November 20, 2001

Mr. Harold W. Keiser
Chief Nuclear Officer & President
PSEG Nuclear LLC - X04
Post Office Box 236
Hancocks Bridge, NJ 08038

SUBJECT: SALEM NUCLEAR GENERATING STATION, UNIT NOS. 1 AND 2, RE: RESPONSE TO BULLETIN 2001-01, "CIRCUMFERENTIAL CRACKING OF REACTOR PRESSURE VESSEL HEAD PENETRATION NOZZLES" (TAC NOS. MB2655 AND MB2656)

Dear Mr. Keiser:

On August 3, 2001, the staff issued Bulletin 2001-01, "Circumferential Cracking of Reactor Pressure Vessel Head Penetration Nozzles," to the industry requesting that addressees provide information related to the structural integrity of the reactor pressure vessel head penetration (VHP) nozzles for their respective facilities, including the extent of VHP nozzle leakage and cracking that has been found to date, the inspections and repairs that have been undertaken to satisfy applicable regulatory requirements, and the basis for concluding that their plans for future inspections will ensure compliance with applicable regulatory requirements at their respective pressurized water reactor (PWR) plants. You were requested to respond to Items 1 and 4 of the Bulletin within 30 days of its issuance.

By letter dated August 31, 2001, PSEG Nuclear LLC (PSEG) provided its response for the Salem Nuclear Generating Station, Unit Nos. 1 and 2 (Salem). The staff finds that you have provided the requested information, and there is reasonable assurance that the public health and safety will be maintained. The proposed inspection scope and schedule described in your response, which included the description of your plan to examine "all" or "100%" of the VHPs at your facility during the next scheduled outage, were integral to the staff's finding. The staff expects that you will submit a revised response to the Bulletin if you make any substantive changes to the schedule and/or scope of future inspections for Salem. If warranted by such changes, the staff will reevaluate this issue.

PSEG is reminded that Item 5 of the Bulletin requested the following information within 30 days after plant restart following the next refueling outage:

a. A description of the extent of VHP nozzle leakage and cracking detected at your plant, including the number, location, size, and nature of each crack detected;
b. If cracking is identified, a description of the inspections (type, scope, qualification requirements, and acceptance criteria), repairs, and other corrective actions you have taken to satisfy applicable regulatory requirements. This information is requested only if there are any changes from prior information submitted in accordance with this bulletin.

If you have any questions, please call me at 301-415-1324, or email me at rxf@nrc.gov.

Sincerely,

/RA/

Robert J. Fretz, Project Manager, Section 2
Project Directorate I
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket Nos. 50-272 and 50-311

cc: See next page

If you have any questions, please call me at 301-415-1324, or email me at rxf@nrc.gov.

Sincerely,

/RA/

Robert J. Fretz, Project Manager, Section 2
Project Directorate I
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket Nos. 50-272 and 50-311

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NAME	RFretz	TLClark	JClifford
DATE	11/9/01	11/9/01	11/15/01

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PSEG Nuclear LLC

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