



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

September 21, 1984

Docket Nos. 50-348  
and 50-364

Mr. R. P. McDonald  
Senior Vice President  
Alabama Power Company  
Post Office Box 2641  
Birmingham, Alabama 35291

Dear Mr. McDonald:

The Commission has issued the enclosed Amendment No. 48 to Facility Operating License No. NPF-2 and Amendment No. 39 to NPF-8 for the Joseph M. Farley Nuclear Plant, Unit Nos. 1 and 2, respectively. The amendments consist of changes to the Technical Specifications in response to your application transmitted by letter dated March 1, 1984.

The amendments modify the Technical Specifications to correct errors in Table 4.3-1 (Units 1 and 2), Table 3.6-1 (Units 1 and 2), Table 3.6-1 Section D (Unit 1), and specification 3.7.11.1.c. Also Figure 6.2-2 (Units 1 and 2) is modified to reflect changes in the organizational chart changing the title of the chemistry supervisor and adding a computer services supervisor to the chart.

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular monthly Federal Register notice.

Sincerely,

Edward A. Reeves, Project Manager  
Operating Reactors Branch #1  
Division of Licensing

Enclosures:

1. Amendment No. 48 to NPF-2
2. Amendment No. 39 to NPF-8
3. Safety Evaluation

cc: w/enclosures  
See next page

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~~Farley~~  
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Mr. R. P. McDonald  
Alabama Power Company

Joseph M. Farley Nuclear Plant  
Units 1 and 2

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

ALABAMA POWER COMPANY

DOCKET NO. 50-348

JOSEPH M. FARLEY NUCLEAR PLANT, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 48  
License No. NPF-2

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Alabama Power Company (the licensee) dated March 1, 1984, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, as amended, the provisions of the Act, and the regulations of the Commission;
  - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-2 is hereby amended to read as follows:


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(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 48, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

  
Steven A. Varga, Chief  
Operating Reactors Branch #1  
Division of Licensing

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance:  
September 21, 1984

ATTACHMENT TO LICENSE AMENDMENT NO. 48  
AMENDMENT NO. 48 FACILITY OPERATING LICENSE NO. NPF-2  
DOCKET NO. 50-348

Revised Appendix A as follows:

<u>Remove Pages</u>	<u>Insert Pages</u>
3/4 3-14	3/4 3-14
3/4 6-17	3/4 6-17
3/4 6-18	3/4 6-18
3/4 7-82	3/4 7-82
6-3	6-3

TABLE 4.3-1 (Continued)

TABLE NOTATION

- \* - With the reactor trip system breakers closed and the control rod drive system capable of rod withdrawal.
- (1) - If not performed in previous 7 days.
- (2) - Heat balance only, above 15% of RATED THERMAL POWER. Adjust channel if absolute difference greater than 2 percent.
- (3) - Compare incore to excore axial flux difference every 31 EFPD. Recalibrate if the absolute difference is greater than or equal to 3 percent.
- (4) - Manual ESF functional input check every 18 months.
- (5) - Each train or logic channel shall be tested at least every 62 days on a STAGGERED TEST BASIS.
- (6) - Neutron detectors may be excluded from CHANNEL CALIBRATION.
- (7) - Below the P-6 (Block of Source Range Reactor Trip) setpoint.
- (8) - Logic only, if not performed in previous 92 days.
- (9) - CHANNEL FUNCTIONAL TEST will consist of verifying that each channel indicates a turbine trip prior to latching the turbine and indicates no turbine trip after latching the turbine.
- (10) - If not performed in the previous 31 days.

TABLE 3.6-1 (Continued)

