

OCT 14 1983

Distribution  
Docket File  
NRC PDR  
Local PDR  
ORB 1  
H Denton  
D Eisenhut  
C Parrish  
E Reeves  
OELD  
SECY  
L J Harmon  
E Jordan  
J Taylor  
T Barnhart (8)  
W Jones  
D Brinkman  
ACRS (10)  
OPA C Miles

Docket Nos. 50-348  
and 50-364

Mr. F. L. Clayton  
Senior Vice President  
Alabama Power Company  
Post Office Box 2641  
Birmingham, Alabama 35291

Dear Mr. Clayton:

The Commission has issued the enclosed Amendment No. 33 to Facility Operating License No. NPF-2 and Amendment No. 26 to NPF-8 for the Joseph M. Farley Nuclear Plant, Unit Nos. 1 and 2, respectively. The amendments consist of changes to the Technical Specifications in response to your application transmitted by letter dated November 16, 1982 supplemented by letters dated February 1, 1983 and June 29, 1983.

The amendments consist of administrative changes to the Technical Specifications for Administrative Controls for Onsite and Offsite Organization, Environmental Qualifications Audits; LCO's and Surveillance Instrumentation, Fire Protection and Yard Fire Hydrants.

We have made modifications to T.S. 4.8.2.5.2 for each unit and minor editorial corrections to T.S. 7.7.11.5 table 3.7-7 for Unit No. 2 and TS 3.3.3.7 Table 3.3.10 for both units. We have discussed these modifications with members of your staff.

A copy of the related Safety Evaluation is also enclosed.

Sincerely,

Edward A. Reeves, Project Manager  
Operating Reactors Branch No. 1  
Division of Licensing

*Transmitted  
before  
issue  
check for  
return of  
amendment,  
if any  
concerned  
to eth*

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Enclosures:

1. Amendment No. 33 to NPF-2
2. Amendment No. 26 to NPF-8
3. Safety Evaluation

cc w/enclosures  
see next page

OFFICE	ORB#1:DL	ORB#L:DL	C-ORB#1:DL	AD/OR:DL	OELD	D:DL	
SURNAME	CPARRISH	EReeves:ef	SVarda	Glainas		DEisenhut	
DATE	10/14/83	10/14/83	10/14/83	10/14/83	10/ /83	10/ /83	

Mr. F. L. Clayton  
Alabama Power Company

cc: Mr. W. O. Whitt  
Executive Vice President  
Alabama Power Company  
Post Office Box 2641  
Birmingham, Alabama 35291

Ruble A. Thomas, Vice President  
Southern Company Services, Inc.  
Post Office Box 2625  
Birmingham, Alabama 35202

George F. Trowbridge, Esquire  
Shaw, Pittman, Potts and Trowbridge  
1800 M Street, N.W.  
Washington, D. C. 20036

Chairman  
Houston County Commission  
Dothan, Alabama 36301

Robert A. Buettner, Esquire  
Balch, Bingham, Baker, Hawthorne,  
Williams and Ward  
Post Office Box 306  
Birmingham, Alabama 35201

Resident Inspector  
U. S. Nuclear Regulatory Commission  
Post Office Box 24-Route 2  
Columbia, Alabama 36319

State Department of Public Health  
ATTN: State Health Officer  
State Office Building  
Montgomery, Alabama 36104

Regional Radiation Representatives  
EPA Region IV  
345 Courtland Street, N.E.  
Atlanta, Georgia 30308

D. Biard MacGuineas, Esquire  
Volpe, Boskey and Lyons  
918 16th Street, N.W.  
Washington, D.C. 20006

Charles R. Lowman  
Alabama Electric Corporation  
P.O. Box 550  
Andalusia, Alabama 36420

Mr. R. P. McDonald  
Vice President - Nuclear Generation  
Alabama Power Company  
P.O. Box 2641  
Birmingham, Alabama 35291

James P. O'Reilly  
Regional Administrator - Region II  
U. S. Nuclear Regulatory Commission  
101 Marietta Street, Suite 3100  
Atlanta, Georgia 30303

UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555



ALABAMA POWER COMPANY

DOCKET NO. 50-348

JOSEPH M FARLEY NUCLEAR PLANT, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 33  
License No. NPF-2

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Alabama Power Company (the licensee) dated November 16, 1982 supplemented by letters dated February 1, 1983 and June 29, 1983, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-2 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 33, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Steven A. Varga, Chief  
Operating Reactors Branch #1  
Division of Licensing

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: October 14, 1983

UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555



ALABAMA POWER COMPANY

DOCKET NO. 50-364

JOSEPH M. FARLEY NUCLEAR PLANT, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No.26  
License No. NPF-8

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Alabama Power Company (the licensee) dated November 16, 1982 supplemented by letters dated February 1, 1983 and June 29, 1983, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

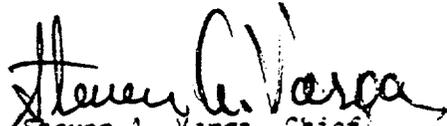
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-8 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 26, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

  
Steven A. Varga, Chief  
Operating Reactors Branch #1  
Division of Licensing

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: October 14, 1983

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 33 TO FACILITY OPERATING LICENSE NO. NPF-2

DOCKET NO. 50-348

Revise Appendix A as follows:

Remove Pages

3/4 3-24  
3/4 3-54  
3/4 3-55  
3/4 3-60  
3/4 3-60a  
3/4 5-2  
3/4 5-4  
3/4 6-10  
3/4 7-87  
3/4 7-93  
3/4 8-12  
3/4 8-13  
3/4 8-14  
6-2  
6-3  
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6-9  
6-10  
6-11  
6-13  
6-14  
6-27

Insert Pages

3/4 3-24  
3/4 3-54  
3/4 3-55  
3/4 3-60  
3/4 3-60a  
3/4 5-2  
3/4 5-4  
3/4 6-10  
3/4 7-87  
3/4 7-93  
3/4 8-12  
3/4 8-13  
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6-2  
6-3  
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TABLE 3.3-3 (Continued)

**ACTION 18** - With the number of OPERABLE Channels one less than the Minimum Channels OPERABLE requirement, restore the inoperable channel to OPERABLE status within 48 hours or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

**ACTION 19** - With the number of OPERABLE Channels one less than the Total Number of Channels, STARTUP and/or POWER OPERATION may proceed provided the following conditions are satisfied:

- a. The inoperable channel is placed in the tripped condition within 1 hour.
- b. The Minimum Channels OPERABLE requirement is met; however, the inoperable channel may be bypassed for up to 2 hours for surveillance testing of other channels per Specification 4.3.2.1.

**ACTION 20** - With the interlock inoperable to the extent that a safeguards function which should not be blocked in the current MODE is blocked, declare the safeguard function(s) inoperable and follow the appropriate ACTION statement(s) of Table 3.3-3 for the affected function(s).

Interlock	Affected Channels on Table 3.3-3
1. P-11	a. Pressurizer Pressure - Low
2. P-12	a. Steam Line Pressure - Low b. Steam Flow in Two Steam Lines High Coincident With T <sub>avg</sub> -Low-Low

**ACTION 21** - With the number of OPERABLE Channels one less than the Minimum Channels OPERABLE requirement, be in at least HOT STANDBY within 6 hours and in at least HOT SHUTDOWN within the following 6 hours; however; one channel may be bypassed for up to 2 hours for surveillance testing provided the other channel is OPERABLE.

**ACTION 22** - With the number of OPERABLE Channels one less than the Total Number of Channels restore the inoperable channel to OPERABLE status within 48 hours or be in at least HOT STANDBY within 6 hours and in HOT SHUTDOWN within the following 6 hours.

**ACTION 23** - With the number of OPERABLE channels one less than the Minimum Number of Channels, operation may proceed until performance of the next required CHANNEL FUNCTIONAL TEST.

TABLE 3.3-0

HIGH ENERGY LINE BREAK ISOLATION INSTRUMENTATION

INSTRUMENT CHANNEL

SENSOR LOCATION

MINIMUM  
OPERABLE  
CHANNELS

1. Pressure Switches

a. PDSH 3367 A,B

RHR Heat Exchanger Room El. 83 ft.

1

b. PSH 2850 A,D  
B,F  
C,E

Auxiliary Feedwater Pump Room El. 100 ft.  
Equipment Room El. 100 ft.  
Auxiliary Feedwater Pump Room Corridor El. 100 ft.

1

1

1

c. PSH 2851 A,F  
B,C  
D,E

Recycle Holdup Tank Room El. 121 ft.  
Moderating Heat Exchanger Room El. 121 ft.  
Moderating Heat Exchanger Valve Room El. 121 ft.

1

1

1

d. PSH 2852 A,H  
B,C,D,E  
F,G

Piping Penetration Room El. 100 ft.  
Letdown Heat Exchanger Room El. 100 ft.  
Recycle Holdup Tank Room El. 121 ft.

1

1

1

e. PSH 2853 A,B  
C,D  
E,F

Piping Penetration Room El. 121 ft.  
Blowdown Heat Exchanger Room El. 130 ft.  
Corridor El. 121 ft.

1

1

1

PART EVALUATION 1

3/4 3-54

AMENDMENT NO. 33

**TABLE 3.3-10 (Continued)**

<b><u>INSTRUMENT CHANNEL</u></b>	<b><u>SENSOR LOCATION</u></b>	<b><u>MINIMUM OPERABLE CHANNELS</u></b>
PSH-2853 G,H I,J	Recycle Evaporator Room El. 121 ft. Corridor El. 121 ft. (I) and Sluice Filter Room El.121 Ft. (J)	1 1
2. . Flooding Detectors		
a. LSH 2828 A,B,C	Main Steam Room El. 127 ft.	2
b. LSH 2829 A,B,C	Main Steam Room El. 127 ft.	2

