

September 20, 1992

Docket Nos. 50-348  
and 50-364

Mr. D. N. Morey, Vice President  
Southern Nuclear Operating Company, Inc.  
Post Office Box 1295  
Birmingham, Alabama 35201-1295

Dear Mr. Morey:

SUBJECT: ISSUANCE OF AMENDMENT NO. 99 TO FACILITY OPERATING LICENSE NO.  
NPF-2 AND AMENDMENT NO. 91 TO FACILITY OPERATING LICENSE NO.  
NPF-8 REGARDING RADIOLOGICAL EFFLUENT TECHNICAL SPECIFICATION -  
JOSEPH M. FARLEY NUCLEAR PLANT, UNITS 1 AND 2 (TAC NOS. M84009  
AND M84010)

The Nuclear Regulatory Commission has issued the enclosed Amendment No. 99 to Facility Operating License No. NPF-2 and Amendment No. 91 to Facility Operating License No. NPF-8 for the Joseph M. Farley Nuclear Plant, Units 1 and 2. The amendments change the Technical Specifications in response to your submittal dated June 23, 1992, as supplemented November 13, 1992.

The amendments change the Technical Specifications to relocate the procedural details of the radiological effluent technical specifications to the Offsite Dose Calculation Manual (ODCM) and relocates the procedural details for solid radioactive wastes to the Process Control Program (PCP). This request is in accordance with Generic Letter 89-01.

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's bi-weekly Federal Register notice.

Sincerely,  
ORIGINAL SIGNED BY:  
Timothy A. Reed, Senior Project Manager  
Project Directorate II-1  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 99 to NPF-2
2. Amendment No. 91 to NPF-8
3. Safety Evaluation

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\*See previous concurrence

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DATE	09/14/93	09/14/93	09/14/93	08/05/93	02/04/93

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AMENDMENT NO. 99 TO FACILITY OPERATING LICENSE NO. NPF-2 - FARLEY, UNIT 1  
AMENDMENT NO. 91 TO FACILITY OPERATING LICENSE NO. NPF-8 - FARLEY, UNIT 2

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

September 20, 1993

Docket Nos. 50-348  
and 50-364

Mr. D. N. Morey, Vice President  
Southern Nuclear Operating Company, Inc.  
Post Office Box 1295  
Birmingham, Alabama 35201-1295

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SUBJECT: ISSUANCE OF AMENDMENT NO. 99 TO FACILITY OPERATING LICENSE NO. NPF-2  
AND AMENDMENT NO. 91 TO FACILITY OPERATING LICENSE NO. NPF-8  
REGARDING RADIOLOGICAL EFFLUENT TECHNICAL SPECIFICATION -  
JOSEPH M. FARLEY NUCLEAR PLANT, UNITS 1 AND 2 (TAC NOS. M84009  
AND M84010)

The Nuclear Regulatory Commission has issued the enclosed Amendment No. 99 to Facility Operating License No. NPF-2 and Amendment No. 91 to Facility Operating License No. NPF-8 for the Joseph M. Farley Nuclear Plant, Units 1 and 2. The amendments change the Technical Specifications in response to your submittal dated June 23, 1992, as supplemented November 13, 1992.

The amendments change the Technical Specifications to relocate the procedural details of the radiological effluent technical specifications to the Offsite Dose Calculation Manual (ODCM) and relocates the procedural details for solid radioactive wastes to the Process Control Program (PCP). This request is in accordance with Generic Letter 89-01.

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's bi-weekly Federal Register notice.

Sincerely,

A handwritten signature in dark ink, appearing to read "Timothy A. Reed", is written over the typed name.

Timothy A. Reed, Senior Project Manager  
Project Directorate II-1  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 99 to NPF-2
2. Amendment No. 91 to NPF-8
3. Safety Evaluation

cc w/enclosures:  
See next page

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Mr. D. N. Morey  
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Company, Inc.

Joseph M. Farley Nuclear Plant

cc:

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SOUTHERN NUCLEAR OPERATING COMPANY, INC.

DOCKET NO. 50-348

JOSEPH M. FARLEY NUCLEAR PLANT, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 99  
License No. NPF-2

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Southern Nuclear Operating Company, Inc. (Southern Nuclear), dated June 23, 1992, as supplemented November 13, 1992, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications, as indicated in the attachment to this license amendment; and paragraph 2.C.(2) of Facility Operating License No. NPF-2 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 99, and the Environmental Protection Plan contained in Appendix B are hereby incorporated in the license. Southern Nuclear shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance and shall be implemented by December 31, 1993.

FOR THE NUCLEAR REGULATORY COMMISSION



S. Singh Bajwa, Acting Director  
Project Directorate II-1  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: September 20, 1993

ATTACHMENT TO LICENSE AMENDMENT NO. 99

TO FACILITY OPERATING LICENSE NO. NPF-2

DOCKET NO. 50-348

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages. The revised areas are indicated by marginal lines.

Remove Pages

Insert Pages

I	I
IV	IV
XI	XI
XVI	XVI
XIX	XIX
XX	XX
1-3	1-3
1-4	1-4
1-5	1-5
1-6	1-6
3/4 3-61	3/4 3-61
3/4 3-62	3/4 3-62
3/4 3-63	--
3/4 3-64	--
3/4 3-65	--
3/4 3-66	3/4 3-66
3/4 3-67	3/4 3-67
3/4 3-68	3/4 3-68
3/4 3-69	3/4 3-69
3/4 3-70	3/4 3-70
3/4 3-71	3/4 3-71
3/4 11-1	3/4 11-1
3/4 11-2	3/4 11-2
3/4 11-3	--
3/4 11-4	--
3/4 11-5	3/4 11-5
3/4 11-6	3/4 11-6
3/4 11-8	3/4 11-8
3/4 11-9	3/4 11-9
3/4 11-10	--
3/4 11-11	--
3/4 11-12	3/4 11-12
3/4 11-13	3/4 11-13
3/4 11-14	3/4 11-14
3/4 11-15	3/4 11-15
3/4 11-17	3/4 11-17
3/4 11-18	3/4 11-18
3/4 11-19	3/4 11-19
3/4 12-1	3/4 12-1
3/4 12-2	3/4 12-2
3/4 12-3	--
3/4 12-4	--
3/4 12-5	--
3/4 12-6	--
3/4 12-7	--

Continued:

ATTACHMENT TO LICENSE AMENDMENT NO. 99  
TO FACILITY OPERATING LICENSE NO. NPF-8

DOCKET NO. 50-348

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages. The revised areas are indicated by marginal lines.

Remove Pages

Insert Pages

3/4 12-8  
3/4 12-9  
3/4 12-10  
3/4 12-11  
B 3/4 3-5  
B 3/4 11-1  
B 3/4 11-2  
B 3/4 11-3  
B 3/4 11-4  
B 3/4 11-5  
B 3/4 12-1  
B 3/4 12-2  
6-15  
--  
--  
--  
6-17  
6-18  
6-19  
6-21  
6-21a  
6-23  
6-24

- -  
- -  
3/4 12-10  
3/4 12-11  
B 3/4 3-5  
B 3/4 11-1  
B 3/4 11-2  
B 3/4 11-3  
B 3/4 11-4  
B 3/4 11-5  
B 3/4 12-1  
B 3/4 12-2  
6-15  
6-15a  
6-15b  
6-15c  
6-17  
6-18  
6-19  
6-21  
6-21a  
6-23  
6-24



## INDEX

### DEFINITIONS

-----

<u>SECTION</u>	<u>PAGE</u>
<u>1.0 DEFINITIONS</u>	
1.1 ACTION .....	1-1
1.2 AXIAL FLUX DIFFERENCE .....	1-1
1.3 CHANNEL CALIBRATION.....	1-1
1.4 CHANNEL CHECK .....	1-1
1.5 CHANNEL FUNCTION TEST .....	1-1
1.6 CONTAINMENT INTEGRITY .....	1-2
1.7 CONTROLLED LEAKAGE .....	1-2
1.8 CORE ALTERATION .....	1-2
1.9 DOSE EQUIVALENT 1-131 .....	1-2
1.10 E-AVERAGE DISINTEGRATION ENERGY .....	1-3
1.11 ENGINEERED SAFETY FEATURES RESPONSE TIME .....	1-3
1.12 FREQUENCY NOTATION .....	1-3
1.13 GASEOUS-RADWASTE-TREATMENT-SYSTEM (Deleted) .....	1-3
1.14 IDENTIFIED LEAKAGE .....	1-3
1.15 LIQUID-RADWASTE-TREATMENT-SYSTEM (Deleted) .....	1-4
1.16 MAJOR-CHANGES-TO-RADIOACTIVE-WASTE-TREATMENT-SYSTEMS (Deleted) .	1-4
1.17 OFFSITE DOSE CALCULATION MANUAL (ODCM) .....	1-4
1.18 OPERABLE - OPERABILITY .....	1-4
1.19 OPERATIONAL MODE - MODE .....	1-5
1.20 PHYSICS TESTS .....	1-5
1.21 PRESSURE BOUNDARY LEAKAGE .....	1-5
1.22 PROCESS CONTROL PROGRAM (PCP) .....	1-5
1.23 PURGE - PURGING .....	1-5
1.24 QUADRANT POWER TILT RATIO .....	1-5
1.25 RATED THERMAL POWER .....	1-6
1.26 REACTOR TRIP SYSTEM RESPONSE TIME .....	1-6
1.27 REPORTABLE EVENT .....	1-6
1.28 SHUTDOWN MARGIN .....	1-6
1.29 SOLIDIFICATION (Deleted) .....	1-6
1.30 SOURCE CHECK .....	1-6
1.31 STAGGERED TEST BASIS .....	1-6
1.32 THERMAL POWER .....	1-7
1.33 UNIDENTIFIED LEAKAGE .....	1-7
1.34 VENTILATION EXHAUST TREATMENT SYSTEM .....	1-7
1.35 VENTING .....	1-7
TABLE 1.1 OPERATIONAL MODES .....	1-8
TABLE 1.2 FREQUENCY NOTATION .....	1-9

## INDEX

### LIMITING CONDITIONS FOR OPERATION AND SURVEILLANCE REQUIREMENTS

<u>SECTION</u>	<u>PAGE</u>
<u>3/4.2 POWER DISTRIBUTION LIMITS</u>	
3/4.2.1 AXIAL FLUX DIFFERENCE .....	3/4 2-1
3/4.2.2 HEAT FLUX HOT CHANNEL FACTOR .....	3/4 2-4
3/4.2.3 NUCLEAR ENTHALPY HOT CHANNEL FACTOR .....	3/4 2-8
3/4.2.4 QUADRANT POWER TILT RATIO .....	3/4 2-11
3/4.2.5 DNB PARAMETERS .....	3/4 2-14
<u>3/4.3 INSTRUMENTATION</u>	
3/4.3.1 REACTOR TRIP SYSTEM INSTRUMENTATION .....	3/4 3-1
3/4.3.2 ENGINEERED SAFETY FEATURE ACTUATION SYSTEM INSTRUMENTATION .....	3/4 3-15
3/4.3.3 MONITORING INSTRUMENTATION	
Radiation Monitoring .....	3/4 3-38
Movable Incore Detectors .....	3/4 3-42
Seismic Monitoring Instrumentation .....	3/4 3-43
Meteorological Instrumentation .....	3/4 3-46
Remote Shutdown Instrumentation .....	3/4 3-49
Chlorine Detection Systems .....	3/4 3-52
High Energy Line Break Sensors .....	3/4 3-53
Accident Monitoring Instrumentation .....	3/4 3-56
<del>Fire Detection-Instrumentation</del> (Deleted) .....	3/4 3-59
<del>Radioactive-Liquid-Effluent-Monitoring</del> (Deleted) .....	3/4 3-61
Waste Gas Monitoring .....	3/4 3-66
3/4.3.4 TURBINE OVERSPEED PROTECTION .....	3/4 3-72