<u>PHASE "A" ISOLATION</u> <u>(Continued)</u>		<u>FUNCTION</u>	<u>ISOLATION TIME</u> <u>(SEC)</u>
32.	CCW-HV-3067	CCW from exc. letdown RCDT HXS	<10
33.	CVC-HV-8860	Accumulators fill line isolation	<10
34.	SS-HV-3766	Accumulator tanks sample isolation valve	<10
35.	SS-HV-3334	Accumulator tanks sample isolation valve	<10
36.	LWP-HV-7126	RCDT vent line isolation valve	<10
37.	LWP-HV-7150	RCDT vent line isolation valve	<10
38.	LWP-HV-3380	Containment sump recirculation valve	<10
39.	CTS-HV-3659	Demineralizer water to reactor HD storage	<10
40.	CBV-HV-3196	Containment purge exhaust isolation valve	<5
41.	CBV-HV-3197	Containment purge supply isolation valve	<5
42.	CBV-HV-2867 C	Containment mini-purge exhaust isolation valve	<5
43.	CBV-HV-2867 D	Containment mini-purge exhaust isolation valve	<5
44.	CBV-HV-2866 C	Containment mini-purge supply isolation valve	<5
45.	CBV-HV-2866 D	Containment mini-purge supply isolation valve	<5
 <b>B. <u>PHASE "B" ISOLATION</u></b>			
1.	CCW-MOV-3052	CCW to RCP coolers	<15
2.	CCW-MOV-3046	CCW from RCP oil coolers	<15
3.	CCW-MOV-3182	CCW from RCP oil coolers	<15
4.	CCW-HV-3184	CCW from RCP THRM BARR	<10
5.	CCW-HV-3045	CCW from RCP THRM BARR	<10
6.	IA-HV-3611	Containment instrument air supply valve	<10
 <b>C. <u>SAFETY INJECTION SIGNAL</u></b>			
1.	CVC-MOV-8107	Charging pumps to regenerative HX	<10
2.	CVC-MOV-8108	Charging pumps to regenerative HX	<10
3.	SW-MOV-3135	SW to RCP motor air coolers	<15
4.	SW-MOV-3131	SW from RCP motor air coolers	<15
5.	SW-MOV-3134	SW from RCP motor air coolers	<15

TABLE 3.6-1 (Continued)

D. <u>MANUAL</u>		<u>FUNCTION</u>	<u>ISOLATION TIME (SEC)</u>
1.	Q1G31V012	Refueling Cavity Supply	NA
2.	Q1G21V005*	Reactor Coolant drain tank	NA
3.	RHR-MOV-8701A**	Reactor coolant LP IC to RHR pump 1A	<120
4.	RHR-MOV-8702A**	Reactor coolant LP 1A to RHR pump 1B	<120
5.	Q1P18V001*	Service Air	NA
6.	Q1P18V002*	Service Air	NA
7.	CBV-MOV-323B	Containment leak rate test valve	NA
8.	CBV-MOV-3239	Containment leak rate test valve	NA
9.	RHR-MOV-8811 A	Containment sump to RHR pump 1A	<17
10.	RHR-MOV-8811 B	Containment sump to RHR pump 1B	<17
11.	RHR-MOV-8812 A	Containment sump to RHR pump 1A	<17
12.	RHR-MOV-8812 B	Containment sump to RHR pump 1B	<17
13.	CS-MOV-8826 A	CS pump 1A containment sump suction isolation	<17
14.	CS-MOV-8826 B	CS pump 1B containment sump suction isolation	<17
15.	CS-MOV-8827 A	CS pump 1A containment sump suction isolation	<17
16.	CS-MOV-8827 B	CS pump 1B containment sump suction isolation	<17
17.	Q1B13V026B*	Pressurizer pressure generator	NA
18.	CBV-MOV-3528 A*	Containment post-LOCA sampling valve 1	NA
19.	CBV-MOV-3528 B*	Containment post-LOCA sampling valve 2	NA
20.	CBV-MOV-3528 C*	Containment post-LOCA sampling valve 3	NA
21.	CBV-MOV-3528 D*	Containment post-LOCA sampling valve 4	NA
22.	CBV-MOV-3739 A*	Containment post-LOCA sampling isolation valve	NA
23.	CBV-MOV-3739 B*	Containment post-LOCA sampling isolation valve	NA
24.	CBV-MOV-3745 A*	Containment post-LOCA sampling return valve	NA
25.	CBV-MOV-3745 B*	Containment post-LOCA sampling return valve	NA
26.	CBV-MOV-3835 A*	Containment post-LOCA sampling return valve	NA
27.	CBV-MOV-3835 B*	Containment post-LOCA sampling return valve	NA
28.	CBV-MOV-3740*	Containment post-LOCA vent isolation valve	NA
29.	CBV-MOV-3530*	Containment post-LOCA vent isolation valve	NA

\*May be opened on an intermittent basis under administrative controls

\*\*May be opened and power removed under administrative controls when the plant is in MODE 4 (for ensuring overpressure protection system operability).