**TABLE 3.3-12  
FIRE DETECTION INSTRUMENTATION**

**Auxiliary Building**

<u>Room/ Fire Zone</u>	<u>Description</u>	<u>Elevation</u>	<u>Total Smoke Detectors</u>	<u>Minimum Of Operable Smoke Detectors</u>
128	RHR Heat Exchanger Room	83'-0"	10	5
129	RHR Low Head Pump Room	77'-0"	1	1
131	RHR Low Head Pump Room	77'-0"	1	1
160	Hatch Area	100'-0"	3	2
161, 162, 163	South Corridor	100'-0"	7	4
172	Hallway	100'-0"	3	2
173	Charging/SI Pump Room	100'-0"	2	1
174	Charging/SI Pump Room	100'-0"	2	1
175	Hallway	100'-0"	2	1
181	Charging/SI Pump Room	100'-0"	2	1
184	Mechanical Penetration Room	100'-0"	8	4
185	CCW Heat Exchanger and Pump	100'-0"	13	7
186	Boric Acid Trans. PPS	100'-0"	3	2
188	Boric Acid Tanks	121'-0"	2	1
190	MCC Panel Room	100'-0"	4	2
191	Auxiliary Feedwater Pump Room	100'-0"	1	1
192	Auxiliary Feedwater Pump Room	100'-0"	1	1
193	Auxiliary Feedwater Pump Room	100'-0"	2	1
202	Communications Rooms	121'-0"	4	2
208, 207	Corridor	121'-0"	4	2
209	Hallway	121'-0"	4	2
210, 211, 228				
234	South Corridor	121'-0"	7	4
212	Battery Room 1A	121'-0"	1	1
213	Battery Service Room	121'-0"	1	1
214	Battery Room 1B	121'-0"	1	1
223	Mechanical Penetration Room	121'-0"	9	5
224	DC Switchgear Room	121'-0"	1	1
225	Battery Charger Room	121'-0"	1	1
226	DC Switchgear Room	121'-0"	1	1
227	West Cable Chase	128'-0"	7	4
229	AC Switchgear Room Train B	121'-0"	4	2
233	AC Switchgear Room Train B	121'-0"	3	2
241	Main Steam and FDW Vlv. Rm.	127'-0"	8	4
244	Mezzanine - Battery Room	131'-0"	3	2
245	Mezzanine - Battery Room	131'-0"	3	2
254	Hot Shutdown Panel Room	121'-0"	1	1
300	West Cable Chase	139'-0"	7	4
312	Corridor	139'-0"	3	2
316, 322	Corridor	139'-0"	3	2
318	Cable Spreading Room	139'-0"	8	4
319, 339, 345	West Corridor	139'-0"	6	3
333	Electrical Penetration Room	139'-0"	1	1
334	Electrical Penetration Room	139'-0"	6	3
335	AC Switchgear Room Train A	139'-0"	3	2
343	AC Switchgear Room Train B	139'-0"	3	2
347	Electrical Penetration Room	139'-0"	2	1

TABLE 3.3-12  
FIRE DETECTION INSTRUMENTATION  
 (Continued)

Auxiliary Building

<u>Room/ Fire Zone</u>	<u>Description</u>	<u>Elevation</u>	<u>Total Smoke Detectors</u>	<u>Minimum of Operable Smoke Detectors</u>
401	Control Room (above ceiling)	155'-0"	13	7
401	Control Room (below ceiling)	155'-0"	3	2
440,455,456	Clean Toilet, Laundry and Drying Areas	155'-0"	3	2
462	Non-Radioactive Vent Equip. Rm.	155'-0"	5	3
465	Vertical Cable Chase	155'-0"	3	2
466	Vertical Cable Chase	155'-0"	4	2
500	Vertical Cable Chase	168'-2"	7	4
416	Control Room Instrument Racks	155'-0"	8	4

Containment\*

55	Containment Coolers	155'-0"	12/Fan	6/Fan
55	Containment	155'-0"	11	6

Service Water Intake Structure

72 A**	Pump Room Area	188'-9"	12	6
72 A**	Strainer Bay	167'-0"	12	6
72 B**	Switchgear Room - Train B	188'-9"	3	2
72 C**	Foyer - Train B	188'-9"	1	1
72 D**	Foyer - Train A	188'-9"	1	1
72 E**	Switchgear Room Train A	188'-9"	3	2
73 **	Battery Room - Train B	188'-9"	1	1
74 **	Battery Room - Train A	188'-9"	1	1

Diesel Generator Building

56 A	Switchgear Room - Train A	155'-0"	12	6
56 B	Foyer	155'-0"	4	2
56 C	Switchgear Room - Train B	155'-0"	12	6
71	Hallway	155'-0"	9	5

Diesel Generator Building (Heat Detectors)

57	Diesel Driven Generator 2C	155'-0"	5	3
58	Diesel Driven Generator 1B	155'-0"	5	3
60	Diesel Driven Generator 1C	155'-0"	5	3
61	Diesel Driven Generator 1-2A	155'-0"	5	3
62	Day Tank Room 2C	155'-0"	1	1
63	Day Tank Room 1B	155'-0"	1	1
65	Day Tank Room 1C	155'-0"	1	1
66	Day Tank Room 1-2A	155'-0"	1	1

\*  
 The Fire Detection instruments located within the Containment are not required to be OPERABLE during the performance of Type A Containment Leakage Rate Tests.  
 \*\* These circuits alarm in the Unit No. 1 control room area but service both Units, and appear in the Technical Specifications for both units.

## EMERGENCY CORE COOLING SYSTEMS

### SURVEILLANCE REQUIREMENTS (Continued)

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- b. At least once per 31 days and within 6 hours after each solution volume increase of greater than or equal to 1% of tank volume by verifying the boron concentration of the accumulator solution.
- c. At least once per 31 days when the RCS pressure is above the P-11 setpoint by verifying that power to each isolation valve operator is disconnected by a locked open disconnect device.
- d. At least once per 18 months by verifying that each accumulator isolation valve opens automatically under each of the following conditions:
  - 1. When the RCS pressure (actual or simulated) exceeds the P-11 (Pressurizer Pressure Block of Safety Injection) setpoint,
  - 2. Upon receipt of a safety injection test signal.

# EMERGENCY CORE COOLING SYSTEMS

## SURVEILLANCE REQUIREMENTS

### 4.5.2 Each ECCS subsystem shall be demonstrated OPERABLE:

- a. At least once per 12 hours by verifying that the following valves are in the indicated positions with the disconnect device to the valve operators locked open:

<u>Valve Number</u>	<u>Valve Function</u>	<u>Valve Position</u>
a. 8884, 8886	Charging Pump to RCS Hot Leg	Closed
b. 8132A, 8132B	Charging Pump discharge isolation	Open*
c. 8889	RHR to RCS Hot Leg Injection	Closed

- b. At least once per 31 days by verifying that each valve (manual, power operated or automatic) in the flow path that is not locked, sealed, or otherwise secured in position, is in its correct position.
- c. By a visual inspection which verifies that no loose debris (rags, trash, clothing, etc.) is present in the containment which could be transported to the containment sump and cause restriction of the pump suction during LOCA conditions. This visual inspection shall be performed:
1. For all accessible areas of the containment prior to establishing CONTAINMENT INTEGRITY, and
  2. Of the areas affected within containment at the completion of each containment entry when CONTAINMENT INTEGRITY is established.
- d. At least once per 18 months by:
1. Verifying automatic isolation and interlock action of the RHR system from the Reactor Coolant System when the Reactor Coolant System pressure is between 700 psig and 750 psig.
  2. A visual inspection of the containment sump and verifying that the subsystem suction inlets are not restricted by debris and that the sump components (trash racks, screens, inner cages) are properly installed and show no evidence of structural distress or corrosion.

\*Will be verified if charging pump 1A is declared inoperable.

## CONTAINMENT SYSTEMS

### CONTAINMENT VENTILATION SYSTEM

#### LIMITING CONDITION FOR OPERATION

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3.6.1.7 The 48-inch containment purge supply and exhaust isolation valves (CBV-HV-3198A, 3198D, 3196, 3197) shall be closed. The 8-inch containment vent supply and exhaust isolation valves may be open.

APPLICABILITY: MODES 1, 2, 3, and 4.

#### ACTION:

With one 48-inch containment purge supply and/or one exhaust isolation valve open, close the open valve(s) within one hour or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

#### SURVEILLANCE REQUIREMENTS

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4.6.1.7.1 The 48-inch containment purge supply and exhaust isolation valves shall be determined closed at least once per 31 days.

4.6.1.7.2 The valve seals of the 48-inch and the 8-inch vent supply and exhaust isolation valves shall be replaced at least once per 5 years.

TABLE 3.7-5

LIST OF SPRINKLER SYSTEMS

<u>Location Room/Fire Zone</u>	<u>Description</u>	<u>Elevation</u>	<u>Sprinkler System</u>
<u>Auxiliary Building</u>			
161	North West Corridor	100'-0"	1A-25
161	North East Corridor	100'-0"	1A-118
185	North West Corner of CCW Pump Rm	100'-0"	1A-119
185	CCW Heat Exchanger and Pump Room	100'-0"	1A-27
190	Aux. Feedwater Pumps Cable Area	100'-0"	1A-62
209	North Corridor	121'-0"	1A-35
210	Auxiliary Bldg. West Corridor	121'-0"	1A-36
223	Pipe Penetration Rm & Tendon Access	121'-0"	1A-45
227	Vertical Cable Chase	128'-0"	1A-23
244	Battery Room Mezzanine	131'-0"	1A-36
245	Battery Room Mezzanine	131'-0"	1A-36
300	Vertical Cable Chase	139'-0"	1A-23
312	North Corridor	139'-0"	1A-48
316	North Corridor	139'-0"	1A-48
318	Cable Spreading Room	139'-0"	1A-43
319	West Corridor	139'-0"	1A-59
404	West Corridor at EL 155'-0"	155'-0"	1A-114
462	Environmental Low Activity Lab and Non-Radioactive Vent Rooms	155'-0"	1A-51
466	Vertical Cable Chase	155'-0"	1A-23
500	Vertical Cable Chase	168'-2"	1A-23
<u>Service Water Intake Structure</u>			
72A	Service Water Pump Protection	102'-0"	1SW-111

TABLE 3.7-7

YARD FIRE HYDRANTS AND ASSOCIATED HYDRANT HOSE HOUSES

<u>LOCATION</u>	<u>HYDRANT NUMBER</u>
1. West of Auxiliary Building	N1Y43V103*
2. West of Service Water Intake	N1Y43V121*
3. East of Service Water Intake	N1Y43V122*
4. North of Diesel Generator Building	N1Y43V120*
5. East of Auxiliary Building	N1Y43V089
6. Northeast of Diesel Generator Building	N1Y43V090

\*Shared between Unit 1 and Unit 2

LIMITING CONDITION FOR OPERATION

3.8.2.5 The following D.C. distribution systems shall be energized and OPERABLE:

TRAIN "A" consisting of 125-volt D.C. Distribution Cabinet 1M, 125-volt battery bank No. 1 and a full capacity charger.

TRAIN "B" consisting of 125-volt D.C. Distribution Cabinet 1N, 125-volt battery bank No. 2, and a full capacity charger.

APPLICABILITY: MODES 1, 2, 3 and 4.

ACTION:

With one 125-volt D.C. distribution train inoperable\*, restore the inoperable distribution system to OPERABLE and energized status within 2 hours or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

SURVEILLANCE REQUIREMENTS

4.8.2.5.1 Each D.C. train shall be determined OPERABLE and energized at least once per 7 days by verifying correct breaker alignment and indicated power availability.

4.8.2.5.2 Each 125-volt battery bank and charger shall be demonstrated OPERABLE:

- a. At least once per 7 days by verifying that:
  1. The electrolyte level of the pilot cell is between the minimum and maximum level indication marks.
  2. The pilot cell specific gravity, corrected to 77°F and full electrolyte level, is greater than or equal to 1.190, and
  3. The pilot cell voltage is greater than or equal to 2.02 volts.
  4. The total battery terminal voltage is greater than or equal to 121.2 volts.
- b. At least once per 92 days by verifying that:
  1. The electrolyte level of each cell is between the minimum and maximum level indication marks.

\* Except during performance of surveillance Requirement 4.8.2.5.2.d and 4.8.2.5.2.e. During this test, one train may be inoperable until the battery is recharged following completion of the battery discharge test.