## INDEX

### LIMITING CONDITIONS FOR OPERATION AND SURVEILLANCE REQUIREMENTS

<u>SECTION</u>	<u>PAGE</u>
<u>3/4.11 RADIOACTIVE EFFLUENTS</u>	
3/4.11.1 LIQUID EFFLUENTS	
Concentration (Deleted) .....	3/4 11-1
Dose (Deleted) .....	3/4 11-5
Liquid-Waste-Treatment (Deleted) .....	3/4 11-6
Liquid Holdup Tanks .....	3/4 11-7
3/4.11.2 GASEOUS EFFLUENTS	
Dose-Rate (Deleted) .....	3/4 11-8
Dose-Noble-Gases (Deleted) .....	3/4 11-12
Dose-Radioiodines, -Particulate, -and- Radionuclides-Other-than-Noble-Gases (Deleted).....	3/4 11-13
Gaseous-Radwaste-Treatment (Deleted) .....	3/4 11-14
Waste Gas Monitoring .....	3/4 11-15
Gas Storage Tanks .....	3/4 11-16
3/4.11.3 RADWASTE-SOLIDIFICATION (Deleted) .....	3/4 11-17
3/4.11.4 TOTAL-DOSE (Deleted) .....	3/4 11-19
<u>3/4.12 RADIOLOGICAL ENVIRONMENTAL MONITORING</u>	
3/4.12.1 MONITORING-PROGRAM (Deleted) .....	3/4 12-1
3/4.12.2 LAND-USE-CENSUS (Deleted) .....	3/4 12-10
3/4.12.3 INTERLABORATORY-COMPARISON (Deleted) .....	3/4 12-11

## INDEX

### BASES

-----

#### SECTION

#### PAGE

#### 3/4.11 RADIOACTIVE EFFLUENTS

3/4.11.1 LIQUID EFFLUENTS .....	B 3/4 11-1
3/4.11.2 GASEOUS EFFLUENTS .....	B 3/4 11-2
3/4.11.3 RADWASTE-SOLIDIFICATION (Deleted) .....	B 3/4 11-5
3/4.11.4 TOTAL-DOSE (Deleted) .....	B 3/4 11-5

#### 3/4.12 RADIOLOGICAL ENVIRONMENTAL MONITORING

3/4.12.1 MONITORING-PROGRAM (Deleted) .....	B 3/4 12-1 nt
3/4.12.2 LAND-USE-CENSUS (Deleted) .....	B 3/4 12-1
3/4.12.3 INTERLABORATORY-COMPARISON-PROGRAM (Deleted) .....	B 3/4 12-2

## INDEX

### **ADMINISTRATIVE CONTROLS**

-----

<u>SECTION</u>	<u>PAGE</u>
Review .....	6-10
Audits .....	6-11
Authority .....	6-12
Records .....	6-12
6.5.3 TECHNICAL REVIEW AND CONTROL	
Activities .....	6-12
Records .....	6-13
<u>6.6 REPORTABLE EVENT ACTION</u> .....	6-14
<u>6.7 SAFETY LIMIT VIOLATION</u> .....	6-14
<u>6.8 PROCEDURES AND PROGRAMS</u> .....	6-14
<u>6.9 REPORTING REQUIREMENTS</u>	
6.9.1 ROUTINE REPORTS	
Startup Report .....	6-15c
Annual Report .....	6-16
Annual Radiological Environmental Operating Report .....	6-17
Semiannual Radioactive Effluent Release Report .....	6-17
Monthly Operating Report .....	6-19
Radial Peaking Factor Limit Report .....	6-19
Annual Diesel Generator Reliability Data Report .....	6-19
Annual Reactor Coolant System Specific Activity Report .....	6-20
Annual Sealed Source Leakage Report .....	6-20
6.9.2 SPECIAL REPORTS .....	6-20
<u>6.10 RECORD RETENTION</u> .....	6-20
<u>6.11 RADIATION PROTECTION PROGRAM</u> .....	6-21a
<u>6.12 HIGH RADIATION AREA</u> .....	6-22

## INDEX

### ADMINISTRATIVE CONTROLS

-----

<u>SECTION</u>	<u>PAGE</u>
<u>6.13 PROCESS CONTROL PROGRAM</u> .....	6-23
<u>6.14 OFFSITE DOSE CALCULATION MANUAL</u> .....	6-23
<u>6.15 MAJOR-CHANGES-TO-RADIOACTIVE</u> <u>WASTE-TREATMENT-SYSTEMS (Deleted)</u> .....	6-24
<u>6.16 ENVIRONMENTAL QUALIFICATION</u> .....	6-25

## DEFINITIONS

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### $\bar{E}$ - AVERAGE DISINTEGRATION ENERGY

1.10  $\bar{E}$  shall be the average sum (weighted in proportion to the concentration of each radionuclide in the reactor coolant at the time of sampling) of the average beta and gamma energies per disintegration (in Mev) for isotopes, other than iodines, with half lives greater than 15 minutes, making up at least 95% of the total non-iodine activity in the coolant.

### ENGINEERED SAFETY FEATURE RESPONSE TIME

1.11 The ENGINEERED SAFETY FEATURE RESPONSE TIME shall be that time interval from when the monitored parameter exceeds its ESF actuation setpoint at the channel sensor until the ESF equipment is capable of performing its safety function (i.e., the valves travel to their required positions, pump discharge pressures reach their required values, etc.). Times shall include diesel generator starting and sequence loading delays where applicable.

### FREQUENCY NOTATION

1.12 The FREQUENCY NOTATION specified for the performance of Surveillance Requirements shall correspond to the intervals defined in Table 1.2.

### GASEOUS RADWASTE TREATMENT SYSTEM

1.13 This definition deleted. Refer to the Offsite Dose Calculation Manual.

### IDENTIFIED LEAKAGE

1.14 IDENTIFIED LEAKAGE shall be:

- a. Leakage (except CONTROLLED LEAKAGE) into closed systems, such as pump seal or valve packing leaks that are captured and conducted to a sump or collecting tank, or
- b. Leakage into the containment atmosphere from sources that are both specifically located and known either not to interfere with the operation of leakage detection systems or not to be PRESSURE BOUNDARY LEAKAGE, or
- c. Reactor coolant system leakage through a steam generator to the secondary system.

## **DEFINITIONS**

-----

### **LIQUID RADWASTE TREATMENT SYSTEM**

1.15 This definition deleted. Refer to the Offsite Dose Calculation Manual.

### **MAJOR CHANGES TO RADIOACTIVE WASTE TREATMENT SYSTEMS**

1.16 This definition deleted. Refer to the Offsite Dose Calculation Manual and the Process Control Program.

### **OFFSITE DOSE CALCULATION MANUAL (ODCM)**

1.17 The OFFSITE DOSE CALCULATION MANUAL shall contain the methodology and parameters used in the calculation of offsite doses resulting from radioactive gaseous and liquid effluents, in the calculation of gaseous and liquid effluent monitoring alarm/trip setpoints, and in the conduct of the Radiological Environmental Monitoring Program. The ODCM shall also contain (1) the Radioactive Effluent Controls and Radiological Environmental Monitoring Programs required by Technical Specification 6.8.3 and (2) descriptions of the information that should be included in the Annual Radiological Environmental Surveillance and Semiannual Radioactive Effluent Release Reports required by Technical Specifications 6.9.1.6, 6.9.1.7, 6.9.1.8 and 6.9.1.9.

### **OPERABLE - OPERABILITY**

1.18 A system, subsystem, train, component or device shall be OPERABLE or have OPERABILITY when it is capable of performing its specified function(s), and when all necessary attendant instrumentation, controls, a normal and an emergency electrical power sources, cooling or seal water, lubrication or other auxiliary equipment that are required for the system, subsystem, train, component or device to perform its function(s) are also capable of performing their related support function(s).



## DEFINITIONS

-----

### OPERATIONAL MODE - MODE

1.19 An OPERATIONAL MODE (i.e., MODE) shall correspond to any one inclusive combination of core reactivity condition, power level and average reactor coolant temperature specified in Table 1.1.

### PHYSICS TESTS

1.20 PHYSICS TESTS shall be those tests performed to measure the fundamental nuclear characteristics of the reactor core and related instrumentation and 1) described in Chapter 14.0 of the FSAR, 2) authorized under the provisions of 10 CFR 50.59, or 3) otherwise approved by the Commission.

### PRESSURE BOUNDARY LEAKAGE

1.21 PRESSURE BOUNDARY LEAKAGE shall be leakage (except steam generator tube leakage) through a non-isolable fault in a Reactor Coolant System Component body, pipe wall or vessel wall.

### PROCESS CONTROL PROGRAM (PCP)

1.22 The PROCESS CONTROL PROGRAM (PCP) shall contain the current formulas, sampling, analyses, tests, and determinations to be made to ensure that processing and packaging of solid radioactive wastes based on demonstrated processing of actual or simulated wet solid wastes will be accomplished in such a way as to assure compliance with 10 CFR Parts 20, 61, and 71; State regulations; burial ground requirements; and other requirements governing the disposal of solid radioactive waste.

### PURGE - PURGING

1.23 PURGE or PURGING is the controlled process of discharging air or gas from a confinement to maintain temperature, pressure, humidity, concentration or other operating condition, in such a manner that replacement air or gas is required to purify the confinement.

### QUADRANT POWER TILT RATIO

1.24 QUADRANT POWER TILT RATIO shall be the ratio of the maximum upper excore detector calibrated output to the average of the upper excore detector calibrated outputs, or the ratio of the maximum lower excore detector calibrated output to the average of the lower excore detector calibrated outputs, whichever is greater. With one excore detector inoperable, the remaining three detectors shall be used for computing the average.

## DEFINITIONS

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### RATED THERMAL POWER

1.25 RATED THERMAL POWER shall be a total reactor core heat transfer rate to the reactor coolant of 2652 MWt.

### REACTOR TRIP SYSTEM RESPONSE TIME

1.26 The REACTOR TRIP SYSTEM RESPONSE TIME shall be the time interval from when the monitored parameter exceeds its trip setpoint at the channel sensor until loss of stationary gripper coil voltage.

### REPORTABLE EVENT

1.27 A REPORTABLE EVENT shall be any of those conditions specified in Section 50.73 to 10 CFR Part 50.

### SHUTDOWN MARGIN

1.28 SHUTDOWN MARGIN shall be the instantaneous amount of reactivity by which the reactor is subcritical or would be subcritical from its present condition assuming all full length rod cluster assemblies (shutdown and control) are fully inserted except for the single rod cluster assembly of highest reactivity worth which is assumed to be fully withdrawn.

### SOLIDIFICATION

1.29 This definition deleted. Refer to the Process Control Program.

### SOURCE CHECK

1.30 A SOURCE CHECK shall be the qualitative assessment of channel response when the channel sensor is exposed to a radioactive source.

### STAGGERED TEST BASIS

1.31 A STAGGERED TEST BASIS shall consist of:

- a. A test schedule for n systems, subsystems, trains or other designated components obtained by dividing the specified test interval into n equal subintervals,
- b. The testing of one system, subsystem, train or other designated component at the beginning of each subinterval.

**INSTRUMENTATION**

**RADIOACTIVE LIQUID EFFLUENT MONITORING INSTRUMENTATION**

This specification deleted. Refer to the Offsite Dose Calculation Manual.

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**3/4 3-63 through 3/4 3-65 Deleted.**

## INSTRUMENTATION

### WASTE GAS MONITORING INSTRUMENTATION

#### LIMITING CONDITION FOR OPERATION

=====

3.3.3.11 The waste gas monitoring instrumentation channels shown in Table 3.3-14 shall be OPERABLE with their alarm/trip setpoints set to ensure that the limits of Specification 3.11.2.5 are not exceeded.

APPLICABILITY: As shown in Table 3.3-14.

#### ACTION:

- a. With a waste gas monitoring instrumentation channel alarm/trip setpoint less conservative than required by the above Specification, declare the channel inoperable and take the ACTION shown in Table 3.3-14.
- b. With less than the minimum number of waste gas monitoring instrumentation channels OPERABLE, take the ACTION shown in Table 3.3-14. Restore the inoperable instrumentation to OPERABLE status within 30 days and, if unsuccessful prepare and submit a Special Report to the Commission pursuant to Specification 6.9.2 to explain why this inoperability was not corrected in a timely manner.
- c. The provisions of Specifications 3.0.3 and 3.0.4 are not applicable.

#### SURVEILLANCE REQUIREMENTS

=====

4.3.3.11 Each waste gas monitoring instrumentation channel shall be demonstrated OPERABLE by performance of the CHANNEL CHECK, CHANNEL CALIBRATION and CHANNEL FUNCTIONAL TEST operations at the frequencies shown in Table 4.3-9.