PLANT SYSTEMS

3/4.7.11 FIRE SUPPRESSION SYSTEMS

FIRE SUPPRESSION WATER SYSTEM

LIMITING CONDITION FOR OPERATION

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3.7.11.1 The fire suppression water system shall be OPERABLE with:

- a. Two high pressure pumps, each with a capacity of 2500 gpm, with their discharge aligned to the fire suppression header,
- b. Separate water supplies, each with a minimum contained volume of 250,000 gallons, and
- c. An OPERABLE flow path capable of taking suction from each tank and transferring the water through distribution piping with OPERABLE sectionalizing control or isolation valves to the yard hydrant curb valves, the last valve ahead of the water flow alarm device on each sprinkler or hose standpipe, and the last valve ahead of the deluge valve on each deluge or spray system required to be OPERABLE per Specification 3.7.11.2, 3.7.11.4 and 3.7.11.5.

APPLICABILITY: At all times.

ACTION:

- a. With one of the above required pumps and/or water supplies inoperable, restore the inoperable equipment to OPERABLE status within 7 days or, in lieu of any other report required by Specification 6.9.1, prepare and submit a Special Report to the Commission pursuant to Specification 6.9.2 within the next 30 days outlining the plans and procedures to be used to provide for the loss of redundancy in this system. The provisions of Specifications 3.0.3 and 3.0.4 are not applicable.
- b. With the fire suppression water system otherwise inoperable:
  1. Establish a backup fire suppression water system within 24 hours, and
  2. In lieu of any other report required by Specification 6.9.1, submit a Special Report in accordance with Specification 6.9.2:
    - a) By telephone within 24 hours,
    - b) Confirmed by telegraph, mailgram or facsimile transmission no later than the first working day following the event, and
    - c) In writing within 14 days following the event, outlining the action taken, the cause of the inoperability and the plans and schedule for restoring the system to OPERABLE status.

