November 8, 2001

Mr. S. K. Gambhir
Division Manager - Nuclear Operations
Omaha Public Power District
Fort Calhoun Station FC-2-4 Adm.
Post Office Box 399
Hwy. 75 - North of Fort Calhoun
Fort Calhoun, NE 68023-0399

SUBJECT: BULLETIN 2001-01, "CIRCUMFERENTIAL CRACKING OF REACTOR PRESSURE VESSEL HEAD PENETRATION NOZZLES" - RESPONSES FOR FORT CALHOUN STATION (TAC NO. MB2631)

Dear Mr. Gambhir :

On August 3, 2001, the NRC staff issued Bulletin 2001-01, "Circumferential Cracking of Reactor Pressure Vessel Head Penetration Nozzles," to the industry requesting that addressees provide information related to the structural integrity of the reactor pressure vessel head penetration (VHP) nozzles for their respective facilities, including the extent of VHP nozzle leakage and cracking that has been found to date, the inspections and repairs that have been undertaken to satisfy applicable regulatory requirements, and the basis for concluding that their plans for future inspections will ensure compliance with applicable regulatory requirements at their respective pressurized water reactor (PWR) plants. You were requested to respond to Items 1 and 4 of the Bulletin within 30 days of its issuance.

You provided your response by letters dated August 31, 2001, and October 19, 2001. The NRC staff finds that you have provided the requested information and there is reasonable assurance that the public health and safety will be maintained. Your proposed inspection scope and schedule described in your responses, which included the description of your plan to examine "all" or "100%" of the VHPs at your facility during the next scheduled outage, were integral to the staff's finding. It is the staff's expectation that you will submit a revised response to the Bulletin if you make any substantive changes to the schedule and/or scope of future inspections for your plant. If warranted by such changes, the staff will reevaluate this issue for the Fort Calhoun Station.

Addressees are reminded that Item 5 of the Bulletin requested the following information within 30 days after plant restart following the next refueling outage:

a. A description of the extent of VHP nozzle leakage and cracking detected at your plant, including the number, location, size, and nature of each crack detected; and

b. If cracking is identified, a description of the inspections (type, scope, qualification requirements, and acceptance criteria), repairs, and other corrective actions you have taken to satisfy applicable regulatory requirements. This information is requested only if there are any changes from prior information submitted in accordance with this Bulletin.

If you have any questions regarding this matter, I may be reached at (301) 415-1445.

Sincerely,

/RA/

Alan B. Wang, Project Manager, Section 2
Project Directorate IV
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket No. 50-285

cc: See next page

If you have any questions regarding this matter, I may be reached at (301) 415-1445.

Sincerely,

/RA/

Alan B. Wang, Project Manager, Section 2
Project Directorate IV
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket No. 50-285

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