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March 19, 1991

ocket Nos. 50-348 and 50-364 Posted

Mr. W. G. Hairston, III Senior Vice President Alabama Power Company 40 Inverness Center Parkway Post Office Box 1295 Birmingham, Alabama 35201 See connection letter of 3-22-91)

Dear Mr. Hairston:

SUBJECT: ISSUANCE O

ISSUANCE OF AMENDMENT NO. 88 TO FACILITY OPERATING LICENSE NO. NPF-2 AND AMENDMENT NO. 82 TO FACILITY OPERATING LICENSE NO.

NPF-8 REGARDING SNUBBER VISUAL EXAMINATION SCHEDULE - JOSEPH M. FARLEY NUCLEAR PLANT, UNITS 1 AND 2,

(TAC NOS. 79696 AND 79697)

The Nuclear Regulatory Commission has issued the enclosed Amendment No. 88 to Facility Operating License No. NPF-2 and Amendment No. 82 to Facility Operating License No. NPF-8 for the Joseph M. Farley Nuclear Plant, Units 1 and 2. The amendments consist of changes to the Technical Specifications in response to your submittal dated February 6, 1991.

The amerdments change the Technical Specifications to provide consistency with the guidance of Generic Letter 90-09 that relates to the revision of the surveillance requirements for snubbers.

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's bi-weekly Federal Register notice.

Sincerely,

Orignal Signed By:

Stephen T. Hoffman, Project Manager Project Directorate II-1 Division of Reactor Projects - I/II Office of Nuclear Reactor Regulation

Enclosures:

- 1. Amendment No. 88 to NPF-2
- 2. Amendment No. 82 to NPF-8
- 3. Safety Evaluation

cc w/enclosures: See next page

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OFFICIAL RECORD COPY

Document Name: FARLEY AMEND 79696/79697

Mr. W. G. Hairston, III Alabama Power Company

cc:

Mr. R. P. McDonald Executive Vice President Nuclear Operations Alabama Power Company P. O. Box 1295 Birmingham, Alabama 35201

Mr. B. L. Moore Manager, Licensing Alabama Power Company P. O. Box 1295 Birmingham, Alabama 35201

Mr. Louis B. Long, General Manager Southern Company Services, Inc. Houston County Commission P. O. Box 2625 Birmingham, Alabama 35202

Mr. D. N. Morey General Manager - Farley Nuclear Plant P. O. Box 470 Ashford, Alabama 36312

Mr. J. D. Woodward Vice-President - Nuclear Farley Project Alabama Power Company P. O. Box 1295 Birmingham, Alabama 35201 Joseph M. Farley Nuclear Plant

Resident Inspector U.S. Nuclear Regulatory Commission P. O. Box 24 - Route 2 Columbia, Alabama 36319

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Chairman Houston County Commission Dothan, Alabama 36301

Claude Earl Fox, M.D. State Health Officer State Department of Public Health State Office Building Montgomery, Alabama 36130

James H. Miller, III, Esq. Balch and Bingham P. O. Box 306 1710 Sixth Avenue North Birmingham, Alabama 35201



UNITED STATES JUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

ALABAMA POWER COMPANY

DOCKET NO. 50-348

JOSEPH M. FARLEY NUCLEAR PLANT, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 88 License No. NPF-2

- 1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Alabama Power Company (the licensee), dated February 6, 1991, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission:
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
- 2. Accordingly, the license is amended by changes to the Technical Specifications, as indicated in the attachment to this license amendment; and paragraph 2.C.(2) of Facility Operating License No. NPF-2 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 88, are hereby incorporated in the license. Alabama Power Company shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Orignal Signed By:

Elinor G. Adensam, Director Project Directorate II-1 Division of Reactor Projects - I/II Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical Specifications

Date of Issuance: March 19, 1991

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ATTACHMENT TO LICENSE AMENDMENT NO. 88

TO FACILITY OPERATING LICENSE NO. NPF-2

DOCKET NO. 50-348

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages. The revised areas are indicated by marginal lines.