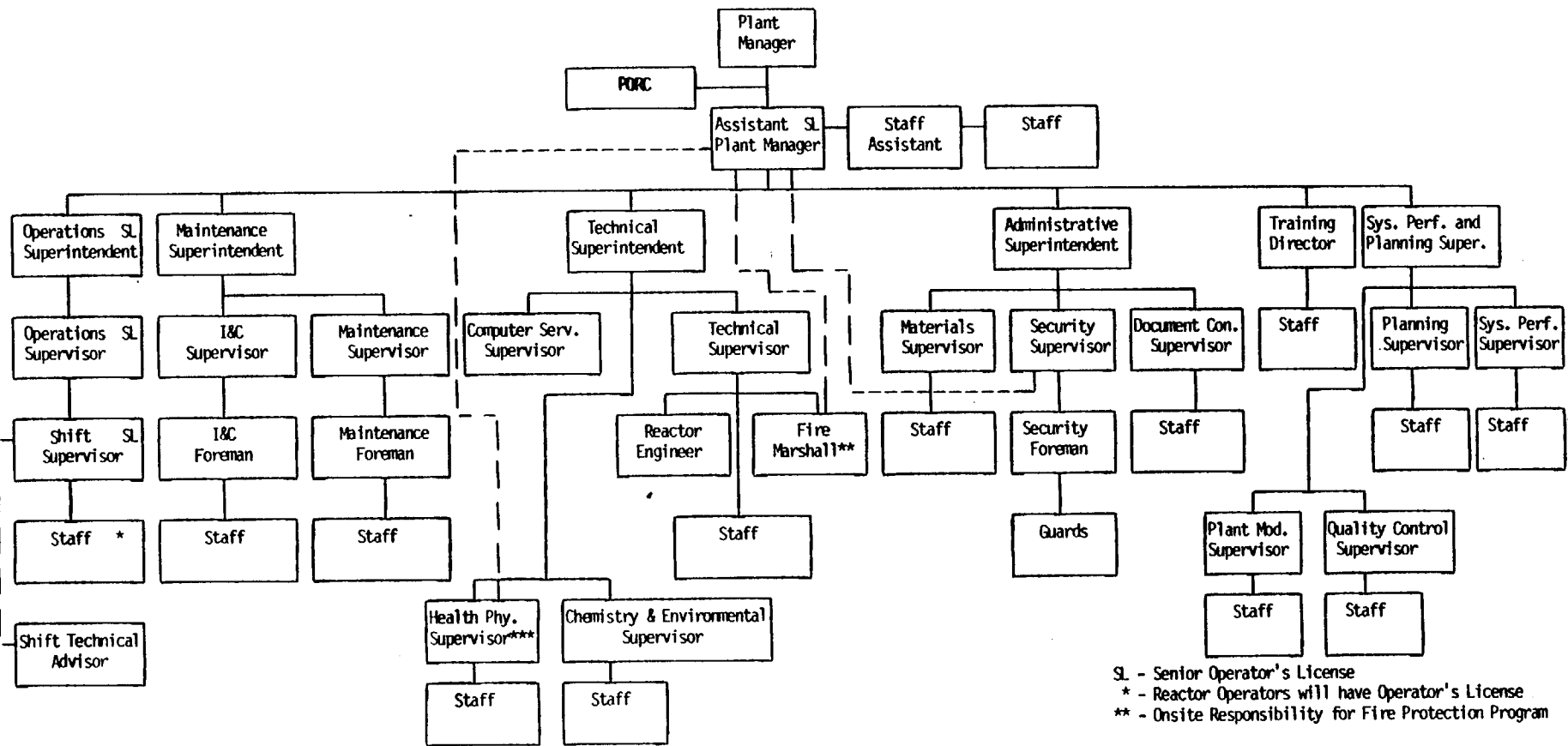


Figure 6.2-2 Facility Organization

\*\*\* In routine matters, the Health Physics Supervisor reports directly to the Technical Superintendent, in matter of radiation policy determination, interpretation or implementation (Based upon the Health Physic Supervisor's judgment) the Health Physics Supervisor may report directly to the Assistant Plant Manager.

PLANT SYSTEMS

3/4.7.11 FIRE SUPPRESSION SYSTEMS

FIRE SUPPRESSION WATER SYSTEM

LIMITING CONDITION FOR OPERATION

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3.7.11.1 The fire suppression water system shall be OPERABLE with:

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

ALABAMA POWER COMPANY

DOCKET NO. 50-364

JOSEPH M. FARLEY NUCLEAR PLANT, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 39  
License No. NPF-8


1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Alabama Power Company (the licensee) dated March 1, 1984, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, as amended, the provisions of the Act, and the regulations of the Commission;
  - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-8 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 39, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

  
Steven A. Varga, Chief  
Operating Reactors Branch #1  
Division of Licensing

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: September 21, 1984

ATTACHMENT TO LICENSE AMENDMENT NO. 39  
AMENDMENT NO.39 FACILITY OPERATING LICENSE NO. NPF-8  
DOCKET NO. 50-364

Revised Appendix A as follows:

Remove Pages

3/4 3-14

3/4 6-17

6-3

Insert Pages

3/4 3-14

3/4 6-17

6-3

TABLE 4.3-1 (Continued)

TABLE NOTATION

- \* - With the reactor trip system breakers closed and the control rod drive system capable of rod withdrawal.
- (1) - If not performed in previous 7 days.
- (2) - Heat balance only, above 15% of RATED THERMAL POWER. Adjust channel if absolute difference greater than 2 percent.
- (3) - Compare incore to excore axial flux difference every 31 EFPD. Recalibrate if the absolute difference is greater than or equal to 3 percent.
- (4) - Manual ESF functional input check every 18 months.
- (5) - Each train or logic channel shall be tested at least every 62 days on a STAGGERED TEST BASIS.
- (6) - Neutron detectors may be excluded from CHANNEL CALIBRATION.
- (7) - Below the P-6 (Block of Source Range Reactor Trip) setpoint.
- (8) - Logic only, if not performed in previous 92 days.
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TABLE 3.6-1 (Continued)

