Docket Sic

DECEMBER 10 1980

Docket No. 50-348

Mr. F. L. Clayton
Senior Vice President
Alabama Power Company
Post Office Box 2641
Birmingham, Alabama 35291

Dear Mr. Clayton:

The Commission has issued the enclosed Amendment No. 17 to Facility Operating License No. NPF-2 for the Joseph M. Farley Nuclear Plant, Unit No. 1. The amendment consists of changes to the Technical Specifications in response to your application transmitted by letters dated October 15, 1979 (modified by letter dated October 3, 1980) and October 23, 1979.

The amendment provides the following:

1. Revised Administrative Controls Technical Specifications for entry into high radiation areas;

2. Added feedwater control system bypass valves response times to Technical Specifications; and

3. Added license condition relating to the approved "Joseph M. Farley Nuclear Contingency Plan." This condition is a followup action to our May 1, 1980 letter which approved the plan.

Minor changes were made to some of your proposals. These changes have been discussed with your staff who concur with our changes.

Our letter dated May 1, 1980 approved the Safeguards Contingency Plan dated March 23, 1979 as revised October 8, 1979 and March 20, 1980. Our action at this time adds a requirement to the license to fully implement and maintain in effect all provisions of the approved plan, including amendments and changes made pursuant to the authority of 10 CFR 50.54(p). We will continue to withhold your Contingency Plan and related materials from public disclosure in accordance with the provisions of 10 CFR 2.790(d).

Our Order of October 24, 1980 issued Technical Specification pages 6-19 and 6-22 relating to Environmental Qualifications of safety-related electrical equipment. These pages plus index page XVIII have been reissued in this letter to correct the page numbers and are transmitted herewith for insertion into the Technical Specifications.

OFFICE ▶	8101200 226					
SURNAME ▶						
DATE ▶						

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Copies of the Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

Original signed by:
S. A. Varga

Steven A. Varga, Chief
Operating Reactors Branch #1
Division of Licensing

Enclosures:
1. Amendment No. 17 to NPF-1
2. Safety Evaluation
3. Notice of Issuance

cc: w/enclosures
See next page

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	11/19/80	11/20/80	11/ /80

DB - no legal objection - note typo on Fed. Reg. notice

OFFICE	COMPLETES TACS 10471, 12502 and 12569	DL:ORB1	DL:ORB1	DL:ORB1
SURNAME		EAReeves:jb	CSParrish	SAVarga
DATE		11/14/80	11/14/80	11/15/80

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

December 10, 1980

Docket No. 50-348

Mr. F. L. Clayton
Senior Vice President
Alabama Power Company
Post Office Box 2641
Birmingham, Alabama 35291

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Minor changes were made to some of your proposals. These changes have been discussed with your staff who concur with our changes.

Our letter dated May 1, 1980 approved the Safeguards Contingency Plan dated March 23, 1979 as revised October 8, 1979 and March 20, 1980. Our action at this time adds a requirement to the license to fully implement and maintain in effect all provisions of the approved plan, including amendments and changes made pursuant to the authority of 10 CFR 50.54(p). We will continue to withhold your Contingency Plan and related materials from public disclosure in accordance with the provisions of 10 CFR 2.790(d).

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Copies of the Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

Steven A. Varga, Chief
Operating Reactors Branch #1
Division of Licensing

Enclosures:
1. Amendment No. 17 to NPF-1
2. Safety Evaluation
3. Notice of Issuance

cc: w/enclosures
See next page

cc: Mr. W. O. Whitt
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State Department of Public Health
ATTN: State Health Officer
State Office Building
Montgomery, Alabama 36104

Director, Criteria and Standards Division
Office of Radiation Programs (ANR-460)
U. S. Environmental Protection Agency
Washington, D. C. 20460

U. S. Environmental Protection Agency
Region IV Office
ATTN: EIS COORDINATOR
345 Courtland Street, N.E.
Atlanta, Georgia 30308

