



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

September 12, 1984

Docket Nos. 50-348
and 50-364

Mr. R. P. McDonald
Senior Vice President
Alabama Power Company
Post Office Box 2641
Birmingham, Alabama 35291

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CMiles
RFerguson
RBallard

Dear Mr. McDonald:

The Commission has issued the enclosed Amendment No. 47 to Facility Operating License No. NPF-2 and Amendment No. 38 to NPF-8 for the Joseph M. Farley Nuclear Plant, Unit Nos. 1 and 2, respectively. The amendments consist of changes to the Technical Specifications in response to your application transmitted by letter dated December 12, 1983.

The amendments modify the Technical Specifications to add: (1) reactor vessel head vents, (2) noble gas effluent monitors, (3) containment water level monitors, and (4) instrumentation for detection of inadequate core cooling required by the Commission in NUREG-0737. These added limiting conditions of operation and surveillance requirements were provided to the licensee in our Generic Letter 83-37 dated November 1, 1983.

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular monthly Federal Register notice.

Sincerely,

/s/EReeves

Edward A. Reeves, Project Manager
Operating Reactors Branch #1
Division of Licensing

Enclosures:

1. Amendment No.47 to NPF-2
2. Amendment No. 38 to NPF-8
3. Safety Evaluation

cc: w/enclosures
See next page

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CParrish
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No input objection

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Mr. R. P. McDonald
Alabama Power Company

Joseph M. Farley Nuclear Plant
Units 1 and 2

cc: Mr. W. O. Whitt
Executive Vice President
Alabama Power Company
Post Office Box 2641
Birmingham, Alabama 35291

D. Biard MacGuineas, Esquire
Volpe, Boskey and Lyons
918 16th Street, N.W.
Washington, DC 20006

Mr. Louis B. Long, General Manager
Southern Company Services, Inc.
Post Office Box 2625
Birmingham, Alabama 35202

Charles R. Lowman
Alabama Electric Corporation
Post Office Box 550
Andalusia, Alabama 36420

Houston County Commission
Dothan, Alabama 36301

James P. O'Reilly
Regional Administrator - Region II
U.S. Nuclear Regulatory Commission
101 Marietta Street, Suite 2900
Atlanta, GA 30303

George F. Trowbridge, Esquire
Shaw, Pittman, Potts and Trowbridge
1800 M Street, N.W.
Washington, DC 20036

Chairman
Houston County Commission
Dothan, Alabama 36301

Robert A. Buettner, Esquire
Balch, Bingham, Baker, Hawthorne,
Williams and Ward
Post Office Box 306
Birmingham, Alabama 35201

Resident Inspector
U.S. Nuclear Regulatory Commission
Post Office Box 24 - Route 2
Columbia, Alabama 36319

State Department of Public Health
ATTN: State Health Officer
State Office Building
Montgomery, Alabama 36104

Regional Radiation Representative
EPA Region IV
345 Courtland Street, N.E.
Atlanta, GA 30308



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

ALABAMA POWER COMPANY

DOCKET NO. 50-348

JOSEPH M. FARLEY NUCLEAR PLANT, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 47
License No. NPF-2

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Alabama Power Company (the licensee) dated December 12, 1983, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, as amended, the provisions of the Act, and the regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-2 is hereby amended to read as follows:

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(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 47, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION


Steven A. Varga, Chief
Operating Reactors Branch #1
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: September 12, 1984

ATTACHMENT TO LICENSE AMENDMENT NO. 47
AMENDMENT NO. 47 FACILITY OPERATING LICENSE NO. NPF-2
DOCKET NO. 50-348

Revised Appendix A as follows:

<u>Remove Pages</u>	<u>Insert Pages</u>
3/4 3-40	3/4 3-40
3/4 3-57	3/4 3-57
3/4 4-36	3/4 4-36
B3/4 4-14	B3/4 4-14
3/4 6-19	3/4 6-19

TABLE 3.3-6 (Continued)