<u>PHASE "A" ISOLATION</u> (Continued)		<u>FUNCTION</u>	<u>ISOLATION TIME</u> (SEC)
32.	CCW-HV-3067	CCW from exc. letdown RCDT HXS	<10
33.	CVC-HV-8860	Accumulators fill line isolation	<10
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35.	SS-HV-3334	Accumulator tanks sample isolation valve	<10
36.	LWP-HV-7126	RCDT vent line isolation valve	<10
37.	LWP-HV-7150	RCDT vent line isolation valve	<10
38.	LWP-HV-3380	Containment sump recirculation valve	<10
39.	CTS-HV-3659	Demineralizer water to reactor HD storage	<5
40.	CBV-HV-3196	Containment purge exhaust isolation valve	<5
41.	CBV-HV-3197	Containment purge supply isolation valve	<5
42.	CBV-HV-2867 C	Containment mini-purge exhaust isolation valve	<5
43.	CBV-HV-2867 D	Containment mini-purge exhaust isolation valve	<5
44.	CBV-HV-2866 C	Containment mini-purge supply isolation valve	<5
45.	CBV-HV-2866 D	Containment mini-purge supply isolation valve	<5
 <b>B. <u>PHASE "B" ISOLATION</u></b>			
1.	CCW-MOV-3052	CCW to RCP coolers	<15
2.	CCW-MOV-3046	CCW from RCP oil coolers	<15
3.	CCW-MOV-3182	CCW from RCP oil coolers	<10
4.	CCW-HV-3184	CCW from RCP THRM BARR	<10
5.	CCW-HV-3045	CCW from RCP THRM BARR	<10
6.	IA-HV-3611	Containment instrument air supply valve	<10
 <b>C. <u>SAFETY INJECTION SIGNAL</u></b>			
1.	CVC-MOV-8107	Charging pumps to regenerative HX	<10
2.	CVC-MOV-8108	Charging pumps to regenerative HX	<10
3.	SW-MOV-3135	SW to RCP motor air coolers	<15
4.	SW-MOV-3131	SW from RCP motor air coolers	<15
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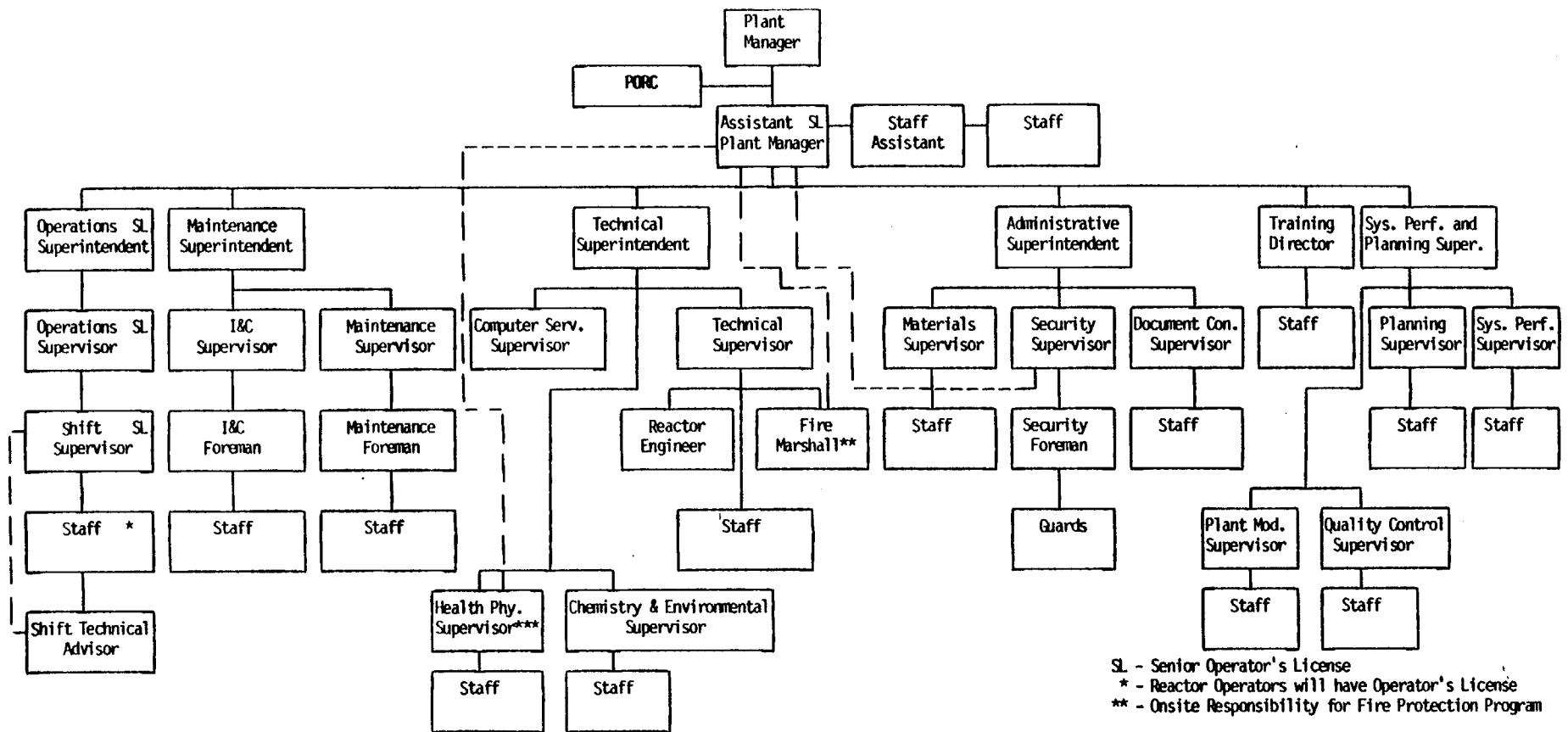