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

ALABAMA POWER COMPANY

DOCKET NO. 50-348

JOSEPH M. FARLEY NUCLEAR PLANT, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 17
License No. NPF-2

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The applications for amendment by Alabama Power Company (the licensee) dated October 15, 1979 (modified by letter dated October 3, 1980) and October 23, 1979, in addition to the licensee's filing dated March 23, 1979, as revised October 8, 1979 and March 20, 1980, comply with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application and filing, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-2 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 17 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

(3) Redesignate paragaph 2.D as 2.D.(1) and add paragraph 2.D.(2) to read as follows:

2.D.(2) The licensee shall fully implement and maintain in effect all provisions of the Commission-approved Safeguards Contingency Plan, including amendments and changes made pursuant to the authority of 10 CFR 50.54(p). The approved Contingency Plan consists of documents withheld from public disclosure pursuant to 10 CFR 2.790(d) identified as "Joseph M. Farley Safeguards Contingency Plan" dated March 23, 1979 as revised October 8, 1979 and March 20, 1980, submitted pursuant to 10 CFR 73.40. The Contingency Plan was to have been fully implemented, in accordance with 10 CFR 73.40(b), within 30 days of the approval by the Commission which was dated May 1, 1980.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Steven A. Varga, Chief
Operating Reactors Branch #1
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: December 10, 1980

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 17 TO FACILITY OPERATING LICENSE NO. NPF-1

DOCKET NO. 50-348

Revise Appendix A as follows:

Remove Pages	Insert Pages
XVIII	XVIII
3/4 3-30	3/4 3-30
6-19	6-19
6-20	6-20
----	6-21

INDEX

ADMINISTRATIVE CONTROLS

INDEX

ADMINISTRATIVE CONTROLS

TABLE 3.3-5 (Continued)

ENGINEERED SAFETY FEATURES RESPONSE TIMES

INITIATING SIGNAL AND FUNCTION	RESPONSE TIME IN SECONDS
7. Containment Pressure--High-High	
a. Steam Line Isolation	$\leq$ 7.0
8. Containment Pressure--High-High-High	
a. Containment Spray	$\leq$ 45.0
b. Containment Isolation-Phase "B"	Not Applicable
9. Steam Generator Water Level--High-High	
a. Turbine Trip-Reactor Trip	$\leq$ 2.5
b. Feedwater Isolation	$\leq$ 32.0###
10. Steam Generator Water Level--Low-Low	
a. Motor-driven Auxiliary Feedwater Pumps**	60.0
b. Turbine-driven Auxiliary Feedwater Pumps***	60.0
11. Undervoltage RCP	
a. Turbine-driven Auxiliary	60.0
12. S.I. Signal	
a. Motor-driven Auxiliary Feedwater Pump	60.0

**On 2/3 Any Steam Generator

***On 2/3 in 2/3 Steam Generators

TABLE 3.3-5 (Continued)

TABLE NOTATION

* Diesel generator starting and sequence loading delays included. Response time limit includes opening of valves to establish SI path and attainment of discharge pressure for centrifugal charging pumps and RHR pumps.

Diesel generator starting and sequence loading delays not included. Offsite power available. Response time limit includes opening of valves to establish SI path and attainment of discharge pressure for centrifugal charging pumps.

Diesel generator starting and sequence loading delays included. Response time limit includes opening of valves to establish SI path and attainment of discharge pressure for centrifugal charging pumps.

Verification shall include testing of all instrumentation, the isolation valves (MOV-3232A, 3232B, 3232C), the control valves (FCV-478, 488, 498) and the bypass valves (FVC-479, 489, 499). The isolation valves must function within 30 seconds and the control valves and bypass valves within 5 seconds.

e. Records of gaseous and liquid radioactive material released to the environs.

f. Records of transient or operational cycles for those facility components identified in Table 5.7-1.

g. Records of reactor tests and equipments.

h. Records of training and qualification for current members of the plant staff.

i. Records of in-service inspections performed pursuant to these Technical Specifications.

j. Records of Quality Assurance activities required by the QA Manual.

k. Records of reviews performed for changes made to procedures or equipment or reviews of tests and experiments pursuant to 10 CFR 50.59.

l. Records of meetings of the PORC and the NORB.

m. Records of secondary water sampling and water quality.

n. Records for Environmental Qualification which are covered under the provisions of paragraph 6.13.