SURVEILLANCE REQUIREMENTS (Continued)

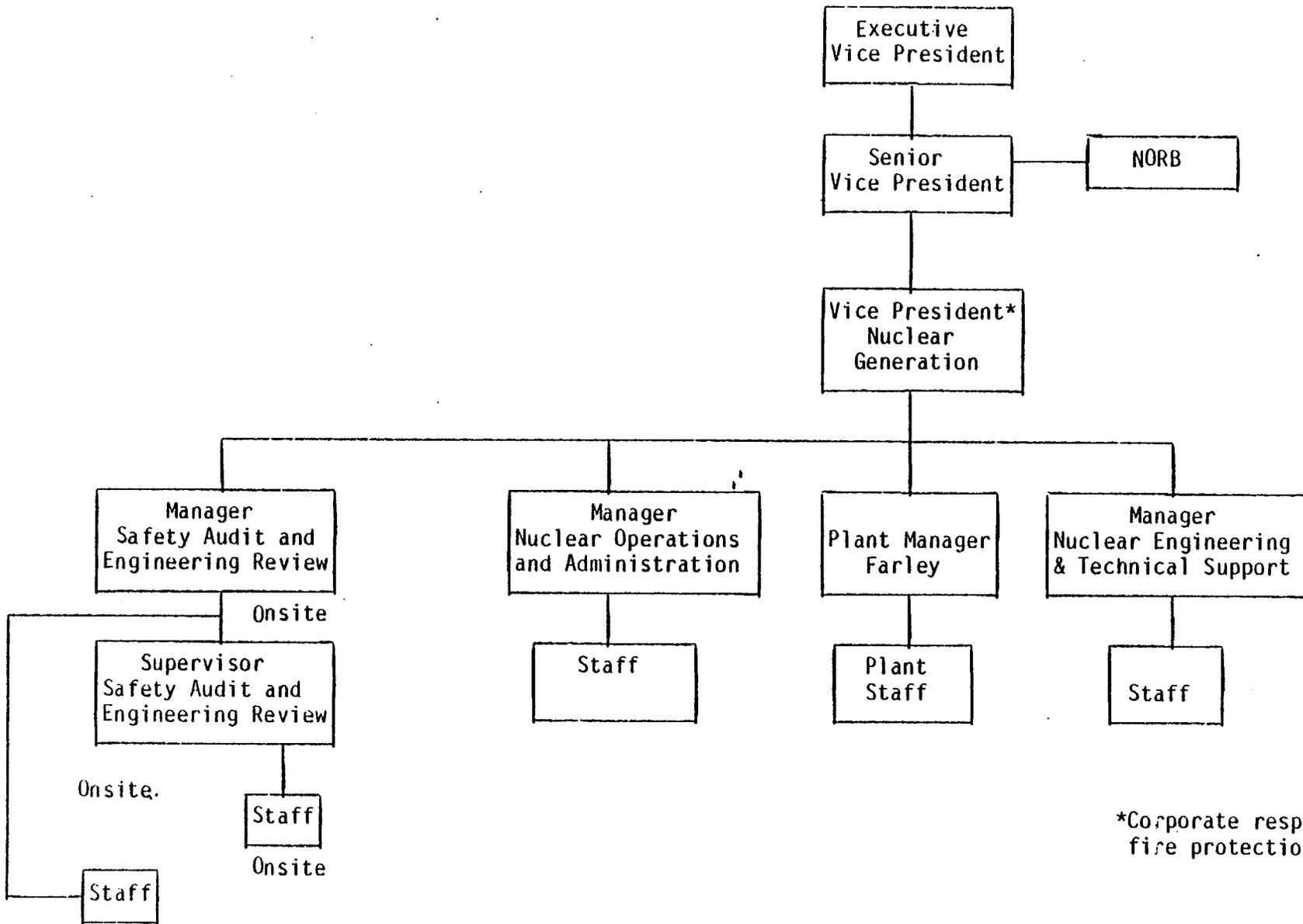
2. The voltage of each connected cell is greater than or equal to 2.02 volts under float charge and has not decreased more than 0.1 volts from the value observed during the original acceptance test; and
  3. The specific gravity, corrected to 77°F and full electrolyte level, of each connected cell is greater than or equal to 1.190 and has not decreased more than 0.08 from the value observed during the previous test.
- c. At least once per 18 months by verifying that:
1. The cells, cell plates and battery racks show no visual indication of physical damage or abnormal deterioration,
  2. The cell-to-cell and terminal connections are clean, tight, and coated with anti-corrosion material, and
  3. The battery charger will supply at least 3 amperes at  $\geq$  125 volts for at least 4 hours.
- d. At least once per 18 month, during a shutdown of either unit, by verifying that the battery capacity is adequate to supply and maintain in OPERABLE status all of the actual emergency loads for 2 hours when the battery is subjected to a battery service test or the individual cell voltage does not decrease below 1.75 volts when the battery is subjected to the following equivalent load profile:

<u>Order in Which Loads are Applied</u>	<u>Current (amps)</u>	<u>Duration</u>
1	25	0 - 0.1 sec
2	1	0.1 sec - 2 hours

ELECTRICAL POWER SYSTEMS

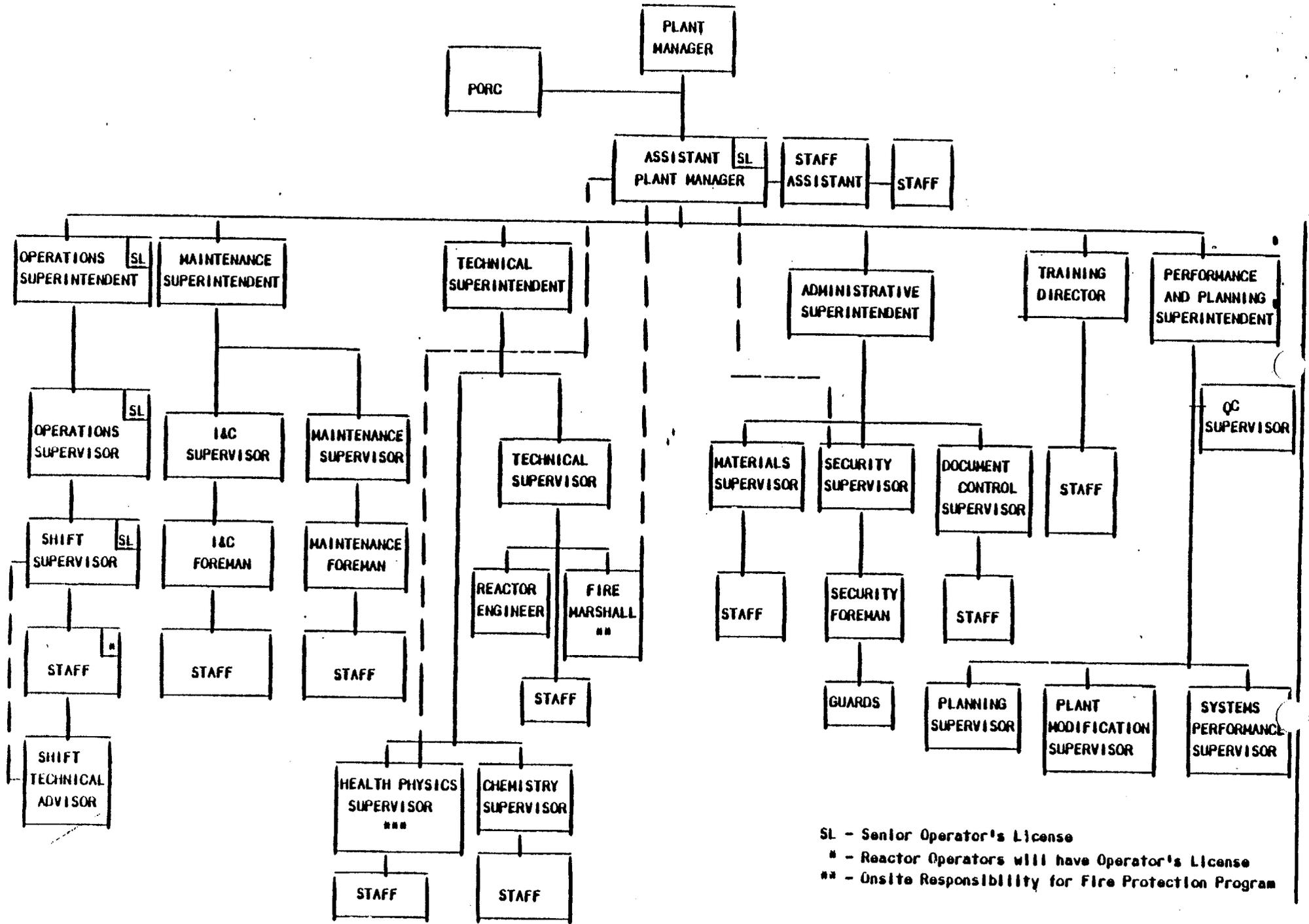
SURVEILLANCE REQUIREMENT (Continued)

- e. At least once per 60 month, during a shutdown of either unit, by verifying that the battery capacity is at least 80% of the manufacturer's rating when subjected to a performance discharge test. This performance discharge test shall be performed subsequent to the satisfactory completion of the required battery service test.



\*Corporate responsibility for fire protection program

Figure 6.2-1 Offsite Organization for Facility Management and Technical Support



SL - Senior Operator's License  
 \* - Reactor Operators will have Operator's License  
 \*\* - Onsite Responsibility for Fire Protection Program

FIGURE 6.2-2 FACILITY ORGANIZATION

\*\*\* In routine matters, the Health Physics Supervisor reports directly to the Technical Superintendent, in matters of radiation policy determination, interpretation or implementation (based upon the Health Physics Supervisor's judgement) the Health Physics Supervisor may report directly to the Assistant Plant Manager.

## ADMINISTRATIVE CONTROLS

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### 6.2.3 SAFETY AUDIT AND ENGINEERING REVIEW GROUP (SAERG)

#### FUNCTION

6.2.3.1 The SAERG shall function to conduct operational evaluations, engineering reviews, and audits for the purpose of improving safety.

#### COMPOSITION

6.2.3.2 The SAERG shall be composed of a multi-disciplined dedicated onsite group with a minimum assigned complement of five engineers or appropriate specialists.

#### RESPONSIBILITIES

6.2.3.3 The SAERG shall be responsible for the following:

- a. Participating in operational evaluations for improvement of safety wherein such evaluations and recommendations therefrom are not limited to the fulfillment of existing programs, policies, procedures, or capabilities of existing equipment and installations.
- b. Systematic engineering reviews of plant performance and activities with results reported independently of onsite operational management to offsite upper management.
- c. Comprehensive plant audits in accordance with audit requirements set forth in quality assurance programs, licensing documents, and other policies and procedures.

#### AUTHORITY

6.2.3.4 The onsite SAERG shall carry out its function reporting offsite directly to the Manager-Safety Audit and Engineering Review who in turn reports directly to the Vice President-Nuclear Generation.

### 6.2.4 SHIFT TECHNICAL ADVISOR

6.2.4.1 The Shift Technical Advisor shall serve in an advisory capacity to the Shift Supervisor primarily in the assessment of accident and transient occurrences.