Remove Pages	<u>Insert Pages</u>
3/4 7-20	3/4 7-20
3/4 7-20a	
3/4 7-21	3/4 7-21
3/4 7-22	3/4 7-22
3/4 7-23	3/4 7-23
3/4 7-24	3/4 7-24
3/4 7-25	3/4 7-25
~ ~	3/4 7-26
	3/4 7-27

3/4.7.9 SNUBBERS

LIMITING CONDITION FOR OPERATION

3.7.9 All snubbers shall be OPERABLE. The only snubbers excluded from this requirement are those installed on nonsafety-related systems and then only if their failure or the failure of the system on which they are installed would have no adverse effect on any safety-related system.

APPLICABILITY: MODES 1, 2, 3 and 4. (MODES 5 and 6 for snubbers located on systems required OPERABLE in those MODES).

ACTION:

With one or more snubbers inoperable, within 72 hours replace or restore the inoperable snubber(s) to OPERABLE status and perform an engineering evaluation per Specification 4.7.9.d on the supported component or declare the supported system inoperable and follow the appropriate ACTION statement for that system.

SURVEILLANCE REQUIREMENTS

4.7.9 Each snubber shall be demonstrated OPERABLE by the performance of the following augmented inservice inspection program in addition to the requirements of Specification 4.0.5.

- a. <u>Inspection Types</u>
 As used in this specification, "type of snubber" shall mean snubbers of the same design and manufacturer, irrespective of capacity.
- b. Visual Inspections
 Snubbers are categorized as inaccessible or accessible during reactor operation. Each of these categories (inaccessible and accessible) may be inspected independently according to the schedule determined by Table 4.7-3. The visual inspection interval for each category of snubber shall be determined based upon the criteria provided in Table 4.7-3 and the first inspection interval determined using this criteria shall be based upon the previous inspection interval as established by the requirements in effect before Amendment No. 88.

AMENDMENT NO. \$3, \$9,

Table 4.7-3

SNUBBER VISUAL INSPECTION INTERVAL

	NUMB	ER OF UNACCEPTABLE S	SNUBBERS
Population	Column A	Column B	Column C
or Category	Extend Interval	Repeat Interval	Reduce Interval
(Notes 1 and 2)	(Notes 3 and 6)	(Notes 4 and 6)	(Notes 5 and 6)
1	0	0	1
80	0	0	2
100	0	1	4
150	0	3	8
200	2	5	13
300	5	12	25
400	8	18	36
500	12	24	48
750	20	40	78
1000 or greater	29	56	109

TABLE NOTATION

Note 1: The next visual inspection interval for a snubber population or category size shall be determined based upon the previous inspection interval and the number of unacceptable snubbers found during that interval. Snubbers may be categorized, based upon their accessibility during power operation, as accessible or inaccessible. These categories may be examined separately or jointly. However, the licensee must make and document that decision before any inspection and shall use that decision as the basis upon which to determine the next inspection interval for that category.

(Notes continued on Page 3/4 7-22)

Table 4.7-3 (Continued)

TABLE NOTATION

- Note 2: Interpolation between population or category sizes and the number of unacceptable snubbers is permissible. Use next lower integer for the value of the limit for Columns A, B, or C if that integer includes a fractional value of unacceptable snubbers as determined by interpolation.
- Note 3: If the number of unacceptable snubbers is equal to or less than the number in Column A, the next inspection interval may be twice the previous interval but not greater than 48 months.
- Note 4: If the number of unacceptable snubbers is equal to or less than the number in Column B but greater than the number in Column A, the next inspection interval shall be the same as the previous interval.
- Note 5: If the number of unacceptable snubbers is equal to or greater than the number in Column C, the next inspection interval shall be two-thirds of the previous interval. However, if the number of unacceptable snubbers is less than the number in Column C but greater than the number in Column B, the next interval shall be reduced proportionally by interpolation, that is, the previous interval shall be reduced by a factor that is one-third of the ratio of the difference between the number of unacceptable snubbers found during the previous interval and the number in Column B to the difference in the numbers in Columns B and C.
- Note 6: The provisions of Specification 4.0.2 are applicable for all inspection intervals up to and including 48 months.

