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Docket No. 50-348

Mr. F. L. Clayton
Senior Vice President
Alabama Power Company
Post Office Box 2641
Birmingham, Alabama 35291

Dear Mr. Clayton:

The Commission has issued the enclosed Amendment No. 29 to Facility Operating License No. NPF-2 for the Joseph M. Farley Nuclear Plant, Unit No. 1. The amendment consists of changes to the Technical Specifications in response to your application transmitted by telecopy and confirmed by letter dated January 14, 1983.

The amendment modifies the Technical Specification Action Statement to extend from 30 hours to 96 hours for one time only the time allowed to place the unit in cold shutdown for the start of the fourth refueling outage.

This amendment supports our letter to you dated January 17, 1983 wherein we confirmed our telephone authorization to you on January 14, 1983 granting the one time only extension.

Copies of the Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

ORIGINAL SIGNED

Edward A. Reeves, Project Manager
Operating Reactors Branch #1
Division of Licensing

Enclosures:
1. Amendment No. 29 to NPF-2
2. Safety Evaluation
3. Notice of Issuance

cc w/enclosures:
See next page

OFFICE	ORB#1:DL	ORB#1:DL	ORB#1:DL	AD/OR:DL	OELD		
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Mr. F. L. Clayton
Alabama Power Company

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

ALABAMA POWER COMPANY

DOCKET NO. 50-348

JOSEPH M. FARLEY NUCLEAR PLANT, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 29
License No. NPF-2

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Alabama Power Company (the licensee) dated January 14, 1983, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-2 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 29, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment was effective on January 14, 1983.

FOR THE NUCLEAR REGULATORY COMMISSION

Steven A. Varga, Chief
Operating Reactors Branch #1
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: February 17, 1983

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 29 TO FACILITY OPERATING LICENSE NO. NPF-2

DOCKET NO. 50-348

Revise Appendix A as follows:

Remove Page*	Insert Page*
3/4 6-1	3/4 6-1

*NOTE: After reaching cold shutdown conditions at the start of fourth refueling outage, the removed page 3/4 6-1 will be reinserted and this insert page 3/4 6-1 is cancelled.

3/4.6 CONTAINMENT SYSTEMS

3/4.6.1 PRIMARY CONTAINMENT

CONTAINMENT INTEGRITY

LIMITING CONDITION FOR OPERATION

3.6.1.1 Primary CONTAINMENT INTEGRITY shall be maintained.

APPLICABILITY: MODES 1, 2, 3 and 4.

ACTION:

Without primary CONTAINMENT INTEGRITY, restore CONTAINMENT INTEGRITY within one hour or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 96**hours.

SURVEILLANCE REQUIREMENTS

4.6.1.1 Primary CONTAINMENT INTEGRITY shall be demonstrated:

a. At least once per 31 days by verifying that all penetrations* not capable of being closed by OPERABLE containment automatic isolation valves and required to be closed during accident conditions are closed by valves, blind flanges, or deactivated automatic valves secured in their positions, except as provided in Table 3.6-1 of Specification 3.6.3.1.

b. By verifying that each containment air lock is OPERABLE per Specification 3.6.1.3.

c. After each closing of each penetration subject to Type B testing, if opened following a Type A or B test, by leak rate testing the seal with gas at P_a (48 psig) and verifying that when the measured leakage rate for these seals is added to the leakage rates determined pursuant to Specification 4.6.1.2.d for all other Type B and C penetrations, the combined leakage rate is less than or equal to 0.60 L_a.

*Except valves, blind flanges, deactivated automatic valves and the equipment hatch which are located inside the containment and are locked, sealed or otherwise secured in the closed position. These penetrations shall be verified closed during each COLD SHUTDOWN except that such verification need not be performed more often than once per 92 days. The blind flange on the fuel transfer canal flange shall be verified closed after each draining of the canal.

**One-time only change during shutdown for the fourth refueling outage.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 29 TO FACILITY OPERATING LICENSE NO. NPF-2

ALABAMA POWER COMPANY

JOSEPH M. FARLEY NUCLEAR PLANT, UNIT NO. 1

DOCKET NO. 50-348

Introduction

By letter dated January 14, 1983, which confirmed an Alabama Power Company (APCo) telecopy request of the same date, APCo requested a one-time change to License No. NPF-2 for Farley Unit 1. The change would allow a one-time extension to the Technical Specification 3.6.1.1 Action Statement required time to achieve cold shutdown from 30 hours to 96 hours.