6.11 RADIATION PROTECTION PROGRAM

Procedures for personnel radiation protection shall be prepared consistent with the requirements of 10 CFR Part 20 and shall be approved, maintained and adhered to for all operations involving personnel radiation exposure.

6.12 HIGH RADIATION AREA

6.12.1 In lieu of the "control device" or "alarm signal" required by paragraph 20.203(c)(2) of 10 CFR 20, each high radiation area in which the intensity of radiation is greater than 100 mrem/hr but less than 1000 mrem/hr shall be barricaded and conspicuously posted as a high radiation area and entrance thereto shall be controlled by requiring issuance of a Radiation Work Permit (RWP).* An individual or group of individuals permitted to enter such areas shall be provided with one or more of the following:

*Health Physics personnel or personnel escorted by health physics personnel shall be exempt from the RWP issuance requirement during the performance of their assigned radiation protection duties, provided they are following plant radiation protection procedures for entry into high radiation areas.

a) A radiation monitoring device which continuously indicates the radiation dose rate in the area.

b) A radiation monitoring device which continuously integrates the radiation dose rate in the area and alarms when a pre-set integrated dose is received. Entry into such areas with this monitoring device may be made after the dose rate levels in the area have been established and personnel have been made knowledgeable of them.

c) A health physics qualified individual (i.e., qualified in radiation protection procedures) with a radiation dose rate monitoring device who is responsible for providing positive exposure control over the activities within the area and who will perform periodic radiation surveillance at the frequency specified in the RWP. The surveillance frequency will be established by the Health Physics Supervisor.

6.12.2 In addition to the requirements of 6.12.1, areas with radiation levels greater than 1000 mrem/hr shall be provided with locked doors to prevent unauthorized entry, and the keys shall be maintained under the administrative control of the shift foreman and/or health physics supervision. Doors shall remain locked except during periods of access by personnel under an approved Radiation Work Permit (RWP) which shall specify the dose rate levels in the immediate work area and the maximum allowable stay time for individuals in that area. For individual areas with radiation levels in excess of 1000 mrem/hr* that are located within large areas, such as PWR containment, where no enclosure exists for purposes of locking, and no enclosure can be reasonably constructed around the individual areas, then that area shall be roped off, conspicuously posted and a flashing light shall be activated as a warning device. In lieu of the stay time specification of the RWP, direct or remote (such as use of closed circuit TV cameras) continuous surveillance may be made by personnel qualified in radiation protection procedures to provide positive exposure control over the activities within the area.

6.13 ENVIRONMENTAL QUALIFICATION

6.13.1 By no later than June 30, 1982 all safety-related electrical equipment in the facility shall be qualified in accordance with the provisions of: Division of Operating Reactors "Guidelines for Evaluating Environmental Qualification of Class 1E Electrical Equipment in Operating Reactors" (DOR Guidelines); or, NUREG-0588 "Interim Staff Position on Environmental Qualification of Safety-Related Electrical Equipment," December 1979. Copies of these documents are attached to the Order for Modification of License No. NPF-2 dated October 24, 1980.

*Measurement made at 18" from source of radioactivity.