TABLE 3.3-14WASTE GAS MONITORING INSTRUMENTATION

<u>INSTRUMENT</u>	<u>MINIMUM CHANNELS OPERABLE</u>	<u>APPLICABILITY</u>	<u>ACTION</u>
1. WASTE GAS HOLDUP SYSTEM MONITORING INSTRUMENTATION			
a. Oxygen Monitors	2/recombiner	**	38
b. Hydrogen Monitors	1/recombiner	**	40

TABLE 3.3-14 (Continued)

TABLE NOTATION

## (Not Used)

\* (Not Used)

\*\* During recombiner operation.

ACTION 35 - (Not Used)

ACTION 36 - (Not Used)

ACTION 37 - (Not Used)

ACTION 38 - With the number of channels OPERABLE one less than required by the Minimum Channels OPERABLE requirement, except for testing, isolate the oxygen supply to the affected recombiner. With no channels OPERABLE, addition of waste gas to the waste gas system may continue provided a grab sample is taken from the on-service waste decay tank and analyzed once per 4 hours and the oxygen concentration remains less than 1%.

TABLE 3.3-14 (Continued)

TABLE NOTATION

ACTION 39 - (Not Used)

ACTION 40 - With no channels OPERABLE, addition of waste gas to the waste gas system may continue provided a grab sample is taken from the on-service waste decay tank and analyzed once per 4 hours and the oxygen concentration remains less than 1%.



TABLE 4.3-9

WASTE GAS MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS

<u>INSTRUMENT</u>	<u>CHANNEL CHECK</u>	<u>SOURCE CHECK</u>	<u>CHANNEL CALIBRATION</u>	<u>CHANNEL FUNCTIONAL TEST</u>	<u>MODES IN WHICH SURVEILLANCE REQUIRED</u>
1. WASTE GAS HOLDUP SYSTEM MONITORING INSTRUMENTATION					
a. Inlet Hydrogen Monitor	D	N.A.	Q(3)	M	**
b. Outlet Hydrogen Monitor	D	N.A.	Q(3)	M	**
c. Inlet Oxygen Monitor	D	N.A.	Q(4)	M	**
d. Outlet Oxygen Monitor	D	N.A.	Q(4)	M	**

TABLE 4.3-9 (Continued)

TABLE NOTATION

- \* (Not Used)
- \*\* During recombiner operation.
- (1) (Not Used)
- (2) (Not Used)
- (3) The CHANNEL CALIBRATION shall include the use of standard gas samples containing in accordance with manufacturer's recommendations.
- (4) The CHANNEL CALIBRATION shall include the use of standard gas samples containing in accordance with manufacturer's recommendations. In addition, a standard gas sample of nominally four volume percent oxygen, balance nitrogen shall be used for inlet oxygen monitor linearity check.

3/4.11 RADIOACTIVE EFFLUENTS

3/4.11.1 LIQUID EFFLUENTS

CONCENTRATION

This specification deleted. Refer to the Offsite Dose Calculation Manual.

**This page intentionally left blank.**

**3/4 11-3 and 3/4 11-4 Deleted.**

**RADIOACTIVE EFFLUENTS**

**DOSE**

This specification deleted. Refer to the Offsite Dose Calculation Manual.

**RADIOACTIVE EFFLUENTS**

**LIQUID WASTE TREATMENT**

This specification deleted. Refer to the Offsite Dose Calculation Manual.

RADIOACTIVE EFFLUENTS

3/4.11.2 GASEOUS EFFLUENTS

DOSE RATE

This specification deleted. Refer to the Offsite Dose Calculation Manual.

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**3/4 11-10 and 3/4 11-11 Deleted.**



**RADIOACTIVE EFFLUENTS**

**DOSE - NOBLE GASES**

This specification deleted. Refer to the Offsite Dose Calculation Manual.

**RADIOACTIVE EFFLUENTS**

**DOSE - RADIOIODINES, RADIOACTIVE MATERIALS IN PARTICULATE FORM, AND RADIONUCLIDES  
OTHER THAN NOBLE GASES**

This specification deleted. Refer to the Offsite Dose Calculation Manual.

**RADIOACTIVE EFFLUENTS**

**GASEOUS RADWASTE TREATMENT**

This specification deleted. Refer to the Offsite Dose Calculation Manual.

## RADIOACTIVE EFFLUENTS

### WASTE GAS MONITORING

#### LIMITING CONDITION FOR OPERATION

-----

3.11.2.5 The concentration of oxygen in any portion of the gaseous radwaste treatment system shall be limited to less than or equal to 2% by volume whenever the hydrogen concentration in that portion of the gaseous radwaste treatment system exceeds 4% by volume.

APPLICABILITY: At all times.

#### ACTION:

- a. With the concentration of oxygen in the gaseous radwaste treatment system greater than 2% by volume but less than or equal to 4% by volume, reduce the concentration of oxygen to within the limit within 48 hours.
- b. With the concentration of oxygen in the gaseous radwaste treatment system greater than 4% by volume, immediately suspend all additions of waste gases to the system and reduce the concentration of oxygen to less than or equal to 4% within 1 hour and less than or equal to 2% by volume within the following 48 hours.
- c. The provisions of Specifications 3.0.3 and 3.0.4 are not applicable.

#### SURVEILLANCE REQUIREMENTS

-----

4.11.2.5 The concentration of hydrogen or oxygen in the gaseous radwaste treatment system shall be determined to be within the above limits by monitoring the waste gases in the gaseous radwaste treatment system with the hydrogen and/or oxygen monitors required OPERABLE by Table 3.3-14 of Specification 3.3.3.11.

**RADIOACTIVE EFFLUENTS**

**3/4.11.3 RADWASTE SOLIDIFICATION**

This specification deleted. Refer to the Process Control Program.

**This page intentionally left blank.**

**RADIOACTIVE EFFLUENTS**

**3/4.11.4 TOTAL DOSE**

This specification deleted. Refer to the Offsite Dose Calculation Manual.

### 3/4.12 RADIOLOGICAL ENVIRONMENTAL MONITORING

#### 3/4.12.1 MONITORING PROGRAM

This specification deleted. Refer to the Offsite Dose Calculation Manual.



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**3/4 12-3 through 3/4 12-9 Deleted.**

**RADIOLOGICAL ENVIRONMENTAL MONITORING**

**3/4.12.2 LAND USE CENSUS**

**This specification deleted. Refer to the Offsite Dose Calculation Manual.**

**RADIOLOGICAL ENVIRONMENTAL MONITORING**

**3/4.12.3 INTERLABORATORY COMPARISON PROGRAM**

This specification deleted. Refer to the Offsite Dose Calculation Manual.

## INSTRUMENTATION

### BASES

-----

#### 3/4.3.3.10 RADIOACTIVE LIQUID EFFLUENT INSTRUMENTATION

This specification deleted. Refer to the Offsite Dose Calculation Manual.

#### 3/4.3.3.11 WASTE GAS MONITORING INSTRUMENTATION

This instrumentation monitors (and controls) the concentrations of potentially explosive gas mixtures in the waste gas holdup system. The OPERABILITY and use of this instrumentation are consistent with the requirements of General Design Criteria 60 and 63 of Appendix A to 10 CFR Part 50.

#### 3/4.3.4 TURBINE OVERSPEED PROTECTION

This specification is provided to ensure that the turbine overspeed protection instrumentation and the turbine speed control valves are OPERABLE and will protect the turbine from excessive overspeed. Protection from turbine excessive overspeed is required since excessive overspeed of the turbine could generate potentially damaging missiles which could impact and damage safety related components, equipment or structures.

### 3/4.11 RADIOACTIVE EFFLUENTS

#### BASES

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#### 3/4.11.1 LIQUID EFFLUENTS

##### 3/4.11.1.1 CONCENTRATION

This specification deleted. Refer to the Offsite Dose Calculation Manual.

##### 3/4.11.1.2 DOSE

This specification deleted. Refer to the Offsite Dose Calculation Manual.

## RADIOACTIVE EFFLUENTS

### BASES

-----

#### 3/4.11.1.3 LIQUID WASTE TREATMENT

This specification deleted. Refer to the Offsite Dose Calculation Manual.

#### 3/4.11.1.4 LIQUID HOLDUP TANKS

Restricting the quantity of radioactive material contained in the specified tanks provides assurance that in the event of an uncontrolled release of the tanks' contents, the resulting concentrations would be less than the limits of 10 CFR Part 20, Appendix B, Table II, Column 2, at the nearest potable water supply and the nearest surface water supply in an unrestricted area.

#### 3/4.11.2 GASEOUS EFFLUENTS

##### 3/4.11.2.1 DOSE RATE

This specification deleted. Refer to the Offsite Dose Calculation Manual.

## RADIOACTIVE EFFLUENTS

### BASES

-----

#### 3/4.11.2.2 DOSE - NOBLE GASES

This specification deleted. Refer to the Offsite Dose Calculation Manual.

#### 3/4.11.2.3 DOSE - RADIOIODINES, RADIOACTIVE MATERIALS IN PARTICULATE FORM AND RADIONUCLIDES OTHER THAN NOBLE GASES

This specification deleted. Refer to the Offsite Dose Calculation Manual.

## RADIOACTIVE EFFLUENTS

### BASES

-----

#### 3/4.11.2.4 GASEOUS RADWASTE TREATMENT

This specification deleted. Refer to the Offsite Dose Calculation Manual.

#### 3/4.11.2.5 WASTE GAS MONITORING

This specification is provided to ensure that the concentration of potentially explosive gas mixtures contained in the waste gas holdup system is maintained below the flammability limits of hydrogen and oxygen. An automatic control feature is included in the system to prevent the oxygen concentration from reaching these flammability limits. The automatic control feature includes isolation of the source of oxygen, to reduce the concentration below the flammability limit. Maintaining the concentration of oxygen below the flammability limit provides assurance that the releases of radioactive materials will be controlled in conformance with the requirements of General Design Criterion 60 of Appendix A to 10 CFR Part 50.



## RADIOACTIVE EFFLUENTS

### BASES

-----

#### 3/4.11.2.6 GAS STORAGE TANKS

Restricting the quantity of radioactivity contained in each gas storage tank provides assurance that in the event of an uncontrolled release of the tanks' contents, the resulting total body exposure to an individual at the nearest exclusion area boundary will not exceed 0.5 rem. This is consistent with Standard Review Plan 15.7.1, "Waste Gas System Failure".

#### 3/4.11.3 RADWASTE SOLIDIFICATION

This specification deleted. Refer to the Process Control Program.

#### 3/4.11.4 TOTAL DOSE

This specification deleted. Refer to the Offsite Dose Calculation Manual.

### 3/4.12 RADIOLOGICAL ENVIRONMENTAL MONITORING

#### BASES

-----

#### 3/4.12.1 MONITORING PROGRAM

This specification deleted. Refer to the Offsite Dose Calculation Manual.

#### 3/4.12.2 LAND USE CENSUS

This specification deleted. Refer to the Offsite Dose Calculation Manual.

**RADIOLOGICAL ENVIRONMENTAL MONITORING**

**BASES**

-----

**3/4.12.3 INTERLABORATORY COMPARISON PROGRAM**

**This specification deleted. Refer to the Offsite Dose Calculation Manual.**

## **ADMINISTRATIVE CONTROLS**

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### **b. In-Plant Radiation Monitoring**

A program which will ensure the capability to accurately determine the airborne iodine concentration in certain plant areas where personnel may be present under accident conditions. This program shall include the following:

- (i) Training of personnel,
- (ii) Procedures for monitoring, and
- (iii) Provisions for maintenance of sampling and analyses equipment.

### **c. Secondary Water Chemistry**

A program for monitoring of secondary water chemistry to inhibit steam generator tube degradation. This program shall include:

- (i) Identification of a sampling schedule for the critical variables and the control points for these variables,
- (ii) Identification of the procedures used to measure the values of the critical variables,
- (iii) Identification of process sampling points, including monitoring the condenser hotwells for evidence of condenser in-leakage,
- (iv) Procedures for the recording and management of data,
- (v) Procedures defining corrective actions for off-control-point chemistry conditions, and
- (vi) A procedure identifying (a) the authority responsible for the interpretation of the data, and (b) the sequence and timing of administrative events required to initiate corrective action.