c. <u>Visual Inspection Acceptance Criteria</u>

Visual inspections shall verify that (1) the snubber has no visible indications of damage or impaired OPERABILITY, (2) attachments to the foundation or supporting structure are functional, and (3) fasteners for the attachment of the snubber to the component and to the snubber anchorage are functional. Snubbers which appear inoperable as a result of visual inspections shall be classified as unacceptable and may be reclassified acceptable for the purpose of establishing the next visual inspection interval, provided that (i) the cause of the rejection is clearly established and remedied for that particular snubber and for other snubbers irrespective of type that may be generically susceptible; and (ii) the affected snubber is functionally tested in the as-found condition and determined OPERABLE per Specifications 4.7.9.e or 4.7.9.f. All snubbers found connected to an inoperable common hydraulic fluid reservoir shall be counted as unacceptable and may be reclassified as acceptable for determining the next inspection interval provided that criterion (i) and (ii) above are met. A review and evaluation shall be performed and documented to justify continued operation with an unacceptable snubber. If continued operation cannot be justified, the snubber shall be declared inoperable and the ACTION requirements shall be met.

d. <u>Functional tests</u>

At least once per 18 months during shutdown, a representative sample of 88 snubbers shall be functionally tested either in place or in a bench test. If more than 3 snubbers do not meet the functional test acceptance criteria of Specifications 4.7.9.e or 4.7.9.f, an additional sample selected according to the expression 22(a-3) shall be functionally tested, where a is the total number of snubbers found inoperable during the functional testing of the initial representative sample.

Functional testing shall continue according to the expression (22)b where b is the number of snubbers found inoperable in the previous re-sample, until no additional inoperable snubbers are found within a sample or until all snubbers have been functionally tested.

Snubbers greater than 50,000 lb. capacity may not be excluded from functional testing requirements.*

^{*} This portion of the specification is not effective until the fifth refueling outage or when a commercial in-place testing device is available whichever is later.

SURVEILLANCE REQUIREMENTS (Continued)

The representative sample selected for functional testing shall include the various configurations, operating environments and the range of size and capacity of snubbers. At least 25% of the snubbers in the initial representative sample shall include snubbers from the following three groups:

- 1. The first snubber away from each reactor vessel nozzle
- 2. Snubbers within five feet of heavy equipment (valve, pump, turbine, motor, etc.)
- 3. Snubbers within 10 feet of the discharge from a safety relief valve.

Snubbers that are especially difficult to remove or in high radiation zones during shutdown shall also be included in the representative sample.* Hydraulic and mechanical snubbers may be used jointly or separately as the basis for the sampling plan.

In addition to the regular sample, snubbers which failed the previous functional test shall be retested during the next test period. If a spare snubber has been installed in place of a failed snubber, then both the failed snubber (if it is repaired and installed in another position) and the spare snubber shall be retested. Test results of these snubbers may not be included for the re-sampling.

If any snubber selected for functional testing either fails to lockup or fails to move, i.e., frozen in place, the cause will be evaluated and if caused by manufacturer or design deficiency all snubbers of the same design subject to the same defect shall be functionally tested. This testing requirement shall be independent of the requirements stated above for snubbers not meeting the functional test acceptance criteria.

^{*} Permanent or other exemptions from functional testing for individual snubbers in these categories may be granted by the Commission only if a justifiable basis for exemption is presented and/or snubber life destructive testing was performed to qualify snubber operability for all design conditions at either the completion of their fabrication or at a subsequent date.