### 6.3 FACILITY STAFF QUALIFICATIONS

6.3.1 Each member of the facility staff shall meet or exceed the minimum qualifications of ANSI N18.1-1971 for comparable positions and the supplemental requirements specified in Sections A and C of Enclosure 1 of the March 28, 1980 NRC letter to all licensees, except for (1) the Health Physics Supervisor who shall meet or exceed the qualifications of Regulatory Guide 1.8, September 1975.

## ADMINISTRATIVE CONTROLS

### AUTHORITY

6.5.1.7 The PORC shall:

- a. Recommend to the Plant Manager in writing, approval or disapproval of items considered under 6.5.1.6(a) through (e) and (j) and (k) above.
- b. Render determinations in writing with regard to whether or not each item considered under 6.5.1.6(a), (c) and (d) above constitutes an unreviewed safety question.
- c. Make recommendations to the Plant Manager in writing that actions reviewed under 6.5.1.6(b) above did not constitute an unreviewed safety question.

### RECORDS

6.5.1.8 The PORC shall maintain written minutes of each meeting and copies shall be provided to the Vice President-Nuclear Generation and Chairman of the Nuclear Operations Review Board.

## 6.5.2 NUCLEAR OPERATIONS REVIEW BOARD (NORB)

### FUNCTION

6.5.2.1 The NORB shall function to provide independent review and audit of designated activities in the areas of:

- a. Nuclear power plant operations
- b. Nuclear engineering
- c. Chemistry and radiochemistry
- d. Metallurgy
- e. Instrumentation and control
- f. Radiological safety
- g. Mechanical and electrical engineering
- h. Quality assurance practices

### COMPOSITION

6.5.2.2 The NORB shall be composed of at least five persons including:

Chairman:	Senior Vice President
Vice Chairman:	Vice President-Nuclear Generation
Secretary:	Manager-Safety Audit & Engineering Review
Member:	Manager-Nuclear Operations and Administration
Member:	Manager-Nuclear Engineering and Technical Support

and other appointed personnel having an academic degree in an engineering or physical science field and a minimum of five years technical experience, of which a minimum of three years shall be in one or more of the areas given in 6.5.2.1.

## ADMINISTRATIVE CONTROLS

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### ALTERNATES

6.5.2.3 All alternate members shall be appointed in writing by the NORB Chairman to serve on a temporary basis; however, no more than two alternates shall participate as voting members in NORB activities at any one time.

### CONSULTANTS

6.5.2.4 Consultants shall be utilized as determined by the NORB Chairman to provide expert advice to the NORB.

### MEETING FREQUENCY

6.5.2.5 The NORB shall meet at least once per calendar quarter during the initial year of unit operation following fuel loading and at least once per six months thereafter.

### QUORUM

6.5.2.6 A quorum shall consist of the Chairman or Vice Chairman plus enough voting members to constitute a majority of the NORB. No more than a minority of the quorum shall have line responsibility for operation of the facility. For the purpose of a quorum those considered to have line responsibility will include the Plant Manager and personnel reporting to the Plant Manager.

### REVIEW

6.5.2.7 The NORB shall review:

- a. Proposed changes to procedures, equipment or systems which involve an unreviewed safety question as defined in Section 50.59, 10 CFR.
- b. Proposed tests or experiments which involve an unreviewed safety question as defined in Section 50.59, 10 CFR.
- c. Violations of codes, regulations, orders, Technical Specifications, license requirements, or of internal procedures or instructions having nuclear safety significance or abnormal degradation of systems designed to contain radioactive material.
- d. Significant operating abnormalities or deviations from normal and expected performance of plant equipment that affect nuclear safety.
- e. Written reports concerning events requiring 24 hour notification to the Commission.
- f. Recognized indications of an unanticipated deficiency in some aspect of design or operation of safety related structures, systems, or components.

## ADMINISTRATIVE CONTROLS

- g. Reports and meetings minutes of the PORC.
- h. Proposed changes to Technical Specifications or this Operating License.
- i. The safety evaluations for proposed 1) procedures 2) changes to procedures, equipment or systems and 3) test or experiments completed under the provision of Section 50.59 10 CFR, to verify that such actions did not constitute an unreviewed safety question.

## AUDITS

6.5.2.8 The following audits shall be conducted under the direction of APCO's Manager - Safety Audit and Engineering Review:

- a. The conformance of facility operation to provisions contained within the Technical Specifications and applicable license conditions at least once per 12 months.
- b. The performance, training and qualifications of the entire facility staff at least once per 12 months.
- c. The results of actions taken to correct deficiencies occurring in facility equipment, structures, systems or method of operation that affect nuclear safety at least once per 6 months.
- d. The performance of activities required by the Operational Quality Assurance Program to meet the criteria of Appendix "B", 10 CFR 50, at least once per 24 months.
- e. The Facility Emergency Plan at least once per 24 months.
- f. The Facility Security Plan at least once per 12 months.
- g. Any other area of facility operation considered appropriate by the NORB or the Senior Vice President.
- h. The Facility Fire Protection Program and implementing procedures at least once per 24 months.
- i. An independent fire protection and loss prevention program inspection and audit of the unit at least once per 12 months utilizing either qualified offsite licensee personnel or an outside fire protection firm.
- j. An inspection and audit of the unit fire protection and loss prevention program by a qualified outside fire consultant at least once per 36 months.
- k. The radiological effluent and environmental monitoring programs and the results thereof at least once per 12 months.

## ADMINISTRATIVE CONTROLS

Reactor Operator's License. For changes to procedures which may involve a change in intent of the approved procedures, the person authorized above to approve the procedure shall approve the change.

- b. Proposed changes or modifications to plant nuclear safety-related structures, systems and components shall be reviewed as designated by the Plant Manager. Each such modification shall be reviewed by an individual/group other than the individual/group which designed the modification, but who may be from the same organization as the individual/group which designed the modifications. Proposed modifications to plant nuclear safety-related structures, systems and components shall be approved prior to implementation by the Plant Manager.
- c. Proposed tests and experiments which affect plant nuclear safety and are not addressed in the Final Safety Analysis Report shall be prepared, reviewed, and approved. Each such test or experiment shall be reviewed by an individual/group other than the individual/group which prepared the proposed test or experiment. Proposed test and experiments shall be approved before implementation by the Plant Manager.
- d. Occurrences reportable pursuant to the Technical Specification 6.0 and violations of Technical Specifications shall be investigated and a report prepared which evaluates the occurrence and which provides recommendations to prevent recurrence. Such reports shall be approved by the Plant Manager and forwarded to the Vice President, Nuclear Generation with a copy to the Manager of Nuclear Operations and Administration; and to the Chairman of the Nuclear Operations Review Board.
- e. Individuals responsible for reviews performed in accordance with 6.5.3.1.a, 6.5.3.1.b, 6.5.3.1.c, and 6.5.3.1.d shall be members of the plant supervisory staff previously designated by the Plant Manager. Each such review shall include a determination of whether or not additional, cross-disciplinary, review is necessary. If deemed necessary, such review shall be performed by the review personnel of the appropriate discipline.
- f. Each review will include a determination of whether or not an unreviewed safety question is involved. Pursuant to 10 CFR 50.59 NRC approval of items involving unreviewed safety question will be obtained prior to Plant Manager approval for implementation.

## RECORDS

6.5.3.2 Records of the above activities shall be provided to the Plant Manager, PORC and/or NORB as necessary for required reviews.

## ADMINISTRATIVE CONTROLS

### 6.6 REPORTABLE OCCURRENCE ACTION

6.6.1 The following actions shall be taken for REPORTABLE OCCURRENCES:

- a. The Commission shall be notified and/or a report submitted pursuant to the requirements of Specification 6.9.
- b. Each REPORTABLE OCCURRENCE requiring 24 hour notification to the Commission shall be reviewed by the PORC and submitted to the NORB and the Vice President - Nuclear Generation with a copy to the Manager of Nuclear Operations and Administration.

### 6.7 SAFETY LIMIT VIOLATION

6.7.1 The following actions shall be taken in the event a Safety Limit is violated:

- a. The facility shall be placed in at least HOT STANDBY within one hour.
- b. The NRC Operations Center shall be notified by telephone as soon as possible and all cases within one hour. The Vice President, Nuclear Generation shall be notified within 24 hours.
- c. A Safety Limit Violation Report shall be prepared. The report shall be reviewed by the PORC. This report shall describe (1) applicable circumstances preceding the violation, (2) effects of the violation upon facility components, systems or structures, and (3) corrective action taken to prevent recurrence.
- d. The Safety Limit Violation Report shall be submitted to the Commission, and the Vice President, Nuclear Generation, with a copy to the Manager of Nuclear Operations and Administration, for NORB review within 14 days of the violation.

### 6.8 PROCEDURES AND PROGRAMS

6.8.1 Written procedures shall be established, implemented and maintained covering the activities reference below:

- a. The applicable procedures recommended in Appendix "A" of Regulatory Guide 1.33, Revision 2, 1978.
- b. Refueling operations.
- c. Surveillance and test activities of safety related equipment.
- d. Security Plan implementation.
- e. Emergency Plan implementation.

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 26 TO FACILITY OPERATING LICENSE NO. NPF-8

DOCKET NO. 50-364

Revise Appendix A as follows:

Remove Pages

3/4 3-24  
3/4 3-54  
3/4 3-55  
3/4 3-60  
3/4 3-60a  
3/4 5-2  
3/4 5-4  
3/4 7-63  
3/4 8-15  
3/4 8-16  
3/4 8-17  
6-2  
6-3  
6-6  
6-9  
6-10  
6-11  
6-13  
6-14  
7-1

Insert Pages

3/4 3-24  
3/4 3-54  
3/4 3-55  
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3/4 3-60a  
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3/4 5-4  
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TABLE 3.3-10

HIGH ENERGY LINE BREAK ISOLATION INSTRUMENTATION

INSTRUMENT CHANNEL

SENSOR LOCATION

MINIMUM  
OPERABLE  
CHANNELS

1. Pressure Switches

- |    |              |  |   |  |
|----|--------------|--|---|--|
| a. | PSH 3367 A,B | RHR Heat Exchanger Room El. 83 ft.                 | 1 |  |
| b. | PSH 2850 A,D | Auxiliary Feedwater Pump Room El. 100 ft.          | 1 |  |
|    | B,F          | Equipment Room El. 100 ft.                         | 1 |  |
|    | C,E          | Auxiliary Feedwater Pump Room Corridor El. 100 ft. | 1 |  |
| c. | PSH 2851 A,F | Recycle Holdup Tank Room El. 121 ft.               | 1 |  |
|    | B,C          | Moderating Heat Exchanger Room El. 121 ft.         | 1 |  |
|    | D,E          | Moderating Heat Exchanger Valve Room El. 121 ft.   | 1 |  |
| d. | PSH 2852 A,H | Piping Penetration Room El. 100 ft.                | 1 |  |
|    | B,C,D,E      | Letdown Heat Exchanger Room El. 100 ft.            | 1 |  |
|    | F,G          | Recycle Holdup Tank Room El. 121 ft.               | 1 |  |
| e. | PSH 2853 A,B | Piping Penetration Room El. 121 ft.                | 1 |  |
|    | C,D          | Blowdown Heat Exchanger Room El. 130 ft.           | 1 |  |
|    | E,F          | Recycle Evaporator Room El. 121 ft.                | 1 |  |

