

April 15, 1986

Docket Nos. 50-348
and 50-364

Mr. R. P. McDonald
Senior Vice President
Alabama Power Company
Post Office Box 2641
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Dear Mr. McDonald:

The Commission has issued the enclosed Amendment No.62 to Facility Operating License No. NPF-2 and Amendment No.53 to NPF-8 for the Joseph M. Farley Nuclear Plant, Unit Nos. 1 and 2, respectively. The amendments consist of changes to the Technical Specifications in response to your application transmitted by letter dated November 22, 1985.

The amendments modify Technical Specification (TS) Table 4.3-1 Note 9 to clarify that the turbine trip surveillance can be done prior to latching the turbine. The change is administrative in nature in that it corrects an unintentional error when Table 4.3-1 notations were developed.

A copy of our related Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's next regular bi-weekly Federal Register notice.

Sincerely,

/s/

Edward A. Reeves, Project Manager
PWR Project Directorate #2
Division of PWR Licensing-A

Enclosures:

1. Amendment No. 62 to NPF-2
2. Amendment No. 53 to NPF-8
3. Safety Evaluation

cc: w/enclosures:
See next page

LA: PAD#2
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3/25/86

PM: PAD#2
E. Reeves;bg
3/25/86

OELD: [Signature]
4/3/2/86

D: PAD#2 [Signature]
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4/1/86

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PDR ADOCK 05000348
PDR

Mr. R. P. McDonald
Alabama Power Company

Joseph M. Farley Nuclear Plant

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

ALABAMA POWER COMPANY

DOCKET NO. 50-348

JOSEPH M. FARLEY NUCLEAR PLANT, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 62
License No. NPF-2

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Alabama Power Company (the licensee) dated November 22, 1985, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, as amended, the provisions of the Act, and the regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-2 is hereby amended to read as follows:

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(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 62, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Lester S. Rubenstein, Director
PWR Project Directorate #2
Division of PWR Licensing-A

Attachment:
Changes to the Technical
Specifications

Date of Issuance: April 15, 1986

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 62 TO FACILITY OPERATING LICENSE NO. NPF-2

DOCKET NO. 50-348

Replace the following page of the Appendix "A" Technical Specifications with the enclosed page as indicated. The revised page is identified by amendment number and contains vertical lines indicating the area of change. The corresponding overleaf page is also provided to maintain document completeness.

Remove Page

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Insert Page

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TABLE 4.3-1 (Continued)

REACTOR TRIP SYSTEM INSTRUMENTATION SURVEILLANCE REQUIREMENTS

<u>FUNCTIONAL UNIT</u>	<u>CHANNEL CHECK</u>	<u>CHANNEL CALIBRATION</u>	<u>CHANNEL FUNCTIONAL TEST</u>	<u>MODES IN WHICH SURVEILLANCE REQUIRED</u>
13. Steam Generator Water Level-- Low-Low	S	R	M	1, 2
14. Steam/Feedwater Flow Mismatch and Low Steam Generator Water Level	S	R	M	1, 2
15. Undervoltage - Reactor Coolant Pumps	N.A.	R	M	1
16. Underfrequency - Reactor Coolant Pumps	N.A.	R	M	1
17. Turbine Trip				
A. Low Auto Stop Oil Pressure	N.A.	N.A.	S/U(9)(1)	N.A.
B. Turbine Throttle Valve Closure	N.A.	N.A.	S/U(9)(1)	N.A.
18. Safety Injection Input from ESF	N.A.	N.A.	M(4)	1, 2
19. Reactor Coolant Pump Breaker Position Trip	N.A.	N.A.	R	1
20. Reactor Trip System Interlocks	N.A.	R	S/U(8)	1
21. Reactor Trip Breaker	N.A.	N.A.	M(5) and S/U(1)	1, 2, and *
22. Automatic Trip Logic	N.A.	N.A.	M(5)	1, 2, and *

FARLEY-UNIT 1

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AMENDMENT NO. 26

TABLE 4.3-1 (Continued)