SURVEILLANCE REQUIREMENTS (Continued)

For the snubber(s) found inoperable, an engineering evaluation shall be performed on the components which are supported by the snubber(s). The purpose of this engineering evaluation shall be to determine if the components supported by the snubber(s) were adversely affected by the inoperability of the snubber(s) in order to ensure that the attached component remains capable of meeting the designed service.

e. <u>Hydraulic Snubbers Functional Test Acceptance Criteria</u>

The hydraulic snubber functional test shall verify that:

- 1. Activation (restraining action) is achieved within the specified range of velocity or acceleration in both tension and compression.
- 2. Snubber bleed, or release rate, where required, is within the specified range in compression or tension. For snubbers specifically required to not displace under continuous load, the ability of the snubber to withstand load without displacement shall be verified.

f. Mechanical Snubbers Functional Test Acceptance Criteria

The mechanical snubber functional test shall verify that:

- 1. The force that initiates free movement of the snubber rod in either tension or compression is less than the specified maximum drag force.
- 2. Activation (restraining action) is achieved within the specified range in both tension and compression.
- 3. Snubber release rate, where required, is within the specified range in compression or tension. For snubbers specifically required not to displace under continuous load, the ability of the snubber to withstand load without displacement shall be verified.

SURVEILLANCE REQUIREMENTS (Continued)

g. Snubber Service Life Monitoring

A record of the service life of each snubber, the date at which the designated service life commences and the installation and maintenance records on which the designated service life is based shall be maintained as required by Specification 6.10.2.n.

Concurrent with the first inservice visual inspection and at least once per 18 months, the installation and maintenance records for each snubber shall be reviewed to verify that the indicated service life has not been exceeded or will not be exceeded prior to the next scheduled snubber service life review. If the indicated service life will be exceeded prior to the next scheduled snubber service life review, the snubber service life shall be reevaluated or the snubber shall be replaced or reconditioned so as to extend its service life beyond the date of the next scheduled service life review. This reevaluation, replacement or reconditioning shall be indicated in the records.

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UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

ALABAMA POWER COMPANY

DOCKET NO. 50-364

JOSEPH M. FARLEY NUCLEAR PLANT, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 82 License No. NPF-8

- 1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Alabama Power Company (the licensee), dated February 6, 1991, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
- 2. Accordingly, the license is amended by changes to the Technical Specifications, as indicated in the attachment to this license amendment; and paragraph 2.C.(2) of Facility Operating License No. NPF-8 is hereby amended to read as follows:

(2) <u>Technical Specifications</u>

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 82, are hereby incorporated in the license. Alabama Power Company shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION
Orignal Signed By:

Elinor G. Adensam, Director Project Directorate II-1 Division of Reactor Projects - I/II Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical Specifications

Date of Issuance: March 19, 1991

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ATTACHMENT TO LICENSE AMENDMENT NO. 82

TO FACILITY OPERATING LICENSE NO. NPF-8

DOCKET NO. 50-364

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages. The revised areas are indicated by marginal lines.

Remove Pages	Insert Pages
3/4 7-20	3/4 7-20
3/4 7-20a	
3/4 7-21	3/4 7-21
3/4 7-22	3/4 7-22
3/4 7-23	3/4 7-23
3/4 7-24	3/4 7-24
3/4 7-25	3/4 7-25
	3/4 7-26
	3/4 7-27

3/4.7.9 **SNUBBERS**

LIMITING CONDITION FOR OPERATION

3.7.9 All snubbers shall be OPERABLE. The only snubbers excluded from this requirement are those installed on nonsafety-related systems and then only if their failure or the failure of the system on which they are installed would have no adverse effect on any safety-related system.

<u>APPLICABILITY:</u> MODES 1, 2, 3 and 4. (MODES 5 and 6 for snubbers located on systems required OPERABLE in those MODES).

ACTION:

With one or more snubbers inoperable, within 72 hours replace or restore the inoperable snubber(s) to OPERABLE status and perform an engineering evaluation per Specification 4.7.9.d on the supported component or declare the supported system inoperable and follow the appropriate ACTION statement for that system.