ACTION STATEMENTS

- ACTION 23 - With the number of channels OPERABLE less than required by the Minimum Channels OPERABLE requirement, perform area surveys of the monitored area with portable monitoring instrumentation at least once per 24 hours.
- ACTION 24 - With the number of channels OPERABLE less than required by the Minimum Channels OPERABLE requirement, comply with the ACTION requirements of Specification 3.4.7.1.
- ACTION 25 - With the number of channels OPERABLE less than required by the Minimum Channels OPERABLE requirement, comply with the ACTION requirements of Specification 3.9.12 and 3.9.13.
- ACTION 26 - With the number of channels OPERABLE less than required by the Minimum Channels OPERABLE requirement, comply with the ACTION requirements of Specification 3.9.9.
- ACTION 27 - With the number of channels OPERABLE less than required by the Minimum Channels OPERABLE requirement, within 1 hour initiate and maintain operation of the control room emergency ventilation system in the recirculation mode of operation.
- ACTION 27a - With the number of OPERABLE Channels less than required by the Minimum Channels OPERABLE requirement, initiate the preplanned alternate method of monitoring the appropriate parameter(s), within 72 hours, and:
- 1) Either restore the inoperable Channel(s) to OPERABLE status within 7 days, or
 - 2) Prepare and submit a Special Report to the Commission pursuant to Specification 6.9.2 within the next 14 days following the event outlining the action taken, the cause of the inoperability and the plans and schedule for restoring the system to OPERABLE status.
- a. With fuel in storage pool.
b. With irradiated fuel in the storage pool.
c. Above background with no flow.
d. With mini-purge in operation.
e. With slow speed main purge in operation.
f. With fast speed main purge in operation.

TABLE 3.3-11

ACCIDENT MONITORING INSTRUMENTATION

<u>INSTRUMENT</u>	<u>REQUIRED NUMBER OF CHANNELS</u>	<u>MINIMUM CHANNELS OPERABLE</u>
1. Reactor Coolant Outlet Temperature-T _{hot} -Wide Range	2	1
2. Reactor Coolant Inlet Temperature-T _{cold} -Wide Range	2	1
3. Reactor Coolant Pressure-Wide Range	2	1
4. Steam Generator Water Level-Wide Range or Narrow Range	2/steam generator	1/steam generator
5. Refueling Water Storage Tank Water Level	2	1
6. Containment Pressure	2	1
7. Pressurizer Water Level	2	1
8. Steam Line Pressure	2/steam generator	1/steam generator
9. Auxiliary Feedwater Flow Rate	2	1
10. Reactor Coolant System Subcooling Margin Monitor	2	1
*11. PORV Position Indicator	1/valve	1/valve
**12. PORV Block Valve Position Indicator	1/valve	1/valve
13. Safety Valve Position Indication (One channel is position indicator and one channel is discharge temperature)	2/valve	1/valve
14. Containment Water Level - Narrow Range	1***	1***
15. Containment Water Level - Wide Range	2	1
16. Incore Thermocouples	4/core quadrant	2/core quadrant

*Not applicable if the associated block valve is in the closed position.

**Not applicable if the block valve is verified in the closed position and power removed.

***Operation may continue up to 30 days with less than minimum channels operable for narrow range instruments.

REACTOR COOLANT SYSTEM

3/4.4.12 REACTOR VESSEL HEAD VENTS

LIMITING CONDITION FOR OPERATION

3.4.12 At least one of the two reactor vessel head vent system paths, consisting of two valves in series powered from the Auxiliary Building D.C. Distribution System, shall be OPERABLE and closed.

APPLICABILITY: MODES 1, 2, 3 and 4.

ACTION:

With both reactor vessel head vent system paths inoperable, maintain the inoperable vent paths closed with power removed from the valve actuators of all valves in the inoperable vent paths; restore at least one of the vent paths to OPERABLE status within 30 days or be in HOT STANDBY within 6 hours and in COLD SHUTDOWN within the following 30 hours.

SURVEILLANCE REQUIREMENTS

4.4.12 Each reactor vessel head vent system shall be demonstrated OPERABLE at least once per 18 months by:

- a. Verifying all manual isolation valves in each vent path are locked in the open position.
- b. Operating each valve in the vent system through one cycle of full travel from the control room during COLD SHUTDOWN or REFUELING.
- c. Verifying flow through the reactor vessel head vent systems by venting during COLD SHUTDOWN or REFUELING.