### **d. Post-accident Sampling**

A program which will ensure the capability to obtain and analyze reactor coolant, radioactive iodines and particulates in plant gaseous effluents, and containment atmosphere samples under accident conditions. The program shall include the training of personnel, the procedures for sampling and analysis and the provisions for maintenance of sampling and analysis equipment.

### **e. Radioactive Effluent Controls Program**

A program shall be provided conforming with 10 CFR 50.36a for the control of radioactive effluents and for maintaining the doses to members of the public from radioactive effluents as low as

## **ADMINISTRATIVE CONTROLS**

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reasonably achievable. The program (1) shall be contained in the ODCM, (2) shall be implemented by operating procedures, and (3) shall include remedial actions to be taken whenever the program limits are exceeded. The program shall include the following elements:

- i) Limitations on the operability of radioactive liquid and gaseous monitoring instrumentation including surveillance tests and setpoint determination in accordance with the methodology in the ODCM,
- ii) Limitations on the concentrations of radioactive material released in liquid effluents to unrestricted areas conforming to 10 CFR Part 20, Appendix B, Table II, Column 2,
- iii) Monitoring, sampling, and analysis of radioactive liquid and gaseous effluents in accordance with 10 CFR 20.106 and with the methodology and parameters in the ODCM,
- iv) Limitations on the annual and quarterly doses or dose commitment to a member of the public from radioactive materials in liquid effluents released from each unit to unrestricted areas conforming to Appendix I to 10 CFR Part 50,
- v) Determination of cumulative and projected dose contributions from radioactive effluents for the current calendar quarter and current calendar year in accordance with the methodology and parameters in the ODCM at least every 31 days,
- vi) Limitations on the operability and use of the liquid and gaseous effluent treatment systems to ensure that the appropriate portions of these systems are used to reduce releases of radioactivity when the projected doses in a 31-day period would exceed 2 percent of the guidelines for the annual dose or dose commitment conforming to Appendix I to 10 CFR Part 50,
- vii) Limitations on the dose rate resulting from radioactive material released in gaseous effluents to areas beyond the site boundary conforming to the doses associated with 10 CFR Part 20, Appendix B, Table II, Column 1,
- viii) Limitations on the annual and quarterly air doses resulting from noble gases released in gaseous effluents from each unit to areas beyond the site boundary conforming to Appendix I to 10 CFR Part 50,

## ADMINISTRATIVE CONTROLS

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- ix) Limitations on the annual and quarterly doses to a member of the public from Iodine-131, Iodine-133, tritium, and all radionuclides in particulate form with half-lives greater than 8 days in gaseous effluents released from each unit to areas beyond the site boundary conforming to Appendix I to 10 CFR Part 50, and
- x) Limitations on the annual dose or dose commitment to any member of the public due to releases of radioactivity and to radiation from uranium fuel cycle sources conforming to 40 CFR Part 190.

### f. Radiological Environmental Monitoring Program

A program shall be provided to monitor the radiation and radionuclides in the environs of the plant. The program shall provide (1) representative measurements of radioactivity in the highest potential exposure pathways, and (2) verification of the accuracy of the effluent monitoring program and modeling of environmental exposure pathways. The program shall (1) be contained in the ODCM, (2) conform to the guidance of Appendix I to 10 CFR Part 50, and (3) include the following:

- i) Monitoring, sampling, analysis, and reporting of radiation and radionuclides in the environment in accordance with the methodology and parameters in the ODCM,
- ii) A Land Use Census to ensure that changes in the use of areas at and beyond the site boundary are identified and that modifications to the monitoring program are made if required by the results of this census, and
- iii) Participation in a Interlaboratory Comparison Program to ensure that independent checks on the precision and accuracy of the measurements of radioactive materials in environmental sample matrices are performed as part of the quality assurance program for environmental monitoring.

### g. Solid Radioactive Wastes Control Program

The PROCESS CONTROL PROGRAM (PCP) shall contain the current formulas, sampling, analyses, test, and determinations to be made to ensure that processing and packaging of solid radioactive wastes based on demonstrated processing of actual or simulated wet solid wastes will be accomplished in such a way as to assure compliance with 10 CFR Parts 20, 61, and 71; State regulations; burial ground requirements; and other requirements governing the disposal of solid radioactive waste.

## **ADMINISTRATIVE CONTROLS**

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### **6.9 REPORTING REQUIREMENTS**

#### **ROUTINE REPORTS**

6.9.1 In addition to the applicable reporting requirements of Title 10, Code of Federal Regulations, the following reports shall be submitted to the Commission, pursuant to 10CFR50.4.

#### **STARTUP REPORT**

6.9.1.1 A summary report of plant startup and power escalation testing shall be submitted following (1) receipt of an operating license, (2) amendment to the license involving a planned increase in power level, (3) installation of fuel that has a different design or has been manufactured by a different fuel supplier, and (4) modifications that may have significantly altered the nuclear, thermal, or hydraulic performance of the plant.

## **ADMINISTRATIVE CONTROLS**

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### **ANNUAL RADIOLOGICAL ENVIRONMENTAL OPERATING REPORT**

6.9.1.6 and 6.9.1.7 The Annual Radiological Environmental Operating Report covering the operation of the unit during the previous calendar year shall be submitted before May 1 of each year. The report shall include summaries, interpretations, and analysis of trends of the results of the Radiological Environmental Monitoring Program for the reporting period. The material provided shall be consistent with the objectives outlined in (1) the ODCM and (2) Sections IV.B.2, IV.B.3, and IV.C of Appendix I to 10 CFR Part 50. A single submittal may be made for a multiple unit station.

### **SEMIANNUAL RADIOACTIVE EFFLUENT RELEASE REPORT**

6.9.1.8 and 6.9.1.9 The Semiannual Radioactive Effluent Release Report covering the operation of the unit during the previous 6 months of operation shall be submitted within 60 days after January 1 and July 1 of each year. The report shall include a summary of the quantities of radioactive liquid and gaseous effluents and solid waste released from the unit. The material provided shall be (1) consistent with the objectives outlined in the ODCM and PCP and (2) in conformance with 10 CFR 50.36a and Section IV.B.1 of Appendix I to 10 CFR Part 50. A single submittal may be made for a multiple unit station. The submittal should combine those sections that are common to all units at the station; however, for units with separate radwaste systems, the submittal shall specify the releases of radioactive material from each unit.



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## ADMINISTRATIVE CONTROLS

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### MONTHLY OPERATING REPORT

6.9.1.10 Routine reports of operating statistics and shutdown experience, including documentation of all challenges to the PORV's or safety valves, shall be submitted on a monthly basis to the Commission, pursuant to 10 CFR 50.4, no later than the 15th of each month following the calendar month covered by the report.

### RADIAL PEAKING FACTOR LIMIT REPORT

6.9.1.11 The  $F_{xy}$  limit for Rated Thermal Power ( $F_{xy}^{RTP}$ ) for all core planes containing bank "D" control rods and all unrodded core planes shall be established and documented in the Radial Peaking Factor Limit Report before each reload cycle (prior to MODE 2) and provided to the Commission, pursuant to 10 CFR 50.4, upon issuance. In the event that the limit would be submitted at some other time during core life, it will be submitted upon issuance, unless otherwise exempted by the Commission.

Any information needed to support  $F_{xy}^{RTP}$  will be by request from the NRC and need not be included in this report.

### ANNUAL DIESEL GENERATOR RELIABILITY DATA REPORT

6.9.1.12 The number of tests (valid or invalid) and the number of failures to start on demand for each diesel generator shall be submitted to the NRC annually. This report shall contain the information identified in Regulatory Position C.3.b of NRC Regulatory Guide 1.108, Revision 1, 1977.

## ADMINISTRATIVE CONTROLS

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- h. Records of annual physical inventory of all sealed source material of record.

6.10.2 The following records shall be retained for the duration of the Unit Operating License.

- a. Records and drawing changes reflecting unit design modifications made to systems and equipment described in the Final Safety Analysis Report.
- b. Records of new and irradiated fuel inventory, fuel transfers and assembly burnup histories.
- c. Records of radiation exposure for all individuals entering radiation control areas.
- d. Records of gaseous and liquid radioactive material released to the environs.
- e. Records of transient or operational cycles for those unit components identified in Table 5.7-1.
- f. Records of reactor tests and experiments.
- g. Records of training and qualification for current members of the facility staff.
- h. Records of in-service inspections performed pursuant to these Technical Specifications.
- i. Records of safety-related Quality Assurance activities required by the Operations Quality Assurance Policy Manual (OQAPM) and not specifically described in Technical Specification 6.10.1.
- j. Records of reviews performed for changes made to procedures or equipment or reviews of tests and experiments pursuant to 10CFR50.59.
- k. Records of meetings of the PORC and the NORB.
- l. Records of secondary water sampling and water quality.
- m. Records of analyses required by the radiological environmental monitoring program.
- n. Records of the service lives of all hydraulic and mechanical snubbers in service after 07-01-84 within the scope of 3/4.7.9 including the date at which the service life commences and associated installation and maintenance records.
- o. Records of reviews performed for changes made to the OFFSITE DOSE CALCULATION MANUAL and the PROCESS CONTROL PROGRAM.

## **ADMINISTRATIVE CONTROLS**

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### **6.11 RADIATION PROTECTION PROGRAM**

Procedures for personnel radiation protection shall be prepared consistent with the requirements of 10CFR Part 20 and shall be approved, maintained and adhered to for all operations involving personnel radiation exposure.

## **ADMINISTRATIVE CONTROLS**

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### **6.13 PROCESS CONTROL PROGRAM (PCP)**

- 6.13.1 The PCP shall be approved by the Commission prior to implementation.
- 6.13.2 Licensee initiated changes to the PCP:
1. Shall be documented and records of reviews performed shall be retained as required by Specification 6.10.2.o. This documentation shall contain:
    - a. Sufficient information to support the change together with the appropriate analyses or evaluations justifying the change(s) and
    - b. A determination that the change will maintain the overall conformance of the solidified waste product to existing requirements of Federal, State, or other applicable regulations.
  2. Shall become effective after review and acceptance by the PORC and the approval of the General Manager - Nuclear Plant.

### **6.14 OFFSITE DOSE CALCULATION MANUAL (ODCM)**

- 6.14.1 The ODCM shall be approved by the Commission prior to implementation.
- 6.14.2 Licensee initiated changes to the ODCM:
1. Shall be documented and records of reviews performed shall be retained as required by Specification 6.10.2.o. This documentation shall contain:
    - a. Sufficient information to support the change together with the appropriate analyses or evaluations justifying the change(s) and
    - b. A determination that the change will maintain the level of radioactive effluent control required by 10 CFR 20.106, 40 CFR Part 190, 10 CFR 50.36a, and Appendix I to 10 CFR Part 50 and not adversely impact the accuracy or reliability of effluent, dose, or setpoint calculations.
  2. Shall become effective after review and acceptance by the PORC and the approval of the General Manager - Nuclear Plant.
  3. Shall be submitted to the Commission in the form of a complete, legible copy of the entire ODCM as a part of or concurrent with the Semiannual Radioactive Effluent Release Report for the period of the report in which any change to the ODCM was made. Each change shall be identified by markings in the margin of the affected pages, clearly indicating the area of the page that was changed, and shall indicate the date (e.g., month/year) the change was implemented.

## ADMINISTRATIVE CONTROLS

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### 6.15 MAJOR CHANGES TO RADIOACTIVE WASTE TREATMENT SYSTEMS (Liquid, Gaseous, and Solid)

This specification deleted. Refer to the Offsite Dose Calculation Manual and the Process Control Program.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SOUTHERN NUCLEAR OPERATING COMPANY, INC.

DOCKET NO. 50-364

JOSEPH M. FARLEY NUCLEAR PLANT, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 91  
License No. NPF-8

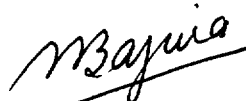
1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Southern Nuclear Operating Company, Inc. (Southern Nuclear), dated June 23, 1992, as supplemented November 13, 1992, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications, as indicated in the attachment to this license amendment; and paragraph 2.C.(2) of Facility Operating License No. NPF-2 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A , as revised through Amendment No. 91 , and the Environmental Protection Plan contained in Appendix B are hereby incorporated in the license. Southern Nuclear shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance and shall be implemented by December 31, 1993.