SURVEILLANCE REQUIREMENTS

- 4.7.9 Each snubber shall be demonstrated OPERABLE by the performance of the following augmented inservice inspection program in addition to the requirements of Specification 4.0.5.
 - a. <u>Inspection Types</u> As used in this specification, "type of snubber" shall mean snubbers of the same design and manufacturer, irrespective of capacity.
 - Snubbers are categorized as inaccessible or accessible during reactor operation. Each of these categories (inaccessible and accessible) may be inspected independently according to the schedule determined by Table 4.7-3. The visual inspection interval for each category of snubber shall be determined based upon the criteria provided in Table 4.7-3 and the first inspection interval determined using this criteria shall be based upon the previous inspection interval as established by the requirements in effect before Amendment No. 82.

Table 4.7-3

SNUBBER VISUAL INSPECTION INTERVAL

	NUMB	SNUBBERS		
Population	Column A	Column B	Column C	
or Category	Extend Interval	Repeat Interval	Reduce Interval	
(Notes 1 and 2)	(Notes 3 and 6)	(Notes 4 and 6)	(Notes 5 and 6)	
1	0	0	1	
80	0	0	2	
100	0	1	4	
150	0	3	8	
	•	3	6	
200	2	5	13	
300	5	12	25	
400	8	18	36	
500	12	24	48	
750	20	40	78	
1000 or greater	29	56	109	

TABLE NOTATION

Note 1: The next visual inspection interval for a snubber population or category size shall be determined based upon the previous inspection interval and the number of unacceptable snubbers found during that interval. Snubbers may be categorized, based upon their accessibility during power operation, as accessible or inaccessible. These categories may be examined separately or jointly. However, the licensee must make and document that decision before any inspection and shall use that decision as the basis upon which to determine the next inspection interval for that category.

(Notes continued on page 3/4 7-22)

Table 4.7-3 (Continued)

TABLE NOTATION

- Note 2: Interpolation between population or category sizes and the number of unacceptable snubbers is permissible. Use next lower integer for the value of the limit for Columns A, B, or C if that integer includes a fractional value of unacceptable snubbers as determined by interpolation.
- Note 3: If the number of unacceptable snubbers is equal to or less than the number in Column A, the next inspection interval may be twice the previous interval but not greater than 48 months.
- Note 4: If the number of unacceptable snubbers is equal to or less than the number in Column B but greater than the number in Column A, the next inspection interval shall be the same as the previous interval.
- Note 5: If the number of unacceptable snubbers is equal to or greater than the number in Column C, the next inspection interval shall be two-thirds of the previous interval. However, if the number of unacceptable snubbers is less than the number in Column C but greater than the number in Column B, the next interval shall be reduced proportionally by interpolation, that is, the previous interval shall be reduced by a factor that is one-third of the ratio of the difference between the number of unacceptable snubbers found during the previous interval and the number in Column B to the difference in the numbers in Columns B and C.
- Note 6: The provisions of Specification 4.0.2 are applicable for all inspection intervals up to and including 48 months.

c. <u>Visual Inspection Acceptance Criteria</u>

Visual inspections shall verify that (1) the snubber has no visible indications of damage or impaired OPERABILITY, (2) attachments to the foundation or supporting structure are functional, and (3) fasteners for the attachment of the snubber to the component and to the snubber anchorage are functional. Snubbers which appear inoperable as a result of visual inspections shall be classified as unacceptable and may be reclassified acceptable for the purpose of establishing the next visual inspection interval, provided that (i) the cause of the rejection is clearly established and remedied for that particular snubber and for other snubbers irrespective of type that may be generically susceptible; and (ii) the affected snubber is functionally tested in the as-found condition and determined OPERABLE per Specifications 4.7.9.e or 4.7.9.f. All snubbers found connected to an inoperable common hydraulic fluid reservoir shall be counted as unacceptable and may be reclassified as acceptable for determining the next inspection interval provided that criterion (i) and (ii) above are met. A review and evaluation shall be performed and documented to justify continued operation with an unacceptable snubber. continued operation cannot be justified, the snubber shall be declared inoperable and the ACTION requirements shall be met.

d. Functional tests

At least once per 18 months during shutdown, a representative sample of 88 snubbers shall be functionally tested either in place or in a bench test. If more than 3 snubbers do not meet the functional test acceptance criteria of Specifications 4.7.9.e or 4.7.9.f, an additional sample selected according to the expression 22(a-3) shall be functionally tested, where a is the total number of snubbers found inoperable during the functional testing of the initial representative sample.