REACTOR COOLANT SYSTEM

BASES

The use of the composite curve is necessary to set conservative heatup limitations because it is possible for conditions to exist such that over the course of the heatup ramp the controlling condition switches from the inside to the outside and the pressure limit must at all times be based on analysis of the most critical criterion.

Finally, the composite curves for the heatup rate data and the cooldown rate data are adjusted for possible errors in the pressure and temperature sensing instruments by the values indicated on the respective curves.

Although the pressurizer operates in temperature ranges above those for which there is reason for concern of non-ductile failure, operating limits are provided to assure compatibility of operation with the fatigue analysis performed in accordance with the AMSE Code requirements.

The OPERABILITY of two RHR relief valves or an RCS vent opening of greater than or equal to 2.85 square inches ensures that the RCS will be protected from pressure transients which could exceed the limits of Appendix G to 10CFR Part 50 when one or more of the RCS cold legs are less than or equal to 310°F. Either RHR relief valve has adequate relieving capability to protect the RCS from overpressurization when the transient is limited to either (1) the start of an idle RCP with the secondary water temperature of the steam generator less than or equal to 50°F above the RCS cold leg temperatures or (2) the start of 3 charging pumps and their injection into a water solid RCS.

3/4.4.11 STRUCTURAL INTEGRITY

The inservice inspection and testing programs for ASME Code Class 1, 2 and 3 components ensure that the structural integrity and operational readiness of these components will be maintained at an acceptable level throughout the life of the plant. These programs are in accordance with Section XI of the ASME Boiler and Pressure Vessel Code and applicable Addenda as required by 10CFR Part 50.55a(g) except where specific written relief has been granted by the Commission pursuant to 10CFR Part 50.55a (g)(6)(i).

3/4. 4.12 Reactor Vessel Head Vents

The OPERABILITY of the Reactor Head Vent System ensures that adequate core cooling can be maintained in the event of the accumulation of non-condensable gases in the reactor vessel. This system is in accordance with 10CFR50.44(c)(3)(iii).

CONTAINMENT SYSTEMS

3/4.6.4 COMBUSTIBLE GAS CONTROL

HYDROGEN ANALYZERS

LIMITING CONDITION FOR OPERATION

3.6.4.1 Two independent containment hydrogen analyzers shall be OPERABLE.

APPLICABILITY: MODES 1 and 2.

ACTION:

- a. With one hydrogen analyzer inoperable, restore the inoperable analyzer to OPERABLE status within 30 days or be in at least HOT STANDBY within the next 6 hours.
- b. With both hydrogen analyzers inoperable, restore at least one analyzer to OPERABLE status within 72 hours or be in at least HOT STANDBY within the next 6 hours.

SURVEILLANCE REQUIREMENTS

4.6.4.1 Each hydrogen analyzer shall be demonstrated OPERABLE at least once per 92 days on a STAGGERED TEST BASIS by performing a CHANNEL CALIBRATION using sample gases containing:

- a. Ten volume percent hydrogen, balance nitrogen, for zero check.
- b. Ten volume percent hydrogen, balance nitrogen, mixed with compressed air, for span check.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

ALABAMA POWER COMPANY

DOCKET NO. 50-364

JOSEPH M. FARLEY NUCLEAR PLANT, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 38
License No. NPF-8

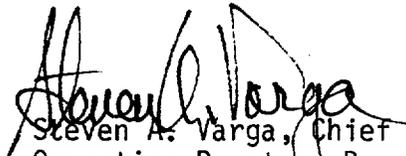
1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Alabama Power Company (the licensee) dated December 12, 1983, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, as amended, the provisions of the Act, and the regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-8 is hereby amended to read as follows:

(2) . Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No.38 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION


Steven A. Varga, Chief
Operating Reactors Branch #1
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: September 12, 1984

ATTACHMENT TO LICENSE AMENDMENT NO. 38
AMENDMENT NO. 38 FACILITY OPERATING LICENSE NO. NPF-8
DOCKET NO. 50-364

Revised Appendix A as follows:

<u>Remove Pages</u>	<u>Insert Pages</u>
3/4 3-40	3/4 3-40
3/4 3-57	3/4 3-57
3/4 4-36	3/4 4-36
B3/4 4-14	B3/4 4-14
3/4 6-19	3/4 6-19

TABLE 3.3-6 (Continued)

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- ACTION 23 - With the number of channels OPERABLE less than required by the Minimum Channels OPERABLE requirement, perform area surveys of the monitored area with portable monitoring instrumentation at least once per 24 hours.
- ACTION 24 - With the number of channels OPERABLE less than required by the Minimum Channels OPERABLE requirement, comply with the ACTION requirements of Specification 3.4.7.1.
- ACTION 25 - With the number of channels OPERABLE less than required by the Minimum Channels OPERABLE requirement, comply with the ACTION requirements of Specification 3.9.12 and 3.9.13.
- ACTION 26 - With the number of channels OPERABLE less than required by the Minimum Channels OPERABLE requirement, comply with the ACTION requirements of Specification 3.9.9.
- ACTION 27 - With the number of channels OPERABLE less than required by the Minimum Channels OPERABLE requirement, within 1 hour initiate and maintain operation of the control room emergency ventilation system in the recirculation mode of operation.
- ACTION 27a - With the number of OPERABLE Channels less than required by the Minimum Channels OPERABLE requirement, initiate the preplanned alternate method of monitoring the appropriate parameter(s), within 72 hours, and:
- 1) Either restore the inoperable Channels(s) to OPERABLE status within 7 days, or
 - 2) Prepare and submit a Special Report to the Commission pursuant to Specification 6.9.2 within the next 14 days following the event outlining the action taken, the cause of the inoperability and the plans and schedule for restoring the system to OPERABLE status.
- a. With fuel in storage pool.
- b. With irradiated fuel in the storage pool.
- c. Above background with no flow.
- d. With mini-purge in operation.
- e. With slow speed main purge in operation.
- f. With fast speed main purge in operation.

TABLE 3.3-11

ACCIDENT MONITORING INSTRUMENTATION

FARLEY - UNIT 2

3/4 3-57

Amendment No. 38

<u>INSTRUMENT</u>	<u>REQUIRED NUMBER OF CHANNELS</u>	<u>MINIMUM CHANNELS OPERABLE</u>
1. Reactor Coolant Outlet Temperature-T _{hot} -Wide Range	2	1
2. Reactor Coolant Inlet Temperature-T _{cold} -Wide Range	2	1
3. Reactor Coolant Pressure-Wide Range	2	1
4. Steam Generator Water Level-Wide Range or Narrow Range	2/steam generator	1/steam generator
5. Refueling Water Storage Tank Water Level	2	1
6. Containment Pressure	2	1
7. Pressurizer Water Level	2	1
8. Steam Line Pressure	2/steam generator	1/steam generator
9. Auxiliary Feedwater Flow Rate	2	1
10. Reactor Coolant System Subcooling Margin Monitor	2	1
*11. PORV Position Indicator	1 /valve	1/valve
**12. PORV Block Valve Position Indicator	1/valve	1/valve
13. Safety Valve Position Indication (One channel is position indicator and one channel is discharge temperature)	2/valve	1/valve
14. Containment Water Level - Narrow Range	1***	1***
15. Containment Water Level - Wide Range	2	1
16. Incore Thermocouples	4/core quadrant	2/core quadrant

*Not applicable if the associated block valve is in the closed position.

**Not applicable if the block valve is verified in the closed position and power removed.

***Operation may continue up to 30 days with less than minimum channels operable for narrow range instruments.

REACTOR COOLANT SYSTEM

3/4.4.12 REACTOR VESSEL HEAD VENTS

LIMITING CONDITION FOR OPERATION

3.4.12 At least one of the two reactor vessel head vent system paths, consisting of two valves in series powered from the Auxiliary Building D.C. Distribution System, shall be OPERABLE and closed.

APPLICABILITY: MODES 1, 2, 3 and 4.