3/4 3-54

AMENDMENT NO. 26

TABLE 3.3-10 (Continued)

<u>INSTRUMENT CHANNEL</u>	<u>SENSOR LOCATION</u>	<u>MINIMUM OPERABLE CHANNELS</u>
PSH-2853 G,H I,J	Recycle Evaporator Room El. 121 ft. Corridor El. 121 ft. (I) and Blowdown Surge Tank Room El. 130 ft. (J)	1 1
2. . Flooding Detectors		
a. LSH 2828 A,B,C	Main Steam Room El. 127 ft.	2
b. LSH 2829 A,B,C	Main Steam Room El. 127 ft.	2

TABLE 3.3-12  
FIRE DETECTION INSTRUMENTATION

Auxiliary Building

<u>Room/ Fire Zone</u>	<u>Description</u>	<u>Elevation</u>	<u>Total Smoke Detectors</u>	<u>Minimum Of Operable Smoke Detectors</u>
2128	RHR Heat Exchanger Room	83'-0"	10	5
2129	RHR Low Head Pump Room	77'-0"	1	1
2131	RHR Low Head Pump Room	77'-0"	1	1
2160	Hatch Area	100'-0"	2	1
2161	South Corridor	100'-0"	2	1
2172	Hallway	100'-0"	3	2
2173	Charging/SI Pump Room	100'-0"	2	1
2174	Charging/SI Pump Room	100'-0"	2	1
2175	Hallway	100'-0"	2	1
2181	Charging/SI Pump Room	100'-0"	2	1
2184	Mechanical Penetration Room	100'-0"	8	4
2185	CCW Heat Exchanger and Pump	100'-0"	13	7
2186	Boric Acid Trans. PPS	100'-0"	3	2
2188	Boric Acid Tanks	121'-0"	2	1
2190	MCC Panel Room	100'-0"	4	2
2191	Auxiliary Feedwater Pump Room	100'-0"	1	1
2192	Auxiliary Feedwater Pump Room	100'-0"	1	1
2193	Auxiliary Feedwater Pump Room	100'-0"	2	1
2202	Hot Shutdown Panel Room	121'-0"	4	2
2208	Corridor	121'-0"	2	1
2209	Hallway	121'-0"	3	2
2210	South Corridor	121'-0"	2	1
2212	Battery Room 1A	121'-0"	1	1
2213	Battery Service Room	121'-0"	1	1
2214	Battery Room 1B	121'-0"	1	1
2223	Mechanical Penetration Room	121'-0"	9	5
2224	DC Switchgear Room	121'-0"	1	1
2226	DC Switchgear Room	121'-0"	1	1
2227	West Cable Chase	128'-0"	7	4
2229	AC Switchgear Room Train B	121'-0"	4	2
2233	AC Switchgear Room Train B	121'-0"	3	2
2241	Main Steam and FDW Vlv. Rm.	127'-0"	8	4
2244	Mezzanine	131'-0"	3	2
2245	Mezzanine	131'-0"	2	1
2254	Hot Shutdown Panel Room	121'-0"	1	1
2300	West Cable Chase	139'-0"	7	4
2309	Hatch Area	139'-0"	1	1
2312	Corridor	139'-0"	4	2
2316	Corridor	139'-0"	2	1
2318	Cable Spreading Room	139'-0"	8	4
2319	West Corridor	139'-0"	2	1
2333	Electrical Penetration Room	139'-0"	1	1
2334	Electrical Penetration Room	139'-0"	3	2
2335	AC Switchgear Room Train A	139'-0"	3	2
2343	AC Switchgear Room Train B	139'-0"	3	2
2347	Electrical Penetration Room	139'-0"	2	1

FIRE DETECTION INSTRUMENTATION  
(Continued)

Auxiliary Building

<u>Room/ Fire Zone</u>	<u>Description</u>	<u>Elevation</u>	<u>Total Smoke Detectors</u>	<u>Minimum of Operable Smoke Detectors</u>
2401 #	Control Room (above ceiling)	155'-0"	16	8
2401 #	Control Room (below ceiling)	155'-0"	3	2
2452	Storage Area	155'-0"	7	4
2462	Non-Radioactive Vent Equip. Rm.	155'-0"	5	3
2466	West Cable Chase	155'-0"	7	4
2471 #	Control Room (Instrument Racks)	155'-0"	9	5
2500	West Cable Chase	168'-2"	7	4

Containment\*

55	Containment Coolers	155'-9"	12/Fan	6/Fan
55	Containemnt	155'-0"	14	7

Service Water Intake Structure

72 A**	Pump Room Area	188'-9"	12	6
72 A**	Strainer Bay	167'-0"	12	6
72 B**	Switichgear Room - Train B	188'-9"	3	2
72 C**	Foyer - Train B	188'-9"	1	1
72 D**	Foyer - Train A	188'-9"	1	1
72 E**	Switchgear Room - Train A	188'-9"	3	2
73**	Battery Room - Train B	188'-9"	1	1
74**	Battery Room - Train A	188'-9"	1	1

Diesel Generator Building

56 A	Switchgear Room - Train A	155'-0"	12	6
56 B	Foyer	155'-0"	4	2
56 C	Switchgear Room - Train B	155'-0"	12	6
71	Hallway	155'-0"	9	5

Diesel Generator Building (Heat Detectors)

57	Diesel Driven Generator 2C	155'-0"	5	3
59	Diesel Driven Generator 2B	155'-0"	5	3
60	Diesel Driven Generator 1C	155'-0"	5	3
61	Diesel Driven Generator 1-2A	155'-0"	5	3
62	Day Tank Room 2C	155'-0"	1	1
64	Day Tank Room 2B	155'-0"	1	1
65	Day Tank Room 1C	155'-0"	1	1
66	Day Tank Room 1-2A	155'-0"	1	1

#These circuits alarm in the Unit 1 control room area, but service Unit 2 only and appear only in the Unit 2 Technical Specifications.

\*The Fire Detection instruments located within the Containment are not required to be OPERABLE during the performance of Type A Containment Leakage Rate Tests.

\*\*These circuits alarm in the Unit 1 control room area but service both units, and appear in the Technical Specifications for both units.

EMERGENCY CORE COOLING SYSTEMS

SURVEILLANCE REQUIREMENTS (Continued)

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- b. At least once per 31 days and within 6 hours after each solution volume increase of greater than or equal to 1% of tank volume by verifying the boron concentration of the accumulator solution.
- c. At least once per 31 days when the RCS pressure is above the P-11 setpoint by verifying that power to each isolation valve operator is disconnected by a locked open disconnect device.
- d. At least once per 18 months by verifying that each accumulator isolation valve opens automatically under each of the following conditions:
  - 1. When the RCS pressure (actual or simulated) exceeds the P-11 (Pressurizer Pressure Block of Safety Injection) setpoint,
  - 2. Upon receipt of a safety injection test signal.

## EMERGENCY CORE COOLING SYSTEMS

### SURVEILLANCE REQUIREMENTS

#### 4.5.2 Each ECCS subsystem shall be demonstrated OPERABLE:

- a. At least once per 12 hours by verifying that the following valves are in the indicated positions with the disconnect device to the valve operators locked open:

<u>Valve Number</u>	<u>Valve Function</u>	<u>Valve Position</u>
a. 8884, 8886	Charging Pump to RCS Hot Leg	Closed
b. 8132A, 8132B	Charging Pump discharge isolation	Open*
c. 8889	RHR to RCS Hot Leg Injection	Closed

- b. At least once per 31 days by verifying that each valve (manual, power operated or automatic) in the flow path that is not locked, sealed, or otherwise secured in position, is in its correct position.
- c. By a visual inspection which verifies that no loose debris (rags, trash, clothing, etc.) is present in the containment which could be transported to the containment sump and cause restriction of the pump suction during LOCA conditions. This visual inspection shall be performed:
1. For all accessible areas of the containment prior to establishing CONTAINMENT INTEGRITY, and
  2. Of the areas affected within containment at the completion of each containment entry when CONTAINMENT INTEGRITY is established.
- d. At least once per 18 months by:
1. Verifying automatic isolation and interlock action of the RHR system from the Reactor Coolant System when the Reactor Coolant System pressure is between 700 psig and 750 psig.
  2. A visual inspection of the containment sump and verifying that the subsystem suction inlets are not restricted by debris and that the sump components (trash racks, screens, inner cages) are properly installed and show no evidence of structural distress or corrosion.

\*Will be verified if charging pump 2A is declared inoperable.

TABLE 3.7-7

YARD FIRE HYDRANTS AND ASSOCIATED HYDRANT HOSE HOUSES

<u>LOCATION</u>	<u>HYDRANT NUMBER</u>
1. Northeast of Auxiliary Building	N1Y43V105
2. Northwest of Auxiliary Building	N1Y43V102
3. West of Auxiliary Building	N1Y43V103*
4. North of Diesel Generator Building	N1Y43V120*
5. West of Service Water Intake	N1Y43V121*
6. East of Service Water Intake	N1Y43V122*

\*Shared between Unit 1 and Unit 2

ELECTRICAL POWER SYSTEM

SERVICE WATER BUILDING D.C. DISTRIBUTION - OPERATING

LIMITING CONDITION FOR OPERATION

3.8.2.5 The following D.C. distribution systems shall be energized and OPERABLE:

TRAIN "A" consisting of 125-volt D.C. Distribution Cabinet 1M, 125-volt battery bank No. 1 and a full capacity charger.

TRAIN "B" consisting of 125-volt D.C. Distribution Cabinet 1N, 125-volt battery bank No. 2, and a full capacity charger.

APPLICABILITY: MODES 1, 2, 3 and 4.

ACTION:

With one 125-volt D.C. distribution train inoperable\*, restore the inoperable distribution system to OPERABLE and energized status within 2 hours or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

SURVEILLANCE REQUIREMENTS

4.8.2.5.1 Each D.C. train shall be determined OPERABLE and energized at least once per 7 days by verifying correct breaker alignment and indicated power availability.

4.8.2.5.2 Each 125-volt battery bank and charger shall be demonstrated OPERABLE:

- a. At least once per 7 days by verifying that:
  1. The electrolyte level of the pilot cell is between the minimum and maximum level indication marks.
  2. The pilot cell specific gravity, corrected to 77° and full electrolyte level, is greater than or equal to 1.190, and
  3. The pilot cell voltage is greater than or equal to 2.02 volts.
  4. The total battery terminal voltage is greater than or equal to 121.2 volts.
- b. At least once per 92 days by verifying that:
  1. The electrolyte level of each cell is between the minimum and maximum level indication marks.

\* Except during performance of surveillance Requirement 4.8.2.5.2.d and 4.8.2.5.2.e. During this test, one train may be inoperable until the battery is recharged following completion of the battery discharge test.