6.13.2 By no later than December 1, 1980, complete and auditible records must be available and maintained at a central location which describe the environmental qualification method used for all safety-related electrical equipment in sufficient detail to document the degree of compliance with the DOR Guidelines or NUREG-0588. Thereafter, such records should be updated and maintained current as equipment is replaced, further tested, or otherwise further qualified.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 17 TO FACILITY OPERATING LICENSE NO. NPF-2

ALABAMA POWER COMPANY

JOSEPH M. FARLEY NUCLEAR PLANT, UNIT NO. 1

DOCKET NO. 50-348

Introduction

Alabama Power Company (APCO), the licensee for the Joseph M. Farley Nuclear Plant, Unit No. 1, proposed changes to Operating License No. NPF-2. These changes are included in APCO letters dated October 15, 1979 (modified by letter dated October 3, 1980), and October 23, 1979. Our followup action to our May 1, 1980 letter approving the Safeguards Contingency Plan is also discussed herein. The license changes included in this amendment and discussed below are as follows:

1. Revised Administrative Controls Technical Specifications for entry into high radiation areas;

2. Added feedwater control system bypass valves response times to Technical Specifications; and

3. Added license condition relating to the approved "Joseph M. Farley Nuclear Contingency Plan." This condition is a followup action to our May 1, 1980 letter which approved the plan.

ADMINISTRATIVE CONTROLS FOR HIGH RADITION AREAS
(Specification 6.12 and 6.12.2)

Discussion and Evaluation

By letter of October 15, 1979, APCO proposed changes to the Administrative Controls Technical Specification for entry into high radiation areas. Entry into high radiation areas requires positive control of personnel within those areas. Conditions for each entry should be prepared in a manner which is both logical from the standpoint of good radiation protection practice and unambiguous so that each of the alternative methods for control of entry will provide reasonable protection of personnel. The current Standard Technical Specifications (STS) has been written to clearly address the manner in which radiation protection practice may be exercised for positive control for entry into high radiation areas. The APCO submittal of October 15, 1979 falls short of this practice for the following reasons:

(1) Specification 6.12.1.(a) as APCO proposed would provide ffor a "control device" or "alarm signal" required by paragraph 20.203(c)(2) of 10 CFR 20. The control device of paragraph 20.203(c)(2)(i) is not applicable to most radiation sources in nuclear power reactors. Paragraph 20.203 (c)(2)(ii) requires a control device to energize a conspicuous visable or audible alarm signal. The APCO proposal falls short of positive control of access into high radiation areas since the proposed system can either be de-energized by personnel, or, if used by itself with no other control device, could be ignored by personnel. Paragraph 20.203(c)(2)(iii) is an alternative addressed in our STS which has all the connotations of unambiguous positive access control. Therefore, Specification 6.12.1.(a) as proposed is acceptable.

(2) Specification 6.12.1.(b) as proposed by APCO also does not provide positive access control. Pocket ionization chambers are unacceptable as survey meters since they are personnel dosimeters and should be used as such unless no survey meters are available. They are after-the-fact monitoring systems and, therefore cannot be considered positive control devices for determining stay time (i.e., their response is too slow for measurement of dose rate in areas where the dose rate may be rapidly changing). Also audible warning devices (e.g., chirpers) require some skill in interpretation of chirp rate as a function of dose rate and must also operate in a low noise area. The sum of the two instruments (i.e., pocket chamber plus chirper) is therefore not equal to or reliabile as a good radiation survey meter. Consequently, proposed Specification 6.12.1.(b) is unacceptable.

(3) Proposed Specification 6.12.1.(c)(d) is acceptable since it conforms to the STS Section 6.12.1.(a) and (b).

(4) Proposed Specification 6.12.1.(e) is acceptable since it conforms to the STS with the addition of "...by the Health Physics supervisor." The Specification approved for Farley is 6.12.1.(c).

The approved changes will provide a clear definitive condition of positive access control for entry into high radiation areas when the radiation levels are in excess of 1000 mR/hr. This action considers the case where it is not reasonable to provide locked enclosures for small areas having radiation levels in excess of 1000 mR/hr. Such areas may be located in much larger areas such as a pressurized water reactor containment. The conditions for entry into such areas require radiation level measurements in the area and delineation of maximum allowable stay-times in addition to use of barricades, posting and flashing lights as the alternative for locked enclosures. Positive exposure control can also be made by continuous surveillance over the activities within the area by personnel qualified in radiation protection.