Figure 6.2-2 Facility Organization

\*\*\* In routine matters, the Health Physics Supervisor reports directly to the Technical Superintendent, in matter of radiation policy determination, interpretation or implementation (Based upon the Health Physic Supervisor's judgment) the Health Physics Supervisor may report directly to the Assistant Plant Manager.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
RELATED TO AMENDMENT NO. 48 TO FACILITY OPERATING LICENSE NO. NPF-2  
AND AMENDMENT NO. 39 TO FACILITY OPERATING LICENSE NO. NPF-8

ALABAMA POWER COMPANY

JOSEPH M. FARLEY NUCLEAR PLANT, UNIT NOS. 1 AND 2

DOCKET NOS. 50-348 AND 50-364

INTRODUCTION

By letter dated March 1, 1984, Alabama Power Company (APCo) proposed changes to Technical Specifications identified as administrative corrections. APCo stated that recent operating experience identified the need for changes. Supporting background and justifications were provided "to achieve consistency throughout the Technical Specifications, correction of an error, or a changes in nomenclature." Our evaluation of the licensee's proposed changes follow.

DISCUSSION AND EVALUATION

1. Reactor Trip System Instrumentation Surveillance Requirements Table 4.3-1 (Units 1 and 2)

In their letter of March 1, 1984, APCo proposed changes to the Technical Specifications which would alter the frequency of performance of a nuclear instrumentation channel calibration. The calibration involves comparison of incore and excore axial flux differences and requires the performance of a flux mapping. The proposed revision would change the frequency of performance from monthly when above 15% rated thermal power to every 31 effective full power days to correct the inconsistency with specifications 4.2.2.2.d.1(b) and 4.2.3.1.b, an administrative error.

The proposed change would coordinate this surveillance with other Technical Specification Surveillances which also require flux mapping, thus reducing the number of flux mapping evolutions performed, and the wear on movable incore detector system components. Because incore to excore axial flux differences vary with core burnup, the use of effective full power days to establish performance frequency is acceptable.

2. Containment Isolation Valves Table 3.6-1 (Units 1 and 2)

In their letter of March 1, 1984, APCo proposed changes to Table 3.6-1 of the Technical Specifications which would change the identification numbers

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of eight containment isolation valves. The revised numbers would match the actual identification numbers of the valves. These proposed changes are purely administrative in nature and are acceptable.

### 3. Facility Organization Figure 6.2-2 (Units 1 and 2)

In their letter of March 1, 1984, APCo proposed several changes to the facility organization. The most significant of these were a change in the title of the Chemistry Supervisor to Chemistry and Environmental Supervisor; and the addition of a Computer Services Supervisor to the organization. No significant responsibility changes were associated with the Chemistry and Environmental Supervisor title change, rather the proposed title more closely reflects the responsibilities of the position. The addition of the Computer Services function reflects the importance the licensee places in this function in supporting the operation of the Farley Nuclear Plant.

Several other minor changes were made to the organization chart including the changes of the title of the Performance and Planning Superintendent to System Performance and Planning Superintendent; and the inclusion in the organization chart of staff personnel reporting to the Planning Supervisor, System Performance Supervisor, Plant Modifications Supervisor, and Quality Control Supervisor.

These proposed changes are purely administrative in nature and are acceptable.

### 4. Fire Suppression Water System Specification 3.7.11.1.c (Unit 1)

In their letter of March 1, 1984, APCo proposed changes to the Technical Specifications which would correct certain references to sections relating to fire suppression water system operability. The proposed changes are purely administrative in nature and are acceptable.

### SAFETY SUMMARY

In conclusion, we find that the licensee's proposed changes to the Technical Specifications are administrative in nature and are acceptable.

### Environmental Consideration

These amendments involve a change in the installation or use of the facilities components located within the restricted areas as defined in 10 CFR 20. The staff has determined that these amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that

these amendments involve no significant hazards consideration and there has been no public comment on such finding. Accordingly, these amendments meets the eligibility criteria for categorical exclusion set forth in 10 CFR Sec 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of these amendments.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

Dated: September 21, 1984

Principal Contributor:

D. S. Price, Region II