ELECTRICAL POWER SYSTEMS

SURVEILLANCE REQUIREMENTS (Continued)

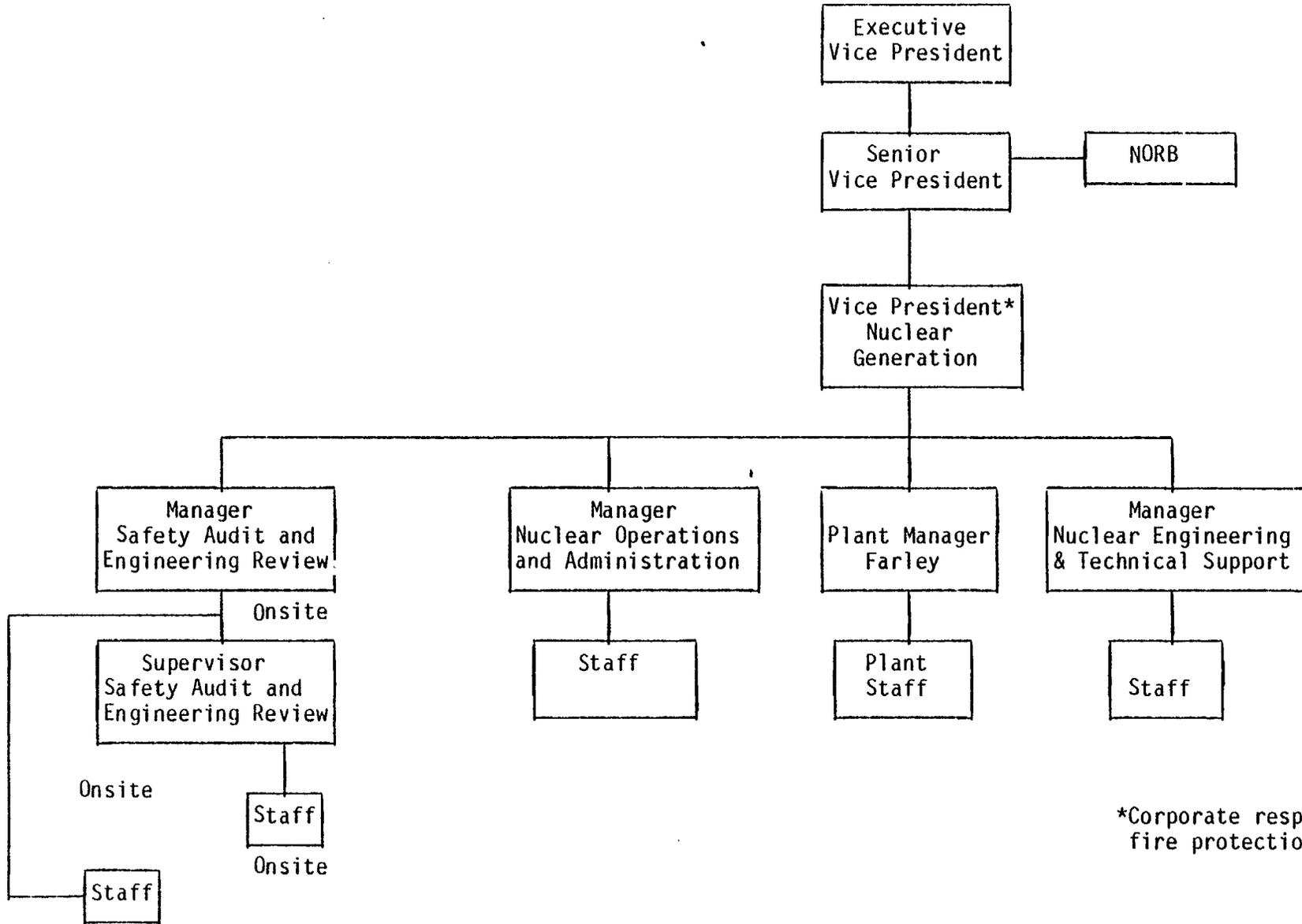
2. The voltage of each connected cell is greater than or equal to 2.02 volts under float charge and has not decreased more than 0.1 volts from the value observed during the original acceptance test, and
  3. The specific gravity, corrected to 77°F and full electrolyte level, of each connected cell is greater than or equal to 1.190 and has not decreased more than 0.08 from the value observed during the previous test.
- c. At least once per 18 months by verifying that:
1. The cells, cell plates and battery racks show no visual indication of physical damage or abnormal deterioration,
  2. The cell-to-cell and terminal connections are clean, tight, and coated with anti-corrosion material, and
  3. The battery charger will supply at least 3 amperes at  $\geq$  125 volts for at least 4 hours.
- d. At least once per 18 month, during a shutdown of either unit, by verifying that the battery capacity is adequate to supply and maintain in OPERABLE status all of the actual emergency loads for 2 hours when the battery is subjected to a battery service test or the individual cell voltage does not decrease below 1.75 volts when the battery is subjected to the following equivalent load profile:

<u>Order in Which Loads are Applied</u>	<u>Current (amps)</u>	<u>Duration</u>
1	25	0 - 0.1 sec
2	1	0.1 sec - 2 hours

ELECTRICAL POWER SYSTEMS

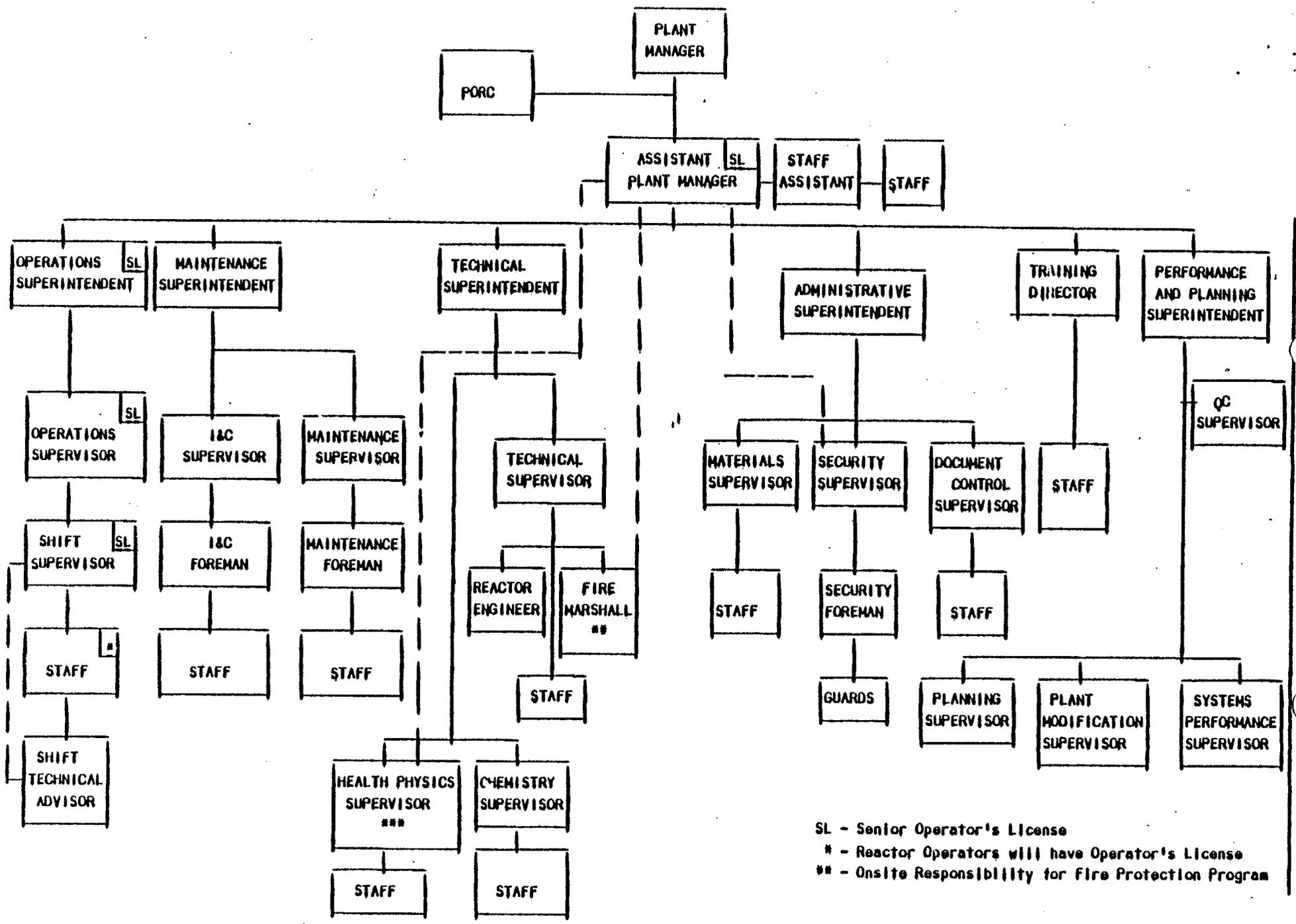
SURVEILLANCE REQUIREMENT (Continued)

- e. At least once per 60 month, during a shutdown of either unit, by verifying that the battery capacity is at least 80% of the manufacturer's rating when subjected to a performance discharge test. This performance discharge test shall be performed subsequent to the satisfactory completion of the required battery service test.



\*Corporate responsibility for fire protection program

Figure 6.2-1 Offsite Organization for Facility Management and Technical Support



SL - Senior Operator's License  
 \* - Reactor Operators will have Operator's License  
 \*\* - Onsite Responsibility for Fire Protection Program

FIGURE 6.2-2 FACILITY ORGANIZATION

\*\*\* In routine matters, the Health Physics Supervisor reports directly to the Technical Superintendent, In matters of radiation policy determination, interpretation or implementation (based upon the Health Physics Supervisor's judgement) the Health Physics Supervisor may report directly to the Assistant Plant Manager.

## ADMINISTRATIVE CONTROLS

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### 6.2.3 SAFETY AUDIT AND ENGINEERING REVIEW GROUP (SAERG)

#### FUNCTION

6.2.3.1 The SAERG shall function to conduct operational evaluations, engineering reviews, and audits for the purpose of improving safety.

#### COMPOSITION

6.2.3.2 The SAERG shall be composed of a multi-disciplined dedicated onsite group with minimum assigned complement of five engineers or appropriate specialists.

#### RESPONSIBILITIES

6.2.3.3 The SAERG shall be responsible for the following:

- a. Participating in operational evaluations for improvement of safety wherein such evaluations and recommendations therefrom are not limited to the fulfillment of existing programs, policies, procedures, or capabilities of existing equipment and installations.
- b. Systematic engineering reviews of plant performance and activities with results reported independently of onsite operational management to offsite upper management.
- c. Comprehensive plant audits in accordance with audit requirements set forth in quality assurance programs, licensing documents, and other policies and procedures.

#### AUTHORITY

6.2.3.4 The onsite SAERG shall carry out its function reporting offsite directly to the Manager-Safety Audit and Engineering Review who in turn reports directly to the Vice President-Nuclear Generation.

### 6.2.4 SHIFT TECHNICAL ADVISOR

6.2.4.1 The Shift Technical Advisor shall serve in an advisory capacity to the Shift Supervisor primarily in the assessment of accident and transient occurrences.