ACTION:

With both reactor vessel head vent system paths inoperable, maintain the inoperable vent paths closed with power removed from the valve actuators of all valves in the inoperable vent paths; restore at least one of the vent paths to OPERABLE status within 30 days or be in HOT STANDBY within 6 hours and in COLD SHUTDOWN within the following 30 hours.

SURVEILLANCE REQUIREMENTS

4.4.12 Each reactor vessel head vent system shall be demonstrated OPERABLE at least once per 18 months by:

- a. Verifying all manual isolation valves in each vent path are locked in the open position.
- b. Operating each valve in the vent system through one cycle of full travel from the control room during COLD SHUTDOWN or REFUELING.
- c. Verifying flow through the reactor vessel head vent systems by venting during COLD SHUTDOWN or REFUELING.

REACTOR COOLANT SYSTEM

BASES

The use of the composite curve is necessary to set conservative heatup limitations because it is possible for conditions to exist such that over the course of the heatup ramp the controlling condition switches from the inside to the outside and the pressure limit must at all times be based on analysis of the most critical criterion.

Finally, the composite curves for the heatup rate data and the cooldown rate data are adjusted for possible errors in the pressure and temperature sensing instruments by the values indicated on the respective curves.

Although the pressurizer operates in temperature ranges above those for which there is reason for concern of non-ductile failure, operating limits are provided to assure compatibility of operation with the fatigue analysis performed in accordance with the AMSE Code requirements.

The OPERABILITY of two RHR relief valves or an RCS vent opening of greater than or equal to 2.85 square inches ensures that the RCS will be protected from pressure transients which could exceed the limits of Appendix G to 10CFR Part 50 when one or more of the RCS cold legs are less than or equal to 310°F. Either RHR relief valve has adequate relieving capability to protect the RCS from overpressurization when the transient is limited to either (1) the start of an idle RCP with the secondary water temperature of the steam generator less than or equal to 50°F above the RCS cold leg temperatures or (2) the start of 3 charging pumps and their injection into a water solid RCS.

3/4.4.11 STRUCTURAL INTEGRITY

The inservice inspection and testing programs for ASME Code Class 1, 2 and 3 components ensure that the structural integrity and operational readiness of these components will be maintained at an acceptable level throughout the life of the plant. These programs are in accordance with Section XI of the ASME Boiler and Pressure Vessel Code and applicable Addenda as required by 10CFR Part 50.55a(g) except where specific written relief has been granted by the Commission pursuant to 10CFR Part 50.55a (g)(6)(i).

3/4. 4.12 Reactor Vessel Head Vents

The OPERABILITY of the Reactor Head Vent System ensures that adequate core cooling can be maintained in the event of the accumulation of non-condensable gases in the reactor vessel. This system is in accordance with 10CFR50.44(c)(3)(iii).

CONTAINMENT SYSTEMS

3/4.6.4 COMBUSTIBLE GAS CONTROL

HYDROGEN ANALYZERS

LIMITING CONDITION FOR OPERATION

3.6.4.1 Two independent containment hydrogen analyzers shall be OPERABLE.

APPLICABILITY: MODES 1 and 2.

ACTION:

- a. With one hydrogen monitor inoperable, restore the inoperable analyzer to OPERABLE status within 30 days or be in at least HOT STANDBY within the next 6 hours.
- b. With both hydrogen monitors inoperable, restore at least one monitor to OPERABLE status within 72 hours or be in at least HOT STANDBY within the next 6 hours.

SURVEILLANCE REQUIREMENTS

4.6.4.1 Each hydrogen analyzer shall be demonstrated OPERABLE at least once per 92 days on a STAGGERED TEST BASIS by performing a CHANNEL CALIBRATION using sample gases containing:

- a. One volume percent hydrogen, balance nitrogen, for zero check.
- b. Four volume percent hydrogen, balance nitrogen, mixed with compressed air, for span check.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 47 TO FACILITY OPERATING LICENSE NO. NPF-2
AND AMENDMENT NO. 38 TO FACILITY OPERATING LICENSE NO. NPF-8