Functional testing shall continue according to the expression (22)b where b is the number of snubbers found inoperable in the previous re-sample, until no additional inoperable snubbers are found within a sample or until all snubbers have been functionally tested.

Snubbers greater than 50,000 lb. capacity may not be excluded from functional testing requirements.*

^{*} This portion of the specification is not effective until the second refueling outage or when a commercial in-place testing device is available whichever is later.

SURVEILLANCE REQUIREMENTS (Continued)

The representative sample selected for functional testing shall include the various configurations, operating environments and the range of size and capacity of snubbers. At least 25% of the snubbers in the initial representative sample shall include snubbers from the following three groups:

- 1. The first snubber away from each reactor vessel nozzle
- Snubbers within five feet of heavy equipment (valve, pump, turbine, motor, etc.)
- 3. Snubbers within 10 feet of the discharge from a safety relief valve.

Snubbers that are especially difficult to remove or in high radiation zones during shutdown shall also be included in the representative sample.* Hydraulic and mechanical snubbers may be used jointly or separately as the basis for the sampling plan.

In addition to the regular sample, snubbers which failed the previous functional test shall be retested during the next test period. If a spare snubber has been installed in place of a failed snubber, then both the failed snubber (if it is repaired and installed in another position) and the spare snubber shall be retested. Test results of these snubbers may not be included for the re-sampling.

If any snubber selected for functional testing either fails to lockup or fails to move, i.e., frozen in place, the cause will be evaluated and if caused by manufacturer or design deficiency all snubbers of the same design subject to the same defect shall be functionally tested. This testing requirement shall be independent of the requirements stated above for snubbers not meeting the functional test acceptance criteria.

^{*} Permanent or other exemptions from functional testing for individual snubbers in these categories may be granted by the Commission only if a justifiable basis for exemption is presented and/or snubber life destructive testing was performed to qualify snubber operability for all design conditions at either the completion of their fabrication or at a subsequent date.

SURVEILLANCE REQUIREMENTS (Continued)

For the snubber(s) found inoperable, an engineering evaluation shall be performed on the components which are supported by the snubber(s). The purpose of this engineering evaluation shall be to determine if the components supported by the snubber(s) were adversely affected by the inoperability of the snubber(s) in order to ensure that the attached component remains capable of meeting the designed service.

e. <u>Hydraulic Snubbers Functional Test Acceptance Criteria</u>

The hydraulic snubber functional test shall verify that:

- 1. Activation (restraining action) is achieved within the specified range of velocity or acceleration in both tension and compression.
- 2. Snubber bleed, or release rate, where required, is within the specified range in compression or tension. For snubbers specifically required to not displace under continuous load, the ability of the snubber to withstand load without displacement shall be verified.

f. Mechanical Snubbers Functional Test Acceptance Criteria

The mechanical snubber functional test shall verify that:

- 1. The force that initiates free movement of the snubber rod in either tension or compression is less than the specified maximum drag force.
- 2. Activation (restraining action) is achieved within the specified range in both tension and compression.
- 3. Snubber release rate, where required, is within the specified range in compression or tension. For snubbers specifically required not to displace under continuous load, the ability of the snubber to withstand load without displacement shall be verified.

SURVEILLANCE REQUIREMENTS (Continued)

g. Snubber Service Life Monitoring

A record of the service life of each snubber, the date at which the designated service life commences and the installation and maintenance records on which the designated service life is based shall be maintained as required by Specification 6.10.2.n.