FOR THE NUCLEAR REGULATORY COMMISSION



S. Singh Bajwa, Acting Director  
Project Directorate II-1  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: September 20, 1993



ATTACHMENT TO LICENSE AMENDMENT NO. 91

TO FACILITY OPERATING LICENSE NO. NPF-8

DOCKET NO. 50-364

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages. The revised areas are indicated by marginal lines.

Remove Pages

I  
IV  
XI  
XVI  
XIX  
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1-3  
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3/4 3-61  
3/4 3-62  
3/4 3-63  
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3/4 12-1  
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Insert Pages

I  
IV  
XI  
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1-3  
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1-6  
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3/4 3-62  
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Continued:

ATTACHMENT TO LICENSE AMENDMENT NO. 91

TO FACILITY OPERATING LICENSE NO. NPF-8

DOCKET NO. 50-364

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages. The revised areas are indicated by marginal lines.

Remove Pages

Insert Pages

3/4 12-6	--
3/4 12-7	--
3/4 12-8	--
3/4 12-9	--
3/4 12-10	3/4 12-10
3/4 12-11	3/4 12-11
B 3/4 3-5	B 3/4 3-5
B 3/4 11-1	B 3/4 11-1
B 3/4 11-2	B 3/4 11-2
B 3/4 11-3	B 3/4 11-3
B 3/4 11-4	B 3/4 11-4
B 3/4 11-5	B 3/4 11-5
B 3/4 12-1	B 3/4 12-1
B 3/4 12-2	B 3/4 12-2
6-15	6-15
--	6-15a
--	6-15b
--	6-15c
6-17	6-17
6-18	6-18
6-19	6-19
6-21	6-21
--	6-21a
6-23	6-23
6-24	6-24

## INDEX

### DEFINITIONS

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#### SECTION

#### PAGE

#### 1.0 DEFINITIONS

1.1	ACTION .....	1-1
1.2	AXIAL FLUX DIFFERENCE .....	1-1
1.3	CHANNEL CALIBRATION.....	1-1
1.4	CHANNEL CHECK .....	1-1
1.5	CHANNEL FUNCTION TEST .....	1-1
1.6	CONTAINMENT INTEGRITY .....	1-2
1.7	CONTROLLED LEAKAGE .....	1-2
1.8	CORE ALTERATION .....	1-2
1.9	DOSE EQUIVALENT I-131 .....	1-2
1.10	E-AVERAGE DISINTEGRATION ENERGY .....	1-3
1.11	ENGINEERED SAFETY FEATURES RESPONSE TIME .....	1-3
1.12	FREQUENCY NOTATION .....	1-3
1.13	GASEOUS-RADWASTE-TREATMENT-SYSTEM (Deleted) .....	1-3
1.14	IDENTIFIED LEAKAGE .....	1-3
1.15	LIQUID-RADWASTE-TREATMENT-SYSTEM (Deleted) .....	1-4
1.16	MAJOR-CHANGES-TO-RADIOACTIVE-WASTE-TREATMENT-SYSTEMS (Deleted) .	1-4
1.17	OFFSITE DOSE CALCULATION MANUAL (ODCM) .....	1-4
1.18	OPERABLE - OPERABILITY .....	1-4
1.19	OPERATIONAL MODE - MODE .....	1-5
1.20	PHYSICS TESTS .....	1-5
1.21	PRESSURE BOUNDARY LEAKAGE .....	1-5
1.22	PROCESS CONTROL PROGRAM (PCP) .....	1-5
1.23	PURGE - PURGING .....	1-5
1.24	QUADRANT POWER TILT RATIO .....	1-5
1.25	RATED THERMAL POWER .....	1-6
1.26	REACTOR TRIP SYSTEM RESPONSE TIME .....	1-6
1.27	REPORTABLE EVENT .....	1-6
1.28	SHUTDOWN MARGIN .....	1-6
1.29	SOLIDIFICATION (Deleted) .....	1-6
1.30	SOURCE CHECK .....	1-6
1.31	STAGGERED TEST BASIS .....	1-6
1.32	THERMAL POWER .....	1-7
1.33	UNIDENTIFIED LEAKAGE .....	1-7
1.34	VENTILATION EXHAUST TREATMENT SYSTEM .....	1-7
1.35	VENTING .....	1-7
TABLE 1.1 OPERATIONAL MODES .....		1-8
TABLE 1.2 FREQUENCY NOTATION .....		1-9

## INDEX

### LIMITING CONDITIONS FOR OPERATION AND SURVEILLANCE REQUIREMENTS

-----

<u>SECTION</u>	<u>PAGE</u>
<u>3/4.2 POWER DISTRIBUTION LIMITS</u>	
3/4.2.1 AXIAL FLUX DIFFERENCE .....	3/4 2-1
3/4.2.2 HEAT FLUX HOT CHANNEL FACTOR .....	3/4 2-4
3/4.2.3 NUCLEAR ENTHALPY HOT CHANNEL FACTOR .....	3/4 2-8
3/4.2.4 QUADRANT POWER TILT RATIO .....	3/4 2-11
3/4.2.5 DNB PARAMETERS .....	3/4 2-14
<u>3/4.3 INSTRUMENTATION</u>	
3/4.3.1 REACTOR TRIP SYSTEM INSTRUMENTATION .....	3/4 3-1
3/4.3.2 ENGINEERED SAFETY FEATURE ACTUATION SYSTEM INSTRUMENTATION .....	3/4 3-15
3/4.3.3 MONITORING INSTRUMENTATION	
Radiation Monitoring .....	3/4 3-38
Movable Incore Detectors .....	3/4 3-42
Seismic Monitoring Instrumentation .....	3/4 3-43
Meteorological Instrumentation .....	3/4 3-46
Remote Shutdown Instrumentation .....	3/4 3-49
Chlorine Detection Systems .....	3/4 3-52
High Energy Line Break Sensors .....	3/4 3-53
Accident Monitoring Instrumentation .....	3/4 3-56
<del>Fire Detection Instrumentation</del> (Deleted) .....	3/4 3-59
<del>Radioactive-Liquid-Effluent-Monitoring</del> (Deleted) .....	3/4 3-61
Waste Gas Monitoring .....	3/4 3-66
3/4.3.4 TURBINE OVERSPEED PROTECTION .....	3/4 3-72

## INDEX

### BASES

-----

<u>SECTION</u>	<u>PAGE</u>
<u>3/4.11 RADIOACTIVE EFFLUENTS</u>	
3/4.11.1 LIQUID EFFLUENTS .....	B 3/4 11-1
3/4.11.2 GASEOUS EFFLUENTS .....	B 3/4 11-2
3/4.11.3 RADWASTE-SOLIDIFICATION (Deleted) .....	B 3/4 11-5
3/4.11.4 TOTAL-DOSE (Deleted) .....	B 3/4 11-5
<u>3/4.12 RADIOLOGICAL ENVIRONMENTAL MONITORING</u>	
3/4.12.1 MONITORING-PROGRAM (Deleted) .....	B 3/4 12-1
3/4.12.2 LAND-USE-CENSUS (Deleted) .....	B 3/4 12-1
3/4.12.3 INTERLABORATORY-COMPARISON-PROGRAM (Deleted) .....	B 3/4 12-2

## INDEX

### LIMITING CONDITIONS FOR OPERATION AND SURVEILLANCE REQUIREMENTS

---

#### SECTION

#### PAGE

#### 3/4.11 RADIOACTIVE EFFLUENTS

##### 3/4.11.1 LIQUID EFFLUENTS

Concentration (Deleted) .....	3/4 11-1
Dose (Deleted) .....	3/4 11-5
Liquid-Waste-Treatment (Deleted) .....	3/4 11-6
Liquid Holdup Tanks .....	3/4 11-7

##### 3/4.11.2 GASEOUS EFFLUENTS

Dose-Rate (Deleted) .....	3/4 11-8
Dose-Noble-Gases (Deleted) .....	3/4 11-12
Dose-Radioiodines, -Particulate, -and- Radionuclides-Other-than-Noble-Gases (Deleted) .....	3/4 11-13
Gaseous-Radwaste-Treatment (Deleted) .....	3/4 11-14
Waste Gas Monitoring .....	3/4 11-15
Gas Storage Tanks .....	3/4 11-16

##### 3/4.11.3 RADWASTE-SOLIDIFICATION (Deleted) .....

3/4 11-17

##### 3/4.11.4 TOTAL-DOSE (Deleted) .....

3/4 11-19

#### 3/4.12 RADIOLOGICAL ENVIRONMENTAL MONITORING

##### 3/4.12.1 MONITORING-PROGRAM (Deleted) .....

3/4 12-1

##### 3/4.12.2 LAND-USE-CENSUS (Deleted) .....

3/4 12-10

##### 3/4.12.3 INTERLABORATORY-COMPARISON (Deleted) .....

3/4 12-11

## INDEX

### ADMINISTRATIVE CONTROLS

-----

<u>SECTION</u>	<u>PAGE</u>
Review .....	6-10
Audits .....	6-11
Authority .....	6-12
Records .....	6-12
6.5.3 TECHNICAL REVIEW AND CONTROL	
Activities .....	6-12
Records .....	6-13
<u>6.6 REPORTABLE EVENT ACTION</u> .....	6-14
<u>6.7 SAFETY LIMIT VIOLATION</u> .....	6-14
<u>6.8 PROCEDURES AND PROGRAMS</u> .....	6-14
<u>6.9 REPORTING REQUIREMENTS</u>	
6.9.1 ROUTINE REPORTS	
Startup Report .....	6-15c
Annual Report .....	6-16
Annual Radiological Environmental Operating Report .....	6-17
Semiannual Radioactive Effluent Release Report .....	6-17
Monthly Operating Report .....	6-19
Radial Peaking Factor Limit Report .....	6-19
Annual Diesel Generator Reliability Data Report .....	6-19
Annual Reactor Coolant System Specific Activity Report .....	6-20
Annual Sealed Source Leakage Report .....	6-20
6.9.2 SPECIAL REPORTS .....	6-20
<u>6.10 RECORD RETENTION</u> .....	6-20
<u>6.11 RADIATION PROTECTION PROGRAM</u> .....	6-21a
<u>6.12 HIGH RADIATION AREA</u> .....	6-22

## INDEX

### ADMINISTRATIVE CONTROLS

-----

<u>SECTION</u>	<u>PAGE</u>
<u>6.13 PROCESS CONTROL PROGRAM</u> .....	6-23
<u>6.14 OFFSITE DOSE CALCULATION MANUAL</u> .....	6-23
<u>6.15 MAJOR-CHANGES-TO-RADIOACTIVE</u> <u>WASTE-TREATMENT-SYSTEMS</u> (Deleted) .....	6-24



## DEFINITIONS

-----

### $\bar{E}$ - AVERAGE DISINTEGRATION ENERGY

1.10  $\bar{E}$  shall be the average sum (weighted in proportion to the concentration of each radionuclide in the reactor coolant at the time of sampling) of the average beta and gamma energies per disintegration (in MeV) for isotopes, other than iodines, with half lives greater than 15 minutes, making up at least 95% of the total non-iodine activity in the coolant.

### ENGINEERED SAFETY FEATURE RESPONSE TIME

1.11 The ENGINEERED SAFETY FEATURE RESPONSE TIME shall be that time interval from when the monitored parameter exceeds its ESF actuation setpoint at the channel sensor until the ESF equipment is capable of performing its safety function (i.e., the valves travel to their required positions, pump discharge pressures reach their required values, etc.). Times shall include diesel generator starting and sequence loading delays where applicable.

### FREQUENCY NOTATION

1.12 The FREQUENCY NOTATION specified for the performance of Surveillance Requirements shall correspond to the intervals defined in Table 1.2.

### GASEOUS RADWASTE TREATMENT SYSTEM

1.13 This definition deleted. Refer to the Offsite Dose Calculation Manual.

### IDENTIFIED LEAKAGE

1.14 IDENTIFIED LEAKAGE shall be:

- a. Leakage (except CONTROLLED LEAKAGE) into closed systems, such as pump seal or valve packing leaks that are captured and conducted to a sump or collecting tank, or
- b. Leakage into the containment atmosphere from sources that are both specifically located and known either not to interfere with the operation of leakage detection systems or not to be PRESSURE BOUNDARY LEAKAGE, or
- c. Reactor coolant system leakage through a steam generator to the secondary system.