Conclusion

The approved, modified Technical Specifications 6.12.1 and 6.12.2 replace the existing specifications and are acceptable. Changes to the APCO proposed revision were discussed with and agreed to by APCO staff.

FEEDWATER CONTROL SYSTEM BYPASS VALVES RESPONSE TIMES
(Specification Table 3.3-5)

Discussion and Evaluation

By letter of October 23, 1979, APCO proposed addition of three feedwater ccontrol system valves (FCV-479, FCV-489 and FCV-499) to Technical Specification Table 3.3-5. These valves were installed as bypass control valves in parallel with the main feedwater control valves (FCV-478, FCV-488 and FCV 498) during the first refueling outage which was completed in late October 1979.

This system modification was accomplished by APCO under 10 CFR 50.59. Bypass control valves provide a means of operating the steam generator level control system at low reactor power levels. The larger size of the main feedwater control valves preclude the use of the main valves at relatively low reactor power levels. Thus, the bypass control valves should result in improved system performance and should result in fewer reactor trips and system transients.

APCO proposed changes to the Technical Specifications to add the bypass control valves to Table 3.3-5 where the main feedwater control valves are shown. This will assure that response time testing is accomplished in a manner consistent with the main feedwater control valves.

Conclusion

Based on the discussion above, we conclude that the added requirement to perform surveillance testing on the bypass control valves is acceptable. FFurther, the testing is similar to testing previously approved on the main feedwater control valves and gives added assurance of valve operability as required by Technical Specification 3.3.2.1, a Limiting Condition for Operation.

CONTINGENCY PLAN LICENSE CONDITION

Discussion

By letter dated March 23, 1979 APCO submitted a Safeguards Security Contingency Plan for the Joseph M. Farley Nuclear Plant as required by

10 CFR 50.34(d) and 10 CFR 73.40. The plan was revised to meet the criteria established by Appendix C to 10 CFR Part 73 and was formulated per Regulatory Guide 5.54 as a self contained plan.

In response to our letter dated August 31, 1979, APCO provided a draft, proposed amendment to the plan by letter of October 8, 1979. Further, in response to our letter dated February 12, 1980, APCO provided in their March 28, 1980 letter, a completely revised text incorporating all previous changes. Our letter dated May 1, 1980 approved the plan as revised. Under provisions of 10 CFR 2.790(d) the plan is being withheld from public disclosure.

Conclusion

Based on our review of the revised Contingency Plan for the Joseph M. Farley Nuclear Plant, we have concluded that the plan for this facility, when fully implemented, will provide the protection needed to meet the general performance requirements of 10 CFR 50.54(p) and 73.40(b) and the objectives of the specific requirements of 10 CFR 73.55(h) and Appendix C to 10 CFR 73. We, therefore, further conclude that your Safeguards Contingency Plan is acceptable.

Changes which would not decrease the effectiveness of your approved Safeguards Contingency Plan may be made without approval by the Commission pursuant to the authority of 10 CFR 50.54(p). A report containing a description of each change shall be furnished to the Director, Office of Nuclear Reactor Regulation, U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, with a copy to the appropriate NRC Regional Office within two months after the change is made. Records of changes made without Commission approval shall be maintained for a period of two years from the date of the change.

Environmental Consideration

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the amendment does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Date: December 10, 1980

7590-01

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NO. 50-348

ALABAMA POWER COMPANY

NOTICE OF ISSUANCE OF AMENDMENT TO FACILITY OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 17 to Facility Operating License No. NPF-2 issued to Alabama Power Company (the licensee), which revised the license and its appended Technical Specifications for operation of the Joseph M. Farley Nuclear Plant, Unit No. 1 (the facility) located in Houston County, Alabama. The Technical Specification change portion of the amendment is effective as of the date of issuance.

The amendment revises administrative controls for entry into high radiation areas and adds feedwater control system bypass valves response times. In addition, the amendment adds a license condition to include the Commission-approved Safeguards Contingency Plan as part of the license.

The applications for the amendment and the licensee's filing comply with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment was not required since this amendment does not involve a significant hazards consideration.