the Reactor Coolant System is minimized and reduces the potential for Reactor Coolant System leakage or failure due to stress corrosion. Maintaining the chemistry within the Steady State Limits provides adequate corrosion protection to ensure the structural integrity of the Reactor Coolant System over the life of the plant. The associated effects of exceeding the oxygen, chloride and fluoride limits are time and temperature dependent. Corrosion studies show that operation may be continued with containment concentration levels in excess of the Steady State Limits, up to the Transient Limits, for the specified limited time intervals without having a significant effect on the structural integrity of the Reactor Coolant System. The time interval permitting continued operation within the restrictions of the Transient Limits provides time for taking corrective actions to restore the containment concentrations to within the Steady State Limits.

The surveillance requirements provide adequate assurance that concentrations in excess of the limits will be detected in sufficient time to take corrective action.

3/4.4.9 SPECIFIC ACTIVITY

The limitations on the specific activity of the primary coolant ensure that the resulting 2 hour doses at the site boundary will not exceed an appropriately small fraction of Part 100 limits in the event of primary-to-secondary leakage as a result of a steam line break.

The ACTION statement permitting POWER OPERATION to continue for limited time periods with the primary coolant's specific activity greater than 0.30 microCuries/gram DOSE EQUIVALENT I-131, but within the allowable limit shown on Figure 3.4-1, accommodates possible iodine spiking phenomenon which may occur following changes in THERMAL POWER.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 142 TO FACILITY OPERATING LICENSE NO. NPF-2
AND AMENDMENT NO. 134 TO FACILITY OPERATING LICENSE NO. NPF-8
SOUTHERN NUCLEAR OPERATING COMPANY, INC., ET AL.
JOSEPH M. FARLEY NUCLEAR PLANT, UNITS 1 AND 2
DOCKET NOS. 50-348 AND 50-364

1.0 INTRODUCTION

By letter dated April 2, 1999, the Southern Nuclear Operating Company, Inc. (SNC), et al., submitted an amendment request to modify the Joseph M. Farley Nuclear Plant, Units 1 and 2, Technical Specifications (TSs). The requested changes would revise TS 3/4.4.9, "Specific Activity," to increase both the maximum instantaneous and the 48-hour values of dose equivalent ^{131}I (iodine-131) in the reactor coolant. SNC proposed to increase these values because of the determination of a decrease in the allowable primary-to-secondary leakage associated with a main steamline break (MSLB) accident. This decrease was from a previously approved value of 23.8 gallons per minute (gpm) for Unit 1 and Unit 2 to a value of 11.8 gpm for both.

2.0 BACKGROUND

In Amendment Nos. 132 and 124 to the Unit 1 and 2 TSs, respectively, the 48-hour specific activity level of dose equivalent ^{131}I was changed to 0.15 micro-curies/gram ($\mu\text{Ci/g}$) from the previously approved value of 0.3 $\mu\text{Ci/g}$. In addition, Table 4.4-4 was changed to require sampling of ^{131}I , ^{133}I , and ^{135}I every 4 hours when the specific activity level of primary coolant exceeds 0.15 $\mu\text{Ci/g}$. Figure 3.4-1 was changed to limit the maximum instantaneous activity level of dose equivalent ^{131}I at rated thermal power levels of 80% or greater to 9 $\mu\text{Ci/g}$. This was a decrease from the previous value of 18 $\mu\text{Ci/g}$. The Bases section of the TS was also changed to reflect the new value of 0.15 $\mu\text{Ci/g}$.

In a letter dated April 2, 1999, SNC submitted an amendment request that proposed to modify TS 3/4.4.9, "Specific Activity," for both Units 1 and 2. In this letter, SNC proposed to increase the 48-hour TS value of dose equivalent ^{131}I from 0.15 $\mu\text{Ci/g}$ to 0.3 $\mu\text{Ci/g}$ and the maximum instantaneous value in the 80%-100% power range from 9 $\mu\text{Ci/g}$ to 18 $\mu\text{Ci/g}$. The maximum instantaneous values for power levels less than 80% would also be increased in Figure 3/4-1. Associated with these increases in allowable values of dose equivalent ^{131}I in primary coolant was a proposed decrease in the allowable primary-to-secondary leakage rate to 11.8 gpm.

Radiological Consequences

The proposed increase in the 48-hour TECHNICAL SPECIFICATION value of dose equivalent ^{131}I in primary coolant from $0.15 \mu\text{Ci/g}$ to $0.3 \mu\text{Ci/g}$ and the proposed increase in the maximum instantaneous value for dose equivalent ^{131}I from $9 \mu\text{Ci/g}$ to $18 \mu\text{Ci/g}$ in concert with the decrease in MSLB accident induced leakage from 23.8 gpm to 11.8 gpm would result in relatively little change to the quantities of radioactivity released in the event of a MSLB. Consequently, the staff concluded that there would be little if any change in doses from those presented in the power uprate amendments, Amendment No. 137 for Unit 1 and Amendment No. 129 for Unit 2. Therefore, the staff did not recalculate the doses for this amendment request and concluded that the previously calculated doses from the power uprate amendment are still relevant. Therefore, the proposed change to allow a reduced total primary-to-secondary leakage rate of 11.8 gpm for an MSLB and an increase in the TS allowable values for the maximum instantaneous dose equivalent ^{131}I and the 48-hour value for dose equivalent ^{131}I is acceptable.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the State of Alabama official was notified of the proposed issuance of the amendments. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

The amendments change a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and change surveillance requirements. The NRC staff has determined that the amendments involve no significant increase in the amounts and no significant change in the types of any effluents that may be released offsite and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration, and there has been no public comment on such finding (64 FR 17201, dated April 8, 1999). Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: J. Zimmerman
J. Hayes

Date: May 10, 1999