## DEFINITIONS

-----

### LIQUID RADWASTE TREATMENT SYSTEM

1.15 This definition deleted. Refer to the Offsite Dose Calculation Manual.

### MAJOR CHANGES TO RADIOACTIVE WASTE TREATMENT SYSTEMS

1.16 This definition deleted. Refer to the Offsite Dose Calculation Manual and the Process Control Program.

### OFFSITE DOSE CALCULATION MANUAL (ODCM)

1.17 The OFFSITE DOSE CALCULATION MANUAL shall contain the methodology and parameters used in the calculation of offsite doses resulting from radioactive gaseous and liquid effluents, in the calculation of gaseous and liquid effluent monitoring alarm/trip setpoints, and in the conduct of the Radiological Environmental Monitoring Program. The ODCM shall also contain (1) the Radioactive Effluent Controls and Radiological Environmental Monitoring Programs required by Technical Specification 6.8.3 and (2) descriptions of the information that should be included in the Annual Radiological Environmental Surveillance and Semiannual Radioactive Effluent Release Reports required by Technical Specifications 6.9.1.6, 6.9.1.7, 6.9.1.8 and 6.9.1.9.

### OPERABLE - OPERABILITY

1.18 A system, subsystem, train, component or device shall be OPERABLE or have OPERABILITY when it is capable of performing its specified function(s), and when all necessary attendant instrumentation, controls, a normal and an emergency electrical power sources, cooling or seal water, lubrication or other auxiliary equipment that are required for the system, subsystem, train, component or device to perform its function(s) are also capable of performing their related support function(s).

## DEFINITIONS

-----

### OPERATIONAL MODE - MODE

1.19 An OPERATIONAL MODE (i.e., MODE) shall correspond to any one inclusive combination of core reactivity condition, power level and average reactor coolant temperature specified in Table 1.1.

### PHYSICS TESTS

1.20 PHYSICS TESTS shall be those tests performed to measure the fundamental nuclear characteristics of the reactor core and related instrumentation and 1) described in Chapter 14.0 of the FSAR, 2) authorized under the provisions of 10 CFR 50.59, or 3) otherwise approved by the Commission.

### PRESSURE BOUNDARY LEAKAGE

1.21 PRESSURE BOUNDARY LEAKAGE shall be leakage (except steam generator tube leakage) through a non-isolable fault in a Reactor Coolant System Component body, pipe wall or vessel wall.

### PROCESS CONTROL PROGRAM (PCP)

1.22 The PROCESS CONTROL PROGRAM (PCP) shall contain the current formulas, sampling, analyses, tests, and determinations to be made to ensure that processing and packaging of solid radioactive wastes based on demonstrated processing of actual or simulated wet solid wastes will be accomplished in such a way as to assure compliance with 10 CFR Parts 20, 61, and 71; State regulations; burial ground requirements; and other requirements governing the disposal of solid radioactive waste.

### PURGE - PURGING

1.23 PURGE or PURGING is the controlled process of discharging air or gas from a confinement to maintain temperature, pressure, humidity, concentration or other operating condition, in such a manner that replacement air or gas is required to purify the confinement.

### QUADRANT POWER TILT RATIO

1.24 QUADRANT POWER TILT RATIO shall be the ratio of the maximum upper excore detector calibrated output to the average of the upper excore detector calibrated outputs, or the ratio of the maximum lower excore detector calibrated output to the average of the lower excore detector calibrated outputs, whichever is greater. With one excore detector inoperable, the remaining three detectors shall be used for computing the average.

## DEFINITIONS

-----

### RATED THERMAL POWER

1.25 RATED THERMAL POWER shall be a total reactor core heat transfer rate to the reactor coolant of 2652 MWt.

### REACTOR TRIP SYSTEM RESPONSE TIME

1.26 The REACTOR TRIP SYSTEM RESPONSE TIME shall be the time interval from when the monitored parameter exceeds its trip setpoint at the channel sensor until loss of stationary gripper coil voltage.

### REPORTABLE EVENT

1.27 A REPORTABLE EVENT shall be any of those conditions specified in Section 50.73 to 10 CFR Part 50.

### SHUTDOWN MARGIN

1.28 SHUTDOWN MARGIN shall be the instantaneous amount of reactivity by which the reactor is subcritical or would be subcritical from its present condition assuming all full length rod cluster assemblies (shutdown and control) are fully inserted except for the single rod cluster assembly of highest reactivity worth which is assumed to be fully withdrawn.

### SOLIDIFICATION

1.29 This definition deleted. Refer to the Process Control Program.

### SOURCE CHECK

1.30 A SOURCE CHECK shall be the qualitative assessment of channel response when the channel sensor is exposed to a radioactive source.

### STAGGERED TEST BASIS

1.31 A STAGGERED TEST BASIS shall consist of:

- a. A test schedule for n systems, subsystems, trains or other designated components obtained by dividing the specified test interval into n equal subintervals,
- b. The testing of one system, subsystem, train or other designated component at the beginning of each subinterval.

INSTRUMENTATION

RADIOACTIVE LIQUID EFFLUENT MONITORING INSTRUMENTATION

This specification deleted. Refer to the Offsite Dose Calculation Manual.

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**3/4 3-63 through 3/4 3-65 Deleted.**

## INSTRUMENTATION

### WASTE GAS MONITORING INSTRUMENTATION

#### LIMITING CONDITION FOR OPERATION

=====

3.3.3.11 The waste gas monitoring instrumentation channels shown in Table 3.3-14 shall be OPERABLE with their alarm/trip setpoints set to ensure that the limits of Specification 3.11.2.5 are not exceeded.

APPLICABILITY: As shown in Table 3.3-14.

#### ACTION:

- a. With a waste gas monitoring instrumentation channel alarm/trip setpoint less conservative than required by the above Specification, declare the channel inoperable and take the ACTION shown in Table 3.3-14.
- b. With less than the minimum number of waste gas monitoring instrumentation channels OPERABLE, take the ACTION shown in Table 3.3-14. Restore the inoperable instrumentation to OPERABLE status within 30 days and, if unsuccessful prepare and submit a Special Report to the Commission pursuant to Specification 6.9.2 to explain why this inoperability was not corrected in a timely manner.
- c. The provisions of Specifications 3.0.3 and 3.0.4 are not applicable.

#### SURVEILLANCE REQUIREMENTS

=====

4.3.3.11 Each waste gas monitoring instrumentation channel shall be demonstrated OPERABLE by performance of the CHANNEL CHECK, CHANNEL CALIBRATION and CHANNEL FUNCTIONAL TEST operations at the frequencies shown in Table 4.3-9.

TABLE 3.3-14WASTE GAS MONITORING INSTRUMENTATION

<u>INSTRUMENT</u>	<u>MINIMUM CHANNELS OPERABLE</u>	<u>APPLICABILITY</u>	<u>ACTION</u>
1. WASTE GAS HOLDUP SYSTEM MONITORING INSTRUMENTATION			
a. Oxygen Monitors	2/recombiner	**	38
b. Hydrogen Monitors	1/recombiner	**	40



TABLE 3.3-14 (Continued)

TABLE NOTATION

## (Not Used)

\* (Not Used)

\*\* During recombiner operation.

ACTION 35 - (Not Used)

ACTION 36 - (Not Used)

ACTION 37 - (Not Used)

ACTION 38 - With the number of channels OPERABLE one less than required by the Minimum Channels OPERABLE requirement, except for testing, isolate the oxygen supply to the affected recombiner. With no channels OPERABLE, addition of waste gas to the waste gas system may continue provided a grab sample is taken from the on-service waste decay tank and analyzed once per 4 hours and the oxygen concentration remains less than 1%.

TABLE 3.3-14 (Continued)

TABLE NOTATION

ACTION 39 - (Not Used)

ACTION 40 - With no channels OPERABLE, addition of waste gas to the waste gas system may continue provided a grab sample is taken from the on-service waste decay tank and analyzed once per 4 hours and the oxygen concentration remains less than 1%.

TABLE 4.3-9

WASTE GAS MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS

<u>INSTRUMENT</u>	<u>CHANNEL CHECK</u>	<u>SOURCE CHECK</u>	<u>CHANNEL CALIBRATION</u>	<u>CHANNEL FUNCTIONAL TEST</u>	<u>MODES IN WHICH SURVEILLANCE REQUIRED</u>
1. WASTE GAS HOLDUP SYSTEM MONITORING INSTRUMENTATION					
a. Inlet Hydrogen Monitor	D	N.A.	Q(3)	M	**
b. Outlet Hydrogen Monitor	D	N.A.	Q(3)	M	**
c. Inlet Oxygen Monitor	D	N.A.	Q(4)	M	**
d. Outlet Oxygen Monitor	D	N.A.	Q(4)	M	**

TABLE 4.3-9 (Continued)

TABLE NOTATION

- \* (Not Used)
- \*\* During recombiner operation.
- (1) (Not Used)
- (2) (Not Used)
- (3) The CHANNEL CALIBRATION shall include the use of standard gas samples containing in accordance with manufacturer's recommendations.
- (4) The CHANNEL CALIBRATION shall include the use of standard gas samples containing in accordance with manufacturer's recommendations. In addition, a standard gas sample of nominally four volume percent oxygen, balance nitrogen shall be used for inlet oxygen monitor linearity check.

### 3/4.11 RADIOACTIVE EFFLUENTS

#### 3/4.11.1 LIQUID EFFLUENTS

##### CONCENTRATION

This specification deleted. Refer to the Offsite Dose Calculation Manual.

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**3/4 11-3 and 3/4 11-4 Deleted.**

RADIOACTIVE EFFLUENTS

DOSE

This specification deleted. Refer to the Offsite Dose Calculation Manual.

RADIOACTIVE EFFLUENTS

LIQUID WASTE TREATMENT

This specification deleted. Refer to the Offsite Dose Calculation Manual.



RADIOACTIVE EFFLUENTS

3/4.11.2 GASEOUS EFFLUENTS

DOSE RATE

This specification deleted. Refer to the Offsite Dose Calculation Manual.

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**3/4 11-10 and 3/4 11-11 Deleted.**

RADIOACTIVE EFFLUENTS

DOSE - NOBLE GASES

This specification deleted. Refer to the Offsite Dose Calculation Manual.

RADIOACTIVE EFFLUENTS

DOSE - RADIOIODINES, RADIOACTIVE MATERIALS IN PARTICULATE FORM, AND RADIONUCLIDES  
OTHER THAN NOBLE GASES

This specification deleted. Refer to the Offsite Dose Calculation Manual.

RADIOACTIVE EFFLUENTS

GASEOUS RADWASTE TREATMENT

This specification deleted. Refer to the Offsite Dose Calculation Manual.

## RADIOACTIVE EFFLUENTS

### WASTE GAS MONITORING

#### LIMITING CONDITION FOR OPERATION

3.11.2.5 The concentration of oxygen in any portion of the gaseous radwaste treatment system shall be limited to less than or equal to 2% by volume whenever the hydrogen concentration in that portion of the gaseous radwaste treatment system exceeds 4% by volume.

APPLICABILITY: At all times.

#### ACTION:

- a. With the concentration of oxygen in the gaseous radwaste treatment system greater than 2% by volume but less than or equal to 4% by volume, reduce the concentration of oxygen to within the limit within 48 hours.
- b. With the concentration of oxygen in the gaseous radwaste treatment system greater than 4% by volume, immediately suspend all additions of waste gases to the system and reduce the concentration of oxygen to less than or equal to 4% within 1 hour and less than or equal to 2% by volume within the following 48 hours.
- c. The provisions of Specifications 3.0.3 and 3.0.4 are not applicable.

#### SURVEILLANCE REQUIREMENTS

4.11.2.5 The concentration of hydrogen or oxygen in the gaseous radwaste treatment system shall be determined to be within the above limits by monitoring the waste gases in the gaseous radwaste treatment system with the hydrogen and/or oxygen monitors required OPERABLE by Table 3.3-14 of Specification 3.3.3.11.

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**RADIOACTIVE EFFLUENTS**

**3/4.11.4 TOTAL DOSE**

This specification deleted. Refer to the Offsite Dose Calculation Manual.



3/4.12 RADIOLOGICAL ENVIRONMENTAL MONITORING

3/4.12.1 MONITORING PROGRAM

This specification deleted. Refer to the Offsite Dose Calculation Manual.