Concurrent with the first inservice visual inspection and at least once per 18 months thereafter, the installation and maintenance records for each snubber shall be reviewed to verify that the indicated service life has not been exceeded or will not be exceeded prior to the next scheduled snubber service life review. If the indicated service life will be exceeded prior to the next scheduled snubber service life review, the snubber service life shall be reevaluated or the snubber shall be replaced or reconditioned so as to extend its service life beyond the date of the next scheduled service life review. This reevaluation, replacement or reconditioning shall be indicated in the records.

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UNITED STATES NUCLEAR REGULATORY COMMISSION

WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION SUPPORTING AMENDMENT NO. 88 TO FACILITY OPERATING LICENSE NO. NPF-2 AND AMENDMENT NO. 82 TO FACILITY OPERATING LICENSE NO. NPF-8

ALABAMA POWER COMPANY

DOCKET NOS. 50-348 AND 50-364

1.0 INTRODUCTION

By letter dated February 6, 1991, Alabama Power Company (the licensee) proposed changes to the Technical Specifications (TS) for the Joseph M. Farley Nuclear Plant, Units 1 and 2. The amendments propose removing the snubber visual examination schedule in the existing TS and replacing it with a refueling outage based visual examination schedule as shown in Table 4.7-2, "Snubber Visual Inspection Interval," of Enclosure B to Generic Letter 90-09.

2.0 EVALUATION

The snubber visual examination schedule in the existing TS is based on the permissible number of inoperable snubbers found during the visual examination. Because the existing snubber visual examination schedule is based only on the absolute number of inoperable snubbers found during the visual examinations irrespective of the total population of snubbers, a large snubber population results in a visual examination schedule that is excessively restrictive. The purpose of the alternative visual examination schedule is to allow the licensee to perform visual examinations and corrective actions during plant outages without reducing the confidence level provided by the existing visual examination schedule. The new visual examination schedule specifies the permissible number of inoperable snubbers for various snubber populations. The basic examination interval is the normal fuel cycle up to 24 months. This interval may be extended to as long as twice the fuel cycle or reduced to as small as two-thirds of the fuel cycle depending on the number of unacceptable snubbers found during the visual examination. The examination interval may vary by ±25 percent to coincide with the actual outage.

In the event one or more snubbers are found inoperable during a visual examination, the Limiting Conditions for Operation (LCO) in the present TS require the licensee to repair or replace the inoperable snubber(s)

within 72 hours or declare the attached system inoperable and follow the appropriate action statement for the system. This LCO will remain in the TS; however, the permissible number of inoperable snubber(s) and the subsequent visual examination interval will now be determined in accordance with the new visual examination schedule in TS Table 4.7-3 (consistent with Table 4.7-2 of Enclosure B to Generic Letter 90-09, dated December 11, 1990). As noted in the guidance for this line item TS improvement, certain corrective actions may have to be performed depending on the number of inoperable snubbers found. All requirements for corrective actions and evaluations associated with the use of the visual examination schedule, as stated in footnotes 1 through 6 of Table 4.7-2 of Enclosure B to Generic Letter 90-09, were added to the TS.

The licensee has proposed changes to TS 4.7.9 for the replacement of the snubber visual examination schedule that are consistent with the guidance provided in Generic Letter 90-09. On the basis of the staff's review, the proposed changes to the TS for Joseph M. Farley Nuclear Plant, Units 1 and 2, are acceptable.

3.0 ENVIRONMENTAL CONSIDERATION

These amendments change a requirement with respect to installation or use of a facility component located within the restricted areas as defined in 10 CFR Part 20 and change the surveillance requirements. The staff has determined that these amendments involve no significant increase in the amounts, and no significant change in the types of any effluents that may be released off site, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that these amendments involve no significant hazards consideration, and there has been no public comment on such finding. Accordingly, these amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of these amendments.

4.0 CONCLUSION

The Commission made a proposed determination that this amendment involves no significant hazards consideration which was published in the Federal Register (56 FR 6417) on February 15, 1991, and consulted with the State of Alabama. No public comments or requests for hearing were received, and the State of Alabama did not have any comments.

The staff has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

Dated: March 19, 1991

Principal Contributor: J. Rajan