TABLE NOTATION

- * - With the reactor trip system breakers closed and the control rod drive system capable of rod withdrawal.
- (1) - If not performed in previous 7 days.
- (2) - Heat balance only, above 15% of RATED THERMAL POWER. Adjust channel if absolute difference greater than 2 percent.
- (3) - Compare incore to excore axial flux difference every 31 EFPD. Recalibrate if the absolute difference is greater than or equal to 3 percent.
- (4) - Manual ESF functional input check every 18 months.
- (5) - Each train or logic channel shall be tested at least every 62 days on a STAGGERED TEST BASIS.
- (6) - Neutron detectors may be excluded from CHANNEL CALIBRATION.
- (7) - Below the P-6 (Block of Source Range Reactor Trip) setpoint.
- (8) - Logic only, if not performed in previous 92 days.
- (9) - CHANNEL FUNCTIONAL TEST will consist of verifying that each channel indicates a turbine trip prior to latching the turbine and indicates no turbine trip prior to P-9.
- (10) - If not performed in the previous 31 days.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

ALABAMA POWER COMPANY

DOCKET NO. 50-364

JOSEPH M. FARLEY NUCLEAR PLANT, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 53
License No. NPF-8


1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Alabama Power Company (the licensee) dated November 22, 1985, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, as amended, the provisions of the Act, and the regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-2 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 53, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION


Lester S. Rubenstein, Director
PWR Project Directorate #2
Division of PWR Licensing-A

Attachment:
Changes to the Technical
Specifications

Date of Issuance: April 15, 1986

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 53 TO FACILITY OPERATING LICENSE NO. NPF-8

DOCKET NO. 50-364

Replace the following page of the Appendix "A" Technical Specifications with the enclosed page as indicated. The revised page is identified by amendment number and contains vertical lines indicating the area of change. The corresponding overleaf page is also provided to maintain document completeness.

Remove Page

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Insert Page

3/4 3-14

TABLE 4.3-1 (Continued)

REACTOR TRIP SYSTEM INSTRUMENTATION SURVEILLANCE REQUIREMENTS

<u>FUNCTIONAL UNIT</u>	<u>CHANNEL CHECK</u>	<u>CHANNEL CALIBRATION</u>	<u>CHANNEL FUNCTIONAL TEST</u>	<u>MODES IN WHICH SURVEILLANCE REQUIRED</u>
13. Steam Generator Water Level-- Low-Low	S	R	M	1, 2
14. Steam/Feedwater Flow Mismatch and Low Steam Generator Water Level	S	R	M	1, 2
15. Undervoltage - Reactor Coolant Pumps	N.A.	R	M	1
16. Underfrequency - Reactor Coolant Pumps	N.A.	R	M	1
17. Turbine Trip				
A. Low Auto Stop Oil Pressure	N.A.	N.A.	S/U(9)(1)	N.A.
B. Turbine Throttle Valve Closure	N.A.	N.A.	S/U(9)(1)	N.A.
18. Safety Injection Input from ESF	N.A.	N.A.	M(4)	1, 2
19. Reactor Coolant Pump Breaker Position Trip	N.A.	N.A.	R	1
20. Reactor Trip System Interlocks	N.A.	R	S/U(8)	1
21. Reactor Trip Breaker	N.A.	N.A.	M(5) and S/U(1)	1, 2, and *
22. Automatic Trip Logic	N.A.	N.A.	M(5)	1, 2, and *

FARLEY-UNIT 2

3/4 3-13

TABLE 4.3-1 (Continued)

TABLE NOTATION

- * - With the reactor trip system breakers closed and the control rod drive system capable of rod withdrawal.
- (1) - If not performed in previous 7 days.
- (2) - Heat balance only, above 15% of RATED THERMAL POWER. Adjust channel if absolute difference greater than 2 percent.
- (3) - Compare incore to excore axial flux difference every 31 EFPD. Recalibrate if the absolute difference is greater than or equal to 3 percent.
- (4) - Manual ESF functional input check every 18 months.
- (5) - Each train or logic channel shall be tested at least every 62 days on a STAGGERED TEST BASIS.
- (6) - Neutron detectors may be excluded from CHANNEL CALIBRATION.
- (7) - Below the P-6 (Block of Source Range Reactor Trip) setpoint.
- (8) - Logic only, if not performed in previous 92 days.
- (9) - CHANNEL FUNCTIONAL TEST will consist of verifying that each channel indicates a turbine trip prior to latching the turbine and indicates no turbine trip prior to P-9.
- (10) - If not performed in the previous 31 days.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 62 TO FACILITY OPERATING LICENSE NO. NPF-2
AND AMENDMENT NO. 53 TO FACILITY OPERATING LICENSE NO. NPF-8