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**3/4 12-3 through 3/4 12-9 Deleted.**

**RADIOLOGICAL ENVIRONMENTAL MONITORING**

**3/4.12.2 LAND USE CENSUS**

**This specification deleted. Refer to the Offsite Dose Calculation Manual.**

RADIOLOGICAL ENVIRONMENTAL MONITORING

3/4.12.3 INTERLABORATORY COMPARISON PROGRAM

This specification deleted. Refer to the Offsite Dose Calculation Manual.

## INSTRUMENTATION

### BASES

-----

#### 3/4.3.3.10 RADIOACTIVE LIQUID EFFLUENT INSTRUMENTATION

This specification deleted. Refer to the Offsite Dose Calculation Manual.

#### 3/4.3.3.11 WASTE GAS MONITORING INSTRUMENTATION

This instrumentation monitors (and controls) the concentrations of potentially explosive gas mixtures in the waste gas holdup system. The OPERABILITY and use of this instrumentation are consistent with the requirements of General Design Criteria 60 and 63 of Appendix A to 10 CFR Part 50.

#### 3/4.3.4 TURBINE OVERSPEED PROTECTION

This specification is provided to ensure that the turbine overspeed protection instrumentation and the turbine speed control valves are OPERABLE and will protect the turbine from excessive overspeed. Protection from turbine excessive overspeed is required since excessive overspeed of the turbine could generate potentially damaging missiles which could impact and damage safety related components, equipment or structures.

### 3/4.11 RADIOACTIVE EFFLUENTS

#### BASES

-----

#### 3/4.11.1 LIQUID EFFLUENTS

##### 3/4.11.1.1 CONCENTRATION

This specification deleted. Refer to the Offsite Dose Calculation Manual.

##### 3/4.11.1.2 DOSE

This specification deleted. Refer to the Offsite Dose Calculation Manual.

## RADIOACTIVE EFFLUENTS

### BASES

-----

#### 3/4.11.1.3 LIQUID WASTE TREATMENT

This specification deleted. Refer to the Offsite Dose Calculation Manual.

#### 3/4.11.1.4 LIQUID HOLDUP TANKS

Restricting the quantity of radioactive material contained in the specified tanks provides assurance that in the event of an uncontrolled release of the tanks' contents, the resulting concentrations would be less than the limits of 10 CFR Part 20, Appendix B, Table II, Column 2, at the nearest potable water supply and the nearest surface water supply in an unrestricted area.

#### 3/4.11.2 GASEOUS EFFLUENTS

##### 3/4.11.2.1 DOSE RATE

This specification deleted. Refer to the Offsite Dose Calculation Manual.

## RADIOACTIVE EFFLUENTS

### BASES

=====

#### 3/4.11.2.2 DOSE - NOBLE GASES

This specification deleted. Refer to the Offsite Dose Calculation Manual.

#### 3/4.11.2.3 DOSE - RADIOIODINES, RADIOACTIVE MATERIALS IN PARTICULATE FORM AND RADIONUCLIDES OTHER THAN NOBLE GASES

This specification deleted. Refer to the Offsite Dose Calculation Manual.



## RADIOACTIVE EFFLUENTS

### BASES

-----

#### 3/4.11.2.4 GASEOUS RADWASTE TREATMENT

This specification deleted. Refer to the Offsite Dose Calculation Manual.

#### 3/4.11.2.5 WASTE GAS MONITORING

This specification is provided to ensure that the concentration of potentially explosive gas mixtures contained in the waste gas holdup system is maintained below the flammability limits of hydrogen and oxygen. An automatic control feature is included in the system to prevent the oxygen concentration from reaching these flammability limits. The automatic control feature includes isolation of the source of oxygen, to reduce the concentration below the flammability limit. Maintaining the concentration of oxygen below the flammability limit provides assurance that the releases of radioactive materials will be controlled in conformance with the requirements of General Design Criterion 60 of Appendix A to 10 CFR Part 50.

## RADIOACTIVE EFFLUENTS

### BASES

-----

#### 3/4.11.2.6 GAS STORAGE TANKS

Restricting the quantity of radioactivity contained in each gas storage tank provides assurance that in the event of an uncontrolled release of the tanks' contents, the resulting total body exposure to an individual at the nearest exclusion area boundary will not exceed 0.5 rem. This is consistent with Standard Review Plan 15.7.1, "Waste Gas System Failure".

#### 3/4.11.3 RADWASTE SOLIDIFICATION

This specification deleted. Refer to the Process Control Program.

#### 3/4.11.4 TOTAL DOSE

This specification deleted. Refer to the Offsite Dose Calculation Manual.

### 3/4.12 RADIOLOGICAL ENVIRONMENTAL MONITORING

#### BASES

-----

#### 3/4.12.1 MONITORING PROGRAM

This specification deleted. Refer to the Offsite Dose Calculation Manual.

#### 3/4.12.2 LAND USE CENSUS

This specification deleted. Refer to the Offsite Dose Calculation Manual.

RADIOLOGICAL ENVIRONMENTAL MONITORING

BASES  
-----

3/4.12.3 INTERLABORATORY COMPARISON PROGRAM

This specification deleted. Refer to the Offsite Dose Calculation Manual.

## ADMINISTRATIVE CONTROLS

---

### b. In-Plant Radiation Monitoring

A program which will ensure the capability to accurately determine the airborne iodine concentration in certain plant areas where personnel may be present under accident conditions. This program shall include the following:

- (i) Training of personnel,
- (ii) Procedures for monitoring, and
- (iii) Provisions for maintenance of sampling and analyses equipment.

### c. Secondary Water Chemistry

A program for monitoring of secondary water chemistry to inhibit steam generator tube degradation. This program shall include:

- (i) Identification of a sampling schedule for the critical variables and the control points for these variables,
- (ii) Identification of the procedures used to measure the values of the critical variables,
- (iii) Identification of process sampling points, including monitoring the condenser hotwells for evidence of condenser in-leakage,
- (iv) Procedures for the recording and management of data,
- (v) Procedures defining corrective actions for off-control-point chemistry conditions, and
- (vi) A procedure identifying (a) the authority responsible for the interpretation of the data, and (b) the sequence and timing of administrative events required to initiate corrective action.

### d. Post-accident Sampling

A program which will ensure the capability to obtain and analyze reactor coolant, radioactive iodines and particulates in plant gaseous effluents, and containment atmosphere samples under accident conditions. The program shall include the training of personnel, the procedures for sampling and analysis and the provisions for maintenance of sampling and analysis equipment.

### e. Radioactive Effluent Controls Program

A program shall be provided conforming with 10 CFR 50.36a for the control of radioactive effluents and for maintaining the doses to members of the public from radioactive effluents as low as

## ADMINISTRATIVE CONTROLS

---

reasonably achievable. The program (1) shall be contained in the ODCM, (2) shall be implemented by operating procedures, and (3) shall include remedial actions to be taken whenever the program limits are exceeded. The program shall include the following elements:

- i) Limitations on the operability of radioactive liquid and gaseous monitoring instrumentation including surveillance tests and setpoint determination in accordance with the methodology in the ODCM,
- ii) Limitations on the concentrations of radioactive material released in liquid effluents to unrestricted areas conforming to 10 CFR Part 20, Appendix B, Table II, Column 2,
- iii) Monitoring, sampling, and analysis of radioactive liquid and gaseous effluents in accordance with 10 CFR 20.106 and with the methodology and parameters in the ODCM,
- iv) Limitations on the annual and quarterly doses or dose commitment to a member of the public from radioactive materials in liquid effluents released from each unit to unrestricted areas conforming to Appendix I to 10 CFR Part 50,
- v) Determination of cumulative and projected dose contributions from radioactive effluents for the current calendar quarter and current calendar year in accordance with the methodology and parameters in the ODCM at least every 31 days,
- vi) Limitations on the operability and use of the liquid and gaseous effluent treatment systems to ensure that the appropriate portions of these systems are used to reduce releases of radioactivity when the projected doses in a 31-day period would exceed 2 percent of the guidelines for the annual dose or dose commitment conforming to Appendix I to 10 CFR Part 50,
- vii) Limitations on the dose rate resulting from radioactive material released in gaseous effluents to areas beyond the site boundary conforming to the doses associated with 10 CFR Part 20, Appendix B, Table II, Column 1,
- viii) Limitations on the annual and quarterly air doses resulting from noble gases released in gaseous effluents from each unit to areas beyond the site boundary conforming to Appendix I to 10 CFR Part 50,

## ADMINISTRATIVE CONTROLS

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- ix) Limitations on the annual and quarterly doses to a member of the public from Iodine-131, Iodine-133, tritium, and all radionuclides in particulate form with half-lives greater than 8 days in gaseous effluents released from each unit to areas beyond the site boundary conforming to Appendix I to 10 CFR Part 50, and
- x) Limitations on the annual dose or dose commitment to any member of the public due to releases of radioactivity and to radiation from uranium fuel cycle sources conforming to 40 CFR Part 190.

### f. Radiological Environmental Monitoring Program

A program shall be provided to monitor the radiation and radionuclides in the environs of the plant. The program shall provide (1) representative measurements of radioactivity in the highest potential exposure pathways, and (2) verification of the accuracy of the effluent monitoring program and modeling of environmental exposure pathways. The program shall (1) be contained in the ODCM, (2) conform to the guidance of Appendix I to 10 CFR Part 50, and (3) include the following:

- i) Monitoring, sampling, analysis, and reporting of radiation and radionuclides in the environment in accordance with the methodology and parameters in the ODCM,
- ii) A Land Use Census to ensure that changes in the use of areas at and beyond the site boundary are identified and that modifications to the monitoring program are made if required by the results of this census, and
- iii) Participation in a Interlaboratory Comparison Program to ensure that independent checks on the precision and accuracy of the measurements of radioactive materials in environmental sample matrices are performed as part of the quality assurance program for environmental monitoring.

### g. Solid Radioactive Wastes Control Program

The PROCESS CONTROL PROGRAM (PCP) shall contain the current formulas, sampling, analyses, test, and determinations to be made to ensure that processing and packaging of solid radioactive wastes based on demonstrated processing of actual or simulated wet solid wastes will be accomplished in such a way as to assure compliance with 10 CFR Parts 20, 61, and 71; State regulations; burial ground requirements; and other requirements governing the disposal of solid radioactive waste.

## ADMINISTRATIVE CONTROLS

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### 6.9 REPORTING REQUIREMENTS

#### ROUTINE REPORTS

6.9.1 In addition to the applicable reporting requirements of Title 10, Code of Federal Regulations, the following reports shall be submitted to the Commission, pursuant to 10CFR50.4.

#### STARTUP REPORT

6.9.1.1 A summary report of plant startup and power escalation testing shall be submitted following (1) receipt of an operating license, (2) amendment to the license involving a planned increase in power level, (3) installation of fuel that has a different design or has been manufactured by a different fuel supplier, and (4) modifications that may have significantly altered the nuclear, thermal, or hydraulic performance of the plant.



## **ADMINISTRATIVE CONTROLS**

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### **ANNUAL RADIOLOGICAL ENVIRONMENTAL OPERATING REPORT**

6.9.1.6 and 6.9.1.7 The Annual Radiological Environmental Operating Report covering the operation of the unit during the previous calendar year shall be submitted before May 1 of each year. The report shall include summaries, interpretations, and analysis of trends of the results of the Radiological Environmental Monitoring Program for the reporting period. The material provided shall be consistent with the objectives outlined in (1) the ODCM and (2) Sections IV.B.2, IV.B.3, and IV.C of Appendix I to 10 CFR Part 50. A single submittal may be made for a multiple unit station.

### **SEMIANNUAL RADIOACTIVE EFFLUENT RELEASE REPORT**

6.9.1.8 and 6.9.1.9 The Semiannual Radioactive Effluent Release Report covering the operation of the unit during the previous 6 months of operation shall be submitted within 60 days after January 1 and July 1 of each year. The report shall include a summary of the quantities of radioactive liquid and gaseous effluents and solid waste released from the unit. The material provided shall be (1) consistent with the objectives outlined in the ODCM and PCP and (2) in conformance with 10 CFR 50.36a and Section IV.B.1 of Appendix I to 10 CFR Part 50. A single submittal may be made for a multiple unit station. The submittal should combine those sections that are common to all units at the station; however, for units with separate radwaste systems, the submittal shall specify the releases of radioactive material from each unit.