### 6.3 FACILITY STAFF QUALIFICATIONS

6.3.1 Each member of the facility staff shall meet or exceed the minimum qualifications of ANSI N18.1-1971 for comparable positions and the supplemental requirements specified in Sections A and C of Enclosure 1 of the March 28, 1980 NRC letter to all licensees, except for (1) the Health Physics Supervisor who shall meet or exceed the qualifications of Regulatory Guide 1.8, September 1975.

## ADMINISTRATIVE CONTROLS

### AUTHORITY

6.5.1.7 The PORC shall:

- a. Recommend to the Plant Manager in writing, approval or disapproval of items considered under 6.5.1.6(a) through (e) and (j) and (k) above.
- b. Render determinations in writing with regard to whether or not each item considered under 6.5.1.6(a), (C) and (d) above constitutes an unreviewed safety question.
- c. Make recommendations to the Plant Manager in writing that actions reviewed under 6.5.1.6(b) above did not constitute an unreviewed safety question.

### RECORDS

6.5.1.8 The PORC shall maintain written minutes of each meeting and copies shall be provided to the Vice President-Nuclear Generation and Chairman of the Nuclear Operations Review Board.

### 6.5.2 NUCLEAR OPERATIONS REVIEW BOARD (NORB)

#### FUNCTION

6.5.2.1 The NORB shall function to provide independent review and audit of designated activities in the areas of:

- a. Nuclear power plant operations
- b. Nuclear engineering
- c. Chemistry and radiochemistry
- d. Metallurgy
- e. Instrumentation and control
- f. Radiological safety
- g. Mechanical and electrical engineering
- h. Quality assurance practices

#### COMPOSITION

6.5.2.2 The NORB shall be composed of at least five persons including:

Chairman:	Senior Vice President
Vice Chairman:	Vice President-Nuclear Generation
Secretary:	Manager-Safety Audit & Engineering Review
Member:	Manager-Nuclear Operations and Administration
Member:	Manager-Nuclear Engineering and Technical Support

and other appointed personnel having an academic degree in an engineering or physical science field and a minimum of five years technical experience, of which a minimum of three years shall be in one or more of the areas given in 6.5.2.1.

## ADMINISTRATIVE CONTROLS

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### ALTERNATES

6.5.2.3 All alternate members shall be appointed in writing by the NORB Chairman to serve on a temporary basis; however, no more than two alternates shall participate as voting members in NORB activities at any one time.

### CONSULTANTS

6.5.2.4 Consultants shall be utilized as determined by the NORB Chairman to provide expert advice to the NORB.

### MEETING FREQUENCY

6.5.2.5 The NORB shall meet at least once per calendar quarter during the initial year of unit operation following fuel loading and at least once per six months thereafter.

### QUORUM

6.5.2.6 A quorum shall consist of the Chairman or Vice Chairman plus enough voting members to constitute a majority of the NORB. No more than a minority of the quorum shall have line responsibility for operation of the facility. For the purpose of a quorum those considered to have line responsibility will include the Plant Manager and personnel reporting to the Plant Manager.

### REVIEW

6.5.2.7 The NORB shall review:

- a. Proposed changes to procedures, equipment or systems which involve an unreviewed safety question as defined in Section 50.59, 10 CFR.
- b. Proposed tests or experiments which involve an unreviewed safety question as defined in Section 50.59, 10 CFR.
- c. Violations of codes, regulations, orders, Technical Specifications, license requirements, or of internal procedures or instructions having nuclear safety significance or abnormal degradation of systems designed to contain radioactive material.
- d. Significant operating abnormalities or deviations from normal and expected performance of plant equipment that affect nuclear safety.
- e. Written reports concerning events requiring 24 hour notification to the Commission.
- f. Recognized indications of an unanticipated deficiency in some aspect of design or operation of safety related structures, systems, or components.

## ADMINISTRATIVE CONTROLS

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- g. Reports and meetings minutes of the PORC.
- h. Proposed changes to Technical Specifications or this Operating License.
- i. The safety evaluations for proposed 1) procedures 2) changes to procedures, equipment or systems and 3) test or experiments completed under the provision of Section 50.59 10 CFR, to verify that such actions did not constitute an unreviewed safety question.

## AUDITS

6.5.2.8 The following audits shall be conducted under the direction of APCO's Manager - Safety Audit and Engineering Review:

- a. The conformance of facility operation to provisions contained within the Technical Specifications and applicable license conditions at least once per 12 months.
- b. The performance, training and qualifications of the entire facility staff at least once per 12 months.
- c. The results of actions taken to correct deficiencies occurring in facility equipment, structures, systems or method of operation that affect nuclear safety at least once per 6 months.
- d. The performance of activities required by the Operational Quality Assurance Program to meet the criteria of Appendix "B", 10 CFR 50, at least once per 24 months.
- e. The Facility Emergency Plan at least once per 24 months.
- f. The Facility Security Plan at least once per 12 months.
- g. Any other area of facility operation considered appropriate by the NORB or the Senior Vice President.
- h. The Facility Fire Protection Program and implementing procedures at least once per 24 months.
- i. An independent fire protection and loss prevention program inspection and audit of the unit at least once per 12 months utilizing either qualified offsite licensee personnel or an outside fire protection firm.
- j. An inspection and audit of the unit fire protection and loss prevention program by a qualified outside fire consultant at least once per 36 months.
- k. The radiological effluent and environmental monitoring programs and the results thereof at least once per 12 months.

## ADMINISTRATIVE CONTROLS

Senior Reactor Operator's License. For changes to procedures which may involve a change in intent of the approved procedures, the person authorized above to approve the procedure shall approve the change.

- b. Proposed changes or modifications to plant nuclear safety-related structures, systems and components shall be reviewed as designated by the Plant Manager. Each such modification shall be reviewed by an individual/group other than the individual/group which designed the modification, but who may be from the same organization as the individual/group which designed the modifications. Proposed modifications to plant nuclear safety-related structures, systems and components shall be approved prior to implementation by the Plant Manager.
- c. Proposed tests and experiments which affect plant nuclear safety and are not addressed in the Final Safety Analysis Report shall be prepared, reviewed, and approved. Each such test or experiment shall be reviewed by an individual/group other than the individual/group which prepared the proposed test or experiment. Proposed test and experiments shall be approved before implementation by the Plant Manager.
- d. Occurrences reportable pursuant to the Technical Specification 6.0 and violations of Technical Specifications shall be investigated and a report prepared which evaluates the occurrence and which provides recommendations to prevent recurrence. Such reports shall be approved by the Plant Manager and forwarded to the Vice President, Nuclear Generation with a copy to the Manager of Nuclear Operations and Administration; and to the Chairman of the Nuclear Operations Review Board.
- e. Individuals responsible for reviews performed in accordance with 6.5.3.1.a, 6.5.3.1.b, 6.5.3.1.c, and 6.5.3.1.d shall be members of the plant supervisory staff previously designated by the Plant Manager. Each such review shall include a determination of whether or not additional, cross-disciplinary, review is necessary. If deemed necessary, such review shall be performed by the review personnel of the appropriate discipline.
- f. Each review will include a determination of whether or not an unreviewed safety question is involved. Pursuant to 10 CFR 50.59 NRC approval of items involving unreviewed safety question will be obtained prior to Plant Manager approval for implementation.

## RECORDS

6.5.3.2 Records of the above activities shall be provided to the Plant Manager, PORC and/or NORB as necessary for required reviews.

## ADMINISTRATIVE CONTROLS

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### 6.6 REPORTABLE OCCURRENCE ACTION

- 6.6.1 The following actions shall be taken for REPORTABLE OCCURRENCES:
- a. The Commission shall be notified and/or a report submitted pursuant to the requirements of Specification 6.9.
  - b. Each REPORTABLE OCCURRENCE requiring 24 hour notification to the Commission shall be reviewed by the PORC and submitted to the NORB and the Vice President - Nuclear Generation with a copy to the Manager of Nuclear Operations and Administration.

### 6.7 SAFETY LIMIT VIOLATION

- 6.7.1 The following actions shall be taken in the event a Safety Limit is violated:
- a. The facility shall be placed in at least HOT STANDBY within one hour.
  - b. The NRC Operations Center shall be notified by telephone as soon as possible and all cases within one hour. The Vice President, Nuclear Generation shall be notified within 24 hours.
  - c. A Safety Limit Violation Report shall be prepared. The report shall be reviewed by the PORC. This report shall describe (1) applicable circumstances preceding the violation, (2) effects of the violation upon facility components, systems or structures, and (3) corrective action taken to prevent recurrence.
  - d. The Safety Limit Violation Report shall be submitted to the Commission, and the Vice President, Nuclear Generation, with a copy to the Manager of Nuclear Operations and Administration, for NORB review within 14 days of the violation.

### 6.8 PROCEDURES AND PROGRAMS

- 6.8.1 Written procedures shall be established, implemented and maintained covering the activities reference below:
- a. The applicable procedures recommended in Appendix "A" of Regulatory Guide 1.33, Revision 2, 1978.
  - b. Refueling operations.
  - c. Surveillance and test activities of safety related equipment.
  - d. Security Plan implementation.
  - e. Emergency Plan implementation.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

OCT 14 1983

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
RELATED TO AMENDMENT NO.33 TO FACILITY OPERATING LICENSE NO. NPF-2  
AND NO.26 FACILITY OPERATING LICENSE NO. NPF-8  
ALABAMA POWER COMPANY  
JOSEPH M. FARLEY NUCLEAR PLANT UNIT NOS. 1 AND 2  
DOCKET NOS. 50-348 AND 50-364

## INTRODUCTION

By letter dated November 16, 1982, supplemented by letters dated February 1, 1983, June 29, 1983 and August 23, 1983 the licensee, Alabama Power Company (APCO) proposed administrative changes to the technical specifications (TS) for the Farley Nuclear Plant, Unit Nos. 1 and 2. The changes would revise administrative controls for: onsite and offsite organization, environmental qualifications of safety-related electrical equipment, audit frequency of the Facility Security Plan, and revise LCO and surveillance for instrumentation, fire protection, electrical power systems, emergency core cooling system (ECCS), containment system and would delete TS 7.1 for Unit 2 only requiring an augmented low power test program.

### ADMINISTRATIVE CONTROLS - ONSITE AND OFFSITE ORGANIZATION

(For each unit - Specifications 6.2.1 (Fig. 6.2-1), 6.2.2 (Fig. 6.2-2), 6.3.1, 6.5.2.2, 6.5.2.6, 6.5.3.1(d), 6.6.1(b), and 6.7.1(b) and (d))

### Discussion and Evaluation

In their letter of November 16, 1982, and in the supplemental letter of June 29, 1983, APCO proposed changes to the Administrative Controls Technical Specifications for each unit, citing corporate reorganization as reason. Formerly, the Plant Manager was directly responsible to the General Manager-Nuclear Generation, who was in turn responsible to the Vice President-Nuclear Generation. In the reorganization, the Plant Manager was made directly responsible to the Vice President-Nuclear Generation and the position of General Manager-Nuclear Generation was retitled "Manager-Nuclear Operations and Administration". The Manager-Nuclear Operations and Administration, still directly responsible to the Vice President-Nuclear Generation, was removed from line responsibility for the facility, altering the quorum for the advisory Nuclear Operations Review Board. APCO specifically proposed changing in each unit's Technical Specifications: Figure 6.2-1 of Specification 6.2.1, the "Offsite Organization for Facility Management and Technical Support" chart, to reflect the changed title and line responsibility; the General Manager's title in Technical Specifications 6.5.2.2, 6.5.3.1(d), and 6.7.1(d); and the personnel responsible for line operation in Technical Specifications 6.5.2.6, 6.5.3.1(d), 6.6.1(b), and 6.7.1(b) and (d).