ALABAMA POWER COMPANY

JOSEPH M. FARLEY NUCLEAR PLANT, UNIT NOS. 1 AND 2

DOCKET NOS. 50-348 AND 50-364

INTRODUCTION

By letter dated November 22, 1985, the licensee proposed a change to the Joseph M. Farley Technical Specifications to remove an ambiguity regarding a turbine trip surveillance which is a Channel Functional Test done prior to each reactor startup, if not performed in the previous 7 days. The channels involved are those initiated by a turbine trip. If a turbine trip occurs, a signal on 2 of 3 of the Low Auto Stop Oil Pressure channels, or a signal on 4 of 4 of the Turbine Throttle Valve Closure channels, initiates a reactor trip; provided that the P-9 setpoint has been reached or exceeded (approximately 50% thermal power or greater). The test would assure that no inadvertent (incorrect) signals remain on the channels that could cause a reactor trip when no condition exists that should cause a reactor trip. A change is proposed to allow the surveillance to be performed prior to latching the turbine.

EVALUATION

The licensee requests a change to Table Notation (9) of Functional Unit 17 of Technical Specification 4.3.1.1, Table 4.3-1. The existing Table 4.3-1 Notation (9) states:

Channel FUNCTIONAL TEST will consist of verifying that each channel indicates a turbine trip prior to latching the turbine and indicates no turbine trip after latching the turbine.

The licensee desires to replace the last four words "after latching the turbine" with the words "prior to P-9."

The licensee's justification is that a turbine trip does not initiate a reactor trip until after P-9. Thus, a spurious trip signal on one channel, or even on all channels, prior to P-9 would not trip the reactor.

Based on our review of licensee's proposed Technical Specifications and the Final Safety Analysis Report, as well as the licensee's justifications for revision of Table Notation (9), we agree with the licensee's statements justifying the proposed change in the Technical Specification.

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This clarification and change of the Technical Specification is acceptable because incorrect channel trip signals prior to P-9 would not result in a reactor trip. This fact alone is sufficient from a safety standpoint. Further, it is noted that even after P-9, an incorrect trip due to spurious trip signals on the channels involved is very unlikely. With regard to the Turbine Throttle Valve Closure channels, an incorrect turbine trip would require an incorrect signal on each of the four out of four channels. Similarly, an incorrect trip because of incorrect signals on the Low Auto Stop Oil Pressure channels would require incorrect trip signals on two out of three of the channels.

SAFETY SUMMARY AND CONCLUSION

The Technical Specification change proposed by the licensee is to clarify that the surveillance testing to check that there are no incorrect reactor trip signals on the Turbine Throttle Stop Valve channels and on the Low Auto Stop Oil Pressure channels need not be performed immediately after latching the turbine provided the tests are conducted prior to P-9 (approximately 50% thermal power). The proposed change has negligible safety significance because of the P-9 interlock which prevents a reactor trip prior to P-9. Essentially the change is administrative in nature and corrects an unintentional error made when the Table 4.3-1 notations were developed. Thus, we conclude that the proposed Technical Specification change is acceptable.

ENVIRONMENTAL CONSIDERATION

These amendments involve a change in the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously published a proposed finding that these amendments involve no significant hazards consideration and there has been no public comment on such finding. Accordingly, these amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR §51.22(c)(9). These amendments also involve changes in recordkeeping, reporting or administrative procedures or requirements. Accordingly, with respect to these items, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR §51.22(c)(10). Pursuant to 10 CFR §51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of these amendments.

CONCLUSION

We have concluded, based on the considerations discussed above, that:
(1) there is reasonable assurance that the health and safety of the public

will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Dated: April 15, 1986

Principal Contributor:

A. Toalston

E. A. Reeves