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## ADMINISTRATIVE CONTROLS

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### MONTHLY OPERATING REPORT

6.9.1.10 Routine reports of operating statistics and shutdown experience, including documentation of all challenges to the PORV's or safety valves, shall be submitted on a monthly basis to the Commission, pursuant to 10 CFR 50.4, no later than the 15th of each month following the calendar month covered by the report.

### RADIAL PEAKING FACTOR LIMIT REPORT

6.9.1.11 The  $F_{xy}$  limit for Rated Thermal Power ( $F_{xy}$ ) for all core planes containing bank "D" control rods and all unrodded core planes shall be established and documented in the Radial Peaking Factor Limit Report before each reload cycle (prior to MODE 2) and provided to the Commission, pursuant to 10 CFR 50.4, upon issuance. In the event that the limit would be submitted at some other time during core life, it will be submitted upon issuance, unless otherwise exempted by the Commission.

Any information needed to support  $F_{xy}$  <sup>RTP</sup> will be by request from the NRC and need not be included in this report.

### ANNUAL DIESEL GENERATOR RELIABILITY DATA REPORT

6.9.1.12 The number of tests (valid or invalid) and the number of failures to start on demand for each diesel generator shall be submitted to the NRC annually. This report shall contain the information identified in Regulatory Position C.3.b of NRC Regulatory Guide 1.108, Revision 1, 1977.

## **ADMINISTRATIVE CONTROLS**

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- h. Records of annual physical inventory of all sealed source material of record.

6.10.2 The following records shall be retained for the duration of the Unit Operating License.

- a. Records and drawing changes reflecting unit design modifications made to systems and equipment described in the Final Safety Analysis Report.
- b. Records of new and irradiated fuel inventory, fuel transfers and assembly burnup histories.
- c. Records of radiation exposure for all individuals entering radiation control areas.
- d. Records of gaseous and liquid radioactive material released to the environs.
- e. Records of transient or operational cycles for those unit components identified in Table 5.7-1.
- f. Records of reactor tests and experiments.
- g. Records of training and qualification for current members of the facility staff.
- h. Records of in-service inspections performed pursuant to these Technical Specifications.
- i. Records of safety-related Quality Assurance activities required by the Operations Quality Assurance Policy Manual (OQAPM) and not specifically described in Technical Specification 6.10.1.
- j. Records of reviews performed for changes made to procedures or equipment or reviews of tests and experiments pursuant to 10CFR50.59.
- k. Records of meetings of the PORC and the NORB.
- l. Records of secondary water sampling and water quality.
- m. Records of analyses required by the radiological environmental monitoring program.
- n. Records of the service lives of all hydraulic and mechanical snubbers within the scope of 3/4.7.9 including the date at which the service life commences and associated installation and maintenance records.
- o. Records of reviews performed for changes made to the OFFSITE DOSE CALCULATION MANUAL and the PROCESS CONTROL PROGRAM.

## **ADMINISTRATIVE CONTROLS**

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### **6.11 RADIATION PROTECTION PROGRAM**

Procedures for personnel radiation protection shall be prepared consistent with the requirements of 10CFR Part 20 and shall be approved, maintained and adhered to for all operations involving personnel radiation exposure.

## **ADMINISTRATIVE CONTROLS**

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### **6.13 PROCESS CONTROL PROGRAM (PCP)**

- 6.13.1 The PCP shall be approved by the Commission prior to implementation.
- 6.13.2 Licensee initiated changes to the PCP:
1. Shall be documented and records of reviews performed shall be retained as required by Specification 6.10.2.o. This documentation shall contain:
    - a. Sufficient information to support the change together with the appropriate analyses or evaluations justifying the change(s) and
    - b. A determination that the change will maintain the overall conformance of the solidified waste product to existing requirements of Federal, State, or other applicable regulations.
  2. Shall become effective after review and acceptance by the PORC and the approval of the General Manager - Nuclear Plant.

### **6.14 OFFSITE DOSE CALCULATION MANUAL (ODCM)**

- 6.14.1 The ODCM shall be approved by the Commission prior to implementation.
- 6.14.2 Licensee initiated changes to the ODCM:
1. Shall be documented and records of reviews performed shall be retained as required by Specification 6.10.2.o. This documentation shall contain:
    - a. Sufficient information to support the change together with the appropriate analyses or evaluations justifying the change(s) and
    - b. A determination that the change will maintain the level of radioactive effluent control required by 10 CFR 20.106, 40 CFR Part 190, 10 CFR 50.36a, and Appendix I to 10 CFR Part 50 and not adversely impact the accuracy or reliability of effluent, dose, or setpoint calculations.
  2. Shall become effective after review and acceptance by the PORC and the approval of the General Manager - Nuclear Plant.
  3. Shall be submitted to the Commission in the form of a complete, legible copy of the entire ODCM as a part of or concurrent with the Semiannual Radioactive Effluent Release Report for the period of the report in which any change to the ODCM was made. Each change shall be identified by markings in the margin of the affected pages, clearly indicating the area of the page that was changed, and shall indicate the date (e.g., month/year) the change was implemented.

## ADMINISTRATIVE CONTROLS

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### 6.15 MAJOR CHANGES TO RADIOACTIVE WASTE TREATMENT SYSTEMS (Liquid, Gaseous, and Solid)

This specification deleted. Refer to the Offsite Dose Calculation Manual and the Process Control Program.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
RELATED TO AMENDMENT NO. 99 TO FACILITY OPERATING LICENSE NO. NPF-2  
AND AMENDMENT NO. 91 TO FACILITY OPERATING LICENSE NO. NPF-8

SOUTHERN NUCLEAR OPERATING COMPANY, INC.

JOSEPH M. FARLEY NUCLEAR PLANT, UNITS 1 AND 2

DOCKET NOS. 50-348 AND 50-364

1.0 INTRODUCTION

By letter dated June 23, 1992, as supplemented November 13, 1992, the Southern Nuclear Operating Company, Inc. proposed to incorporate programmatic controls for radiological effluents and radiological environmental monitoring in the Administrative Controls section of the Technical Specifications (TS) consistent with the requirements of 10 CFR 20.106, 40 CFR Part 190, 10 CFR 50.36a, and Appendix I to 10 CFR Part 50. At the same time, the licensee proposed to transfer the procedural details of the Radiological Effluent Technical Specifications (RETS) from the TS to the Offsite Dose Calculation Manual (ODCM) or to the Process Control Program (PCP) for solid radioactive wastes, as appropriate. With these changes, the specifications related to RETS reporting requirements were simplified. Finally, changes to the definitions of the ODCM and PCP were proposed consistent with these changes. Guidance on these proposed changes was provided to all power reactor licensees and applicants by Generic Letter (GL) 89-01, dated January 31, 1989. The November 13, 1992; letter provided administrative corrections that did not change the initial proposed no significant hazards consideration determination.

2.0 EVALUATION

The licensee's proposed changes to the TS are in accordance with the guidance provided in GL 89-01 and are addressed below.

- (1) The licensee has proposed to incorporate programmatic controls for radioactive effluents and radiological environmental monitoring in TS 6.8.3, Procedures and Programs, as noted in the guidance provided in GL 89-01. The programmatic controls ensure that programs are established, implemented, and maintained to ensure that operating procedures are provided to control radioactive effluents consistent with the requirements of 10 CFR 20.106, 40 CFR Part 190, 10 CFR 50.36a, and Appendix I to 10 CFR Part 50.
- (2) The licensee has confirmed that the detailed procedural requirements addressing Limiting Conditions for Operation, their applicability, remedial actions, associated surveillance requirements, or reporting requirements for the following TS have been prepared to implement



the relocation of these procedural details to the ODCM or PCP. These changes to the ODCM and PCP have been prepared in accordance with the new Administrative Controls in the TS on changes to the ODCM and PCP so that they will be implemented in the ODCM or PCP when this amendment is issued.

SPECIFICATION

TITLE

3/4.3.3.10	RADIOACTIVE LIQUID EFFLUENT MONITORING INSTRUMENTATION
3/4.3.3.11	WASTE GAS MONITORING INSTRUMENTATION
3/4.11.1.1	LIQUID EFFLUENTS: CONCENTRATION
3/4.11.1.2	LIQUID EFFLUENTS: DOSE
3/4.11.1.3	LIQUID EFFLUENTS: LIQUID WASTE TREATMENT
3/4.11.2.1	GASEOUS EFFLUENTS: DOSE RATE
3/4.11.2.2	RADIOACTIVE EFFLUENTS: DOSE - NOBLE GASES
3/4.11.2.3	GASEOUS EFFLUENTS: DOSE - RADIOIODINES RADIOACTIVE MATERIAL IN PARTICULATE FORM, AND RADIONUCLIDES OTHER THAN NOBLE GASES
3/4.11.2.4	RADIOACTIVE EFFLUENTS: GASEOUS RADWASTE TREATMENT
3/4.11.3	RADIOACTIVE EFFLUENTS: RADWASTE SOLIDIFICATION
3/4.11.4	RADIOACTIVE EFFLUENTS: TOTAL DOSE
3/4.12.1	RADIOLOGICAL ENVIRONMENTAL MONITORING: MONITORING PROGRAM
3/4.12.2	RADIOLOGICAL ENVIRONMENTAL MONITORING: LAND USE CENSUS
3/4.12.3	RADIOLOGICAL ENVIRONMENTAL MONITORING: INTERLABORATORY COMPARISON PROGRAM
6.9.1.6	REPORTING REQUIREMENTS: ANNUAL RADIOLOGICAL ENVIRONMENTAL
6.9.1.7	OPERATING REPORT
6.9.1.8	REPORTING REQUIREMENTS: SEMI-ANNUAL RADIOACTIVE EFFLUENT
6.9.1.9	RELEASE REPORT
6.9.1.10	MONTHLY OPERATING REPORT
6.10.2	RECORD RETENTION

- 6.13.2            PROCESS CONTROL PROGRAM (PCP)
- 6.14.2            OFFSITE DOSE CALCULATION MANUAL (ODCM)
- 6.15.1            MAJOR CHANGES TO RADIOACTIVE WASTE TREATMENT SYSTEMS  
                  (Liquid, Gaseous, and Solid)

In addition, the bases associated with the above listed TS will be deleted.

The procedural details that have been removed from the TS are not required by the Commission's regulations to be included in the TS. They have been prepared for incorporation in the ODCM or PCP upon issuance of this license amendment and may be subsequently changed by the licensee without prior NRC approval. Changes to the ODCM and PCP are documented and will be retained for the duration of the operating licensee in accordance with TS 6.10.2.o.

- (3) The licensee has proposed replacing the following TS in the administrative controls section of the TS with the updated TS provided by GL-89-01:

<u>SPECIFICATION</u>	<u>TITLE</u>
6.9.1.6 6.9.1.7	ANNUAL RADIOLOGICAL ENVIRONMENTAL OPERATING REPORT
6.9.1.8 6.9.1.9	SEMI-ANNUAL RADIOACTIVE EFFLUENT RELEASE REPORT
6.13.2	PROCESS CONTROL PROGRAM (PCP)
6.14.2	OFFSITE DOSE CALCULATION MANUAL (ODCM)

In accordance with the guidance of GL 89-01 the following sections, which are included under the heading of Radioactive Effluents, have been retained in the TS.

<u>SPECIFICATION</u>	<u>TITLE</u>
3/4.3.3.11	WASTE GAS MONITORING INSTRUMENTATION
3/4.11.1.4	LIQUID HOLDUP TANKS
3/4.11.2.5	WASTE GAS MONITORING
3/4.11.2.6	GAS STORAGE TANKS

The staff finds that the changes included in the proposed TS amendment request are consistent with the guidance provided in GL 89-01. Because the control of radioactive effluents continues to be limited in accordance with operating procedures that must satisfy the regulatory requirements of 10 CFR 20.106, 40 CFR Part 190, 10 CFR 50.36a, and Appendix I to 10 CFR Part 50, the NRC staff concludes that these changes are administrative in nature and there is no impact on plant safety as a consequence. Accordingly, the staff finds the proposed change acceptable.

### 3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the State of Alabama official was notified of the proposed issuance of the amendment. The State official had no comments.

### 4.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The amendment also relates to changes in recordkeeping, reporting, or administrative procedures or requirements. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (57 FR 45088). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

### 5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributors: T. Dunning  
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Date: September 20, 1993