In their letter of February 1, 1983, APCO proposed for each unit changes to Figure 6.2-2, the onsite "Facility Organization" chart, to reflect the addition of a Quality Control Supervisor and a Plant Modifications Supervisor. In their

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letter of August 23, 1983, APCO proposed additional changes to Figure 6.2-2, and a change to Technical Specification 6.3.1, which specifies minimum facility staff qualifications. These changes are to reflect APCO's having created two positions, the Health Physics Supervisor and the Chemistry Supervisor, from a former position which combined the two roles. The proposed chart reflects that the Health Physics Supervisor reports to the Technical Superintendent except for in matters of radiation policy, where he/she reports to the Assistant Plant Manager. APCO reported one other change to Figure 6.2-2, this to show the change of the title of the Training Supervisor to "Training Director".

The reorganizational changes that allow the Plant Manager to report directly to the corporate officer responsible for the facility will not diminish plant supervision. No minimum job qualifications specified in the FSAR were lessened in the reorganization and addition of personnel; the personnel additions would in fact allow for an increase in the supervision of the facility. All of the changes proposed to the Administrative Controls Technical Specifications are administrative, and are necessary in order to reflect the present facility personnel organization.

The changes are accepted as proposed by the licensee, and are incorporated into the Technical Specifications.

#### ADMINISTRATIVE CONTROLS - ENVIRONMENTAL QUALIFICATIONS (Specification 6.16 - Unit No. 1 only)

##### Discussion and Evaluation

In their letter of February 1, 1983, APCO cited 10 CFR 50.49, which suspends the June 30, 1982 deadline for completion of the environmental qualification of safety-related electrical equipment, and the recordkeeping which would have been required for the program. APCO proposed the deletion of the Unit No. 1 Administrative Controls Technical Specification that calls for compliance with the now suspended regulation deadline. APCO's proposal is acceptable as 10 CFR 50.49 supersedes the Technical Specification.

The proposed is accepted as submitted by the licensee. Technical Specification 6.16 in license NPF-2 (Unit 1) is deleted.

#### ADMINISTRATIVE CONTROLS - AUDITS (Specification 6.5.2.8(f) for each unit)

##### Discussion and Evaluation

In their letter of February 1, 1983, APCO proposed a change to the Administrative Controls Technical Specification 6.5.2.8(f) for each unit, citing in 10 CFR 73.55(g)(4) the requirement that the Facility Security Plan be audited every 12 months, not every 24 months as instructed in the Specification. The proposed change is acceptable since the Technical Specifications should be updated to conform with the Code of Federal Regulations.

For each unit, Specification 6.5.2.8(f) is accepted as submitted by APCO and replaces the existing Specification.

#### LCOs AND SURVEILLANCE - INSTRUMENTATION

(For each unit - Specifications 3.3.2 (Table 3.3-3), 3.3.3.7 (Table 3.3-10), and 3.3.3.9 (Table 3.3-12))

##### Discussion and Evaluation

APCO proposed in their letter dated February 1, 1983, a change to each unit's Technical Specification 3.3.2 (Table 3.3-3); in each Action Statement 20 the Pressurizer Pressure Interlock is misidentified as "P-4". APCO stated that the errors should be corrected to read as "P-11". These proposed corrections are administrative in nature; they do not affect the performance of a system and are necessary in order to reflect plant design.

APCO also proposed changes to Technical Specifications 3.3.3.7 (Table 3.3-10) and 3.3.3.9 (Table 3.3-12) for both units, respectively dealing with High Energy Line Break Sensor and Fire Detector locations. The proposed changes to Table 3.3-10 are administrative, correcting the sensor location descriptions as to agree with as-built conditions, noting sensors which service both units, and Unit No. 2 sensors which alarm in the Unit No. 1 control room area. These notes were modified by the staff with the licensee's concurrence. The changes to Table 3.3-12 are also administrative, these being: the additional tabulation of sensors added in order to comply with NRC regulations; the correction of two room descriptions in which sensors are located (Unit No. 2 only); and changes to Table 3.3-12 making more clear the fire detector locations for both units. The changes to both Tables 3.3-10 and 3.3-12 are acceptable; each change in no manner affects the performance of a system and is necessary to update the Technical Specifications.

The proposed changes are accepted as submitted and amend the corresponding Specifications for each unit.

LCO's AND SURVEILLANCE - FIRE PROTECTION - YARD FIRE HYDRANTS, HYDRANT HOSE STATIONS (Specification 3.7.11.5 (Table 3.7-7))

##### Discussion and Evaluation

APCO proposed in the letter dated February 1, 1983, to amend the Technical Specification 3.7.11.5 (Table 3.7-7) for each unit noting that two Unit No. 2's fire hydrants were mistakenly listed in the Unit No. 1 Technical Specifications, and that a fire hydrant servicing both units was misnumbered in the Unit No. 2 Technical Specifications. APCO also suggested denoting with asterisks and footnotes the hydrants shared by both units. The Staff in review corrected the listed location of a hydrant in the Unit No. 2 table; Hydrant N1Y43102 is northwest, not north, of the Auxiliary Building. The Staff also noted that the "List of Sprinkler Systems" on Page 3/4 7-87 of the Unit No. 1 Technical Specifications was titled "Table 3.7-7"; this is an error. The title was corrected to read "Table 3.7-5". The actual Table 3.7-7 appears on page 3/4 7-93 of the Unit No. 1 Technical Specifications.

Based on the above discussion the proposed administrative changes are acceptable

#### LCOs AND SURVEILLANCE - ELECTRICAL POWER SYSTEMS

(For each unit - Specifications 3.8.2.5 and 4.8.2.5.2)

### Discussion and Evaluation

APCO noted in their letter of February 1, 1983, that Technical Specifications 4.8.2.5.2(d), 4.8.2.5.2(e), and 3.8.2.5 (Action Statement) require that the plant must be in shutdown within eight hours after one train of the Service Water Building D.C. Distribution System becomes inoperable. Since Units No. 1 and No. 2 share a single system, the Technical Specification for each unit incorrectly implies that both units must be in shutdown during a battery discharge test. The intent of the Technical Specifications is not to require that a train be tested only during a shutdown of both units, but that during the discharge and recharge steps of a test, that at least one unit must be in shutdown. With the exemption delineated in the Technical Specification 3.8.2.5 ACTION Statement for each unit - which allows for the brief inoperability of a single D.C. distribution system - there must be at all times at least one operable train per operating unit.

APCO proposed the deletion of the words "during shutdown" in 4.8.2.5.2(d) and (e) of the Technical Specifications of each unit. This deletion would not be administrative as APCO stated in their letter of February 1, 1983, but would be a technical change. This change is unacceptable, since it would allow for the operation of both units for a considerable time period with but one operable train; also, the discharge tests should be performed during regularly scheduled shutdowns, and there should be no need to test the batteries during the operation of both units.

The Technical Specification for each unit will be corrected by an administrative change in replacing the words "during shutdown" in 4.8.2.5.2(d) and (e) with "during a shutdown of either unit"; as well, APCO's proposed footnote to the 3.8.2.5 ACTION STATEMENT - which allows for the testing described in 4.8.2.5.2(d) and (e) without the shutdown of both units - will be added as submitted. These changes will relieve the licensee of unreasonable hardship, yet will take safety into account by requiring that at least one unit is to be in shutdown during a battery bank and charger test. The licensee has a choice of testing Train "A" during one unit's shutdown and Train "B" during the other unit's shutdown, or testing both trains during the shutdown of single unit (without having both trains inoperable concurrently in either scheme).

With the modifications suggested by the Staff and agreed to by the licensee, the submitted administrative changes to each unit's Specifications are accepted and amend the existing Specifications.

### LCO's AND SURVEILLANCE - ECCS

(For each unit - Specifications 4.5.1.1(c), 4.5.2(a) and 4.5.2(d))

### Discussion and Evaluation

In the letter of February 1, 1983, APCO proposed administrative changes to Technical Specifications 4.5.1.1(c) and 4.5.2(a), noting that automatic disconnect devices had been installed for several ECCS valves within the containment area of each unit in accordance with NUREG-0737, Item II.B.2. APCO suggested that the Specifications should, for clarity, read "disconnect device" instead of "breaker". APCO also proposed changing 4.5.2(d) for each unit, to reflect that each RHR system automatic isolation and interlock action is verified between 700 and 750 psig, not "above 750 psig" as stated in the Specifications. APCO also stated that the footnote to the Unit No. 2 Technical Specification 4.5.2(a) should be edited to read "pump 2A" instead of "pump 1A"; this will be an administrative change. The Staff agrees with APCO that the above changes are justified, and will not affect the performance of a system.

The change proposals are accepted as submitted by APCO and amend the existing Technical Specifications for each unit.

LCOs AND SURVEILLANCE - CONTAINMENT SYSTEM  
(For Unit No. 1 - Specifications 3.6.1.7 and 4.6.1.7.2)

### Discussion and Evaluation

APCO proposed in their letter dated February 1, 1983, the changing of Technical Specifications 3.6.1.7 and 4.6.1.7.2 for each unit to read "8 inch containment vent" (valves) instead of "18 inch containment purge" (3.6.1.7) and "18 inch purge" (4.6.1.7.2). APCO is replacing, in the current fourth refueling outage, the Unit No. 1 purge valves with vent valves in compliance with NUREG-0737, Item II.E.4.2. The changes would reflect this compliance.

The proposed changes are accepted as submitted and are incorporated into the Technical Specifications for each unit.

AUGMENTED LOW POWER TEST PROGRAM  
(Specification 7.1 For Unit No. 2 only)

### Discussion and Evaluation

APCO proposed, in their letter dated February 1, 1983, the deletion of the Technical Specification 7.1 for Unit No. 2, which had exempted the licensee from following the requirements identified in Table A of the NRC Safety Evaluation enclosed with Amendment 2 to license NPF-3. The test program was performed in 1981; the exemptions are no longer needed and Specification 7.1 should be deleted.

Based on the above discussion we find the deletion of Technical Specification 7.1 for Unit 2 acceptable.

ENVIRONMENTAL CONSIDERATION

We have determined that the two amendments do not authorize changes in effluent types or total amounts, nor increases in power levels, and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendments involve actions which are insignificant from the standpoint of environmental impact and, pursuant to 10 CFR 51.5(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of these amendments.

CONCLUSION

We have concluded, based on the considerations discussed above, that:  
(1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and  
(2) such activities will be conducted in compliance with the Commission's regulations and the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Dated: October 14, 1983

Principal Contributor: Frank Watts