ALABAMA POWER COMPANY

JOSEPH M. FARLEY NUCLEAR PLANT, UNIT NOS. 1 AND 2

DOCKET NOS. 50-348 AND 50-364

INTRODUCTION AND BACKGROUND

In November 1980, the staff issued NUREG-0737, "Clarification of TMI Action Plan Requirements", which included all TMI Action Plan items approved by the Commission for implementation at nuclear power reactors. NUREG-0737 identifies those items for which Technical Specifications were scheduled for implementation after December 31, 1981. The staff provided guidance on the scope of Technical Specifications for all of these items in Generic Letter 83-37. Generic Letter 83-37 was issued to all Pressurized Water Reactor (PWR) licensees on November 1, 1983. In this Generic Letter, the staff requested licensees to:

1. review their facility's Technical Specifications to determine if they were consistent with the guidance provided in the Generic Letter, and
2. submit an application for a license amendment where deviations or absence of Technical Specifications were found.

By letter dated December 12, 1983, Alabama Power Company (the licensee) responded to Generic Letter 83-37 by submitting Technical Specification change requests for Farley Units 1 and 2. This evaluation covers the following TMI Action Plan items:

1. Reactor Coolant System Vents (II.B.1)
2. Noble Gas Effluent Monitors (II.F.1.1)
3. Containment Water Level Monitor (II.F.1.5)
4. Containment Hydrogen Analyzer (II.F.1.6)

EVALUATION

1. Reactor Coolant System Vents (II.B.1)

Our guidance for RCS vents identified the need for at least one operable vent path at the reactor vessel head and the pressurizer steam space, for Westinghouse reactors. Generic Letter 83-37 also provided limiting conditions for operation and the surveillance requirements for the RCS vents.

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PDR

The licensee has proposed TSs for reactor vessel head vents that are consistent with our guidance contained in Generic Letter 83-37. The licensee indicated that TSs for pressurizer steam space are covered by existing Technical Specifications for the pressurizer relief valves. We have reviewed the proposed TSs for reactor vessel head vents and existing TSs for pressurizer relief valves, and conclude that the licensee has provided TSs that satisfy the intent of the guidelines contained in Generic letter 83-37. Therefore, we find the proposed TSs to be acceptable.

2. Noble Gas Effluent Monitors (II.F.1.1)

The licensee has supplemented the existing normal range monitors to provide noble gas monitoring in accordance with TMI Action Plan Item II.F.1.1. The existing TSs for Farley Units 1 and 2 include TSs for noble gas effluent monitors. However, the licensee has proposed a revision to the present TSs to ensure consistency with the guidelines contained in Generic Letter 83-37. We have reviewed the proposed revision and determined that it is consistent with the guidelines contained in Generic Letter 83-37. Therefore, we find the proposed TSs to be acceptable.

3. Containment Water Level Monitor (II.F.1.5)

Narrow range and wide range containment water level monitors provide the capability required by TMI Action Plan Item II.F.1.5. The TSs for both units contain limiting conditions of operation and surveillance requirements that are consistent with the guidance contained in Generic Letter 83-37. We conclude that the proposed TSs for containment water level monitors are acceptable.

4. Containment Hydrogen Analyzers (II.F.1.6)

The licensee installed containment hydrogen analyzers that provide the capability required by TMI Action Plan Item II.F.1.6. The existing TSs for Farley Units 1 and 2 include TSs for these analyzers. However, the licensee has proposed to revise limiting conditions for operation to be consistent with the guidelines contained in Generic Letter 83-37. We have reviewed the proposed changes and determined that they are consistent with the guidelines contained in Generic Letter 83-37. Therefore, we find the proposed changes to be acceptable.

Environmental Consideration

These amendments involve a change in the installation or use of the facilities components located within the restricted areas as defined in 10 CFR 20. The staff has determined that these amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite and that there is no significant increase in individual or cumulative occupation radiation exposure. The Commission has previously issued a proposed finding that these amendments involve no significant hazards consideration and there has been no public comment on such finding. Accordingly, these amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR Sec 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of these amendments.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

Dated: September 12, 1984

Principal Contributor:

C. Patel