Background

Technical Specification (TS) 3.6.1.1 requires maintenance of containment integrity while in Modes 1, 2, 3 and 4. Technical Specification 4.6.1.2.d requires tests of containment penetrations at intervals no greater than 24 months. During tests of electrical penetrations prior to the shutdown from the fourth refueling outage scheduled at midnight on January 14, 1983 excessive leakage was found in one electrical penetration. The leakage required APCo to declare the penetration inoperable. Thus, Action Statement for TS 3.6.1.1 required restoration of containment integrity within one hour or be in at least hot standby within the next 6 hours and in cold shutdown within the following 30 hours.

On this basis, APCo decided to shutdown for the fourth refueling outage prior to midnight as scheduled. However, the TS requirement to achieve cold shutdown within 30 hours created schedular conflicts for tests to be done with the reactor coolant system (RCS) pressurized. To maintain the RCS pressurized requires keeping the plant at temperatures above cold shutdown. Therefore, APCo proposed a one-time change to the 30 hour time limit on an emergency basis. Our evaluation follows.

Discussion and Evaluation

The Standard Technical Specifications Action Statements require placing the plant in cold shutdown within 30 hours. Otherwise, the RCS energy due to coolant temperatures above 200°F would create a potential release of radioactive materials beyond the leak rates assumed in the accident analyses. The 30 hours

is easily achievable using normal pressurized water reactor procedures. However, in the case as described by APCo, other valid reasons existed to make it desirable to maintain the RCS temperature and pressure above cold shutdown conditions for sometime longer than the normal 30 hours.

Alabama Power Company described the reasons why the one-time change from 30 hours to 96 hours would not involve any appreciable increased risk to the health and safety of the public. Reasons given by APCo were as follows:

(1) The containment electrical penetration in question has both an inner and outer seal. Tests show that only one seal was leaking. Thus, physical containment integrity still exists with one good seal.

(2) Reduced potential of offsite radiological exposure.

(3) Small probability of an accident during the short time extension from 30 to 96 hours.

(4) The reactor is subcritical with all full length control rods inserted.

Later during discussions with the APCo staff, item (2) above was explained as being a reference to onsite personnel exposures which would be less if the extension was granted. With this clarification, we agree with APCo stated reasons.

Therefore, based on the considerations noted above, we concluded that there was no undue risk to the health and safety of the public by this action. Our action at this time is only for record purposes to document in the license the authorization previously given verbally on January 14, 1983 and documented in our letter dated January 17, 1983.

Environmental Consideration

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of an accident previously evaluated, does not create the possibility of an accident of a type different from any evaluated previously, and does not involve a significant reduction in a margin of safety, the amendment does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: February 17, 1983

Principal Contributor:
E. A. Reeves

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NO. 50-348

ALABAMA POWER COMPANY

NOTICE OF ISSUANCE OF AMENDMENT TO FACILITY OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 29 to Facility Operating License No. NPF-2 issued to Alabama Power Company (the licensee), which revised Technical Specifications for operation of the Joseph M. Farley Nuclear Plant, Unit No. 1 (the facility) located in Houston County, Alabama. The amendment was effective on January 14, 1983.

The amendment modifies the Technical Specification Action Statement to extend from 30 hours to 96 hours for one time only the time allowed to place the unit in cold shutdown for the start of the fourth refueling outage.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment was not required since this amendment does not involve a significant hazards consideration.

The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of this amendment.

For further details with respect to this action, see (1) the application for amendment dated January 14, 1983, (2) the Commission's letter dated January 17, 1983, (3) the Amendment No. 29 to License No. NPF-2, and (4) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D.C. and at the George S. Houston Memorial Library, 212 W. Burdeshaw Street, Dothan, Alabama 36303. A copy of items (2), (3) and (4) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 17th day of February, 1983.

FOR THE NUCLEAR REGULATORY COMMISSION

/Steven A. Varga, Chief
Operating Reactors Branch #1
Division of Licensing