

Mr. G. R. Peterson
 Site Vice President
 Catawba Nuclear Station
 Duke Energy Corporation
 4800 Concord Road
 York, South Carolina 29745-9635

April 9, 1999

SUBJECT: ISSUANCE OF AMENDMENTS - CATAWBA NUCLEAR STATION, UNITS 1 AND 2 (TAC NOS. MA4766 AND MA4767)

Dear Mr. Peterson:

The Nuclear Regulatory Commission has issued the enclosed Amendment No. 178 to Facility Operating License NPF-35 and Amendment No. 170 to Facility Operating License NPF-52 for the Catawba Nuclear Station, Units 1 and 2, in response to your application dated February 18, 1999.

The amendments revise the Technical Specifications Surveillance Requirement (SR) 3.6.16.1 regarding surveillance of reactor building access openings, SR 3.6.16.3 regarding surveillance of reactor building structural integrity, and Administrative Controls 5.5.2 regarding the Containment Leakage Rate Testing Program. The revised requirements would provide scheduling flexibility without decreasing quality and safety margin.

A copy of the related Safety Evaluation is also enclosed. A Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,
 Original signed by:
 Peter S. Tam, Senior Project Manager, Section 1
 Project Directorate II
 Division of Licensing Project Management
 Office of Nuclear Reactor Regulation

Docket Nos. 50-413 and 50-414

- Enclosures: 1. Amendment No. 178 to NPF-35
 2. Amendment No. 170 to NPF-52
 3. Safety Evaluation

cc w/encls: See next page

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UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001
April 9, 1999

Mr. G. R. Peterson
Site Vice President
Catawba Nuclear Station
Duke Energy Corporation
4800 Concord Road
York, South Carolina 29745-9635

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A copy of the related Safety Evaluation is also enclosed. A Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

A handwritten signature in black ink that reads "Peter S. Tam".

Peter S. Tam, Senior Project Manager, Section 1
Project Directorate II
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket Nos. 50-413 and 50-414

Enclosures:

1. Amendment No. 178 to NPF-35
2. Amendment No. 170 to NPF-52
3. Safety Evaluation

cc w/encls: See next page

Catawba Nuclear Station

cc:

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Catawba Nuclear Station

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

DUKE ENERGY CORPORATION

NORTH CAROLINA ELECTRIC MEMBERSHIP CORPORATION

SALUDA RIVER ELECTRIC COOPERATIVE, INC.

DOCKET NO. 50-413

CATAWBA NUCLEAR STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 178
License No. NPF-35

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment to the Catawba Nuclear Station, Unit 1 (the facility) Facility Operating License No. NPF-35 filed by the Duke Energy Corporation, acting for itself, North Carolina Electric Membership Corporation and Saluda River Electric Cooperative, Inc. (licensees), dated February 18, 1999, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations as set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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2. Accordingly, the license is hereby amended by page changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-35 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 178 , which are attached hereto, are hereby incorporated into this license. Duke Energy Corporation shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Richard L. Emch, Jr.

Richard L. Emch, Jr, Chief, Section 1
Project Directorate II
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Attachment:
Technical Specification
Changes

Date of Issuance: April 9, 1999



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

DUKE ENERGY CORPORATION
NORTH CAROLINA MUNICIPAL POWER AGENCY NO. 1
PIEDMONT MUNICIPAL POWER AGENCY
DOCKET NO. 50-414
CATAWBA NUCLEAR STATION, UNIT 2
AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 170
License No. NPF-52

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment to the Catawba Nuclear Station, Unit 2 (the facility) Facility Operating License No. NPF-52 filed by the Duke Energy Corporation, acting for itself, North Carolina Municipal Power Agency No. 1 and Piedmont Municipal Power Agency (licensees), dated February 18, 1999, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations as set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

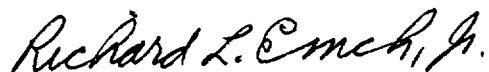
2. Accordingly, the license is hereby amended by page changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-52 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 170 , which are attached hereto, are hereby incorporated into this license. Duke Energy Corporation shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Richard L. Emch, Jr., Chief, Section 1
Project Directorate II
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Attachment:
Technical Specification
Changes

Date of Issuance: April 9, 1999

ATTACHMENT TO LICENSE AMENDMENT NO. 178
TO FACILITY OPERATING LICENSE NO. NPF-35
AND LICENSE AMENDMENT NO. 170
TO FACILITY OPERATING LICENSE NO. NPF-52
DOCKET NOS. 50-413 AND 50-414

Replace the following pages of the joint Technical Specifications (Appendix A of the Operating Licenses) with the enclosed pages. The revised pages are identified by amendment number and contain vertical lines indicating the areas of change.

Remove

3.6.16-1
3.6.16-2
5.5-1

Insert

3.6.16.1
3.6.16-2
5.5-1

Replace the following pages of the Technical Specifications Bases with the enclosed pages. The revised pages are identified by amendment number and contain vertical lines indicating the areas of change.

Remove

B 3.6.16-1
B 3.6.16-2
B 3.6.16-3

Insert

B 3.6.16-1
B 3.6.16-2
B 3.6.16-3

3.6 CONTAINMENT SYSTEMS

3.6.16 Reactor Building

LCO 3.6.16 The reactor building shall be OPERABLE.

APPLICABILITY: MODES 1, 2, 3, and 4.

ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. Reactor building inoperable.	A.1 Restore reactor building to OPERABLE status.	24 hours
B. Required Action and associated Completion Time not met.	B.1 Be in MODE 3.	6 hours
	<u>AND</u> B.2 Be in MODE 5.	36 hours

SURVEILLANCE REQUIREMENTS

SURVEILLANCE	FREQUENCY
SR 3.6.16.1 Verify the door in each access opening is closed, except when the access opening is being used for normal transit entry and exit.	31 days

(continued)

SURVEILLANCE REQUIREMENTS (continued)

SURVEILLANCE	FREQUENCY
SR 3.6.16.2 Verify each Annulus Ventilation System train produces a pressure equal to or more negative than -0.5 inch water gauge in the annulus within 1 minute after a start signal.	18 months on a STAGGERED TEST BASIS
SR 3.6.16.3 Verify reactor building structural integrity by performing a visual inspection of the exposed interior and exterior surfaces of the reactor building.	3 times every 10 years, coinciding with containment visual examinations required by SR 3.6.1.1

5.0 ADMINISTRATIVE CONTROLS

5.5 Programs and Manuals

The following programs shall be established, implemented, and maintained.

5.5.1 Offsite Dose Calculation Manual (ODCM)

The ODCM shall contain the methodology and parameters used in the calculation of offsite doses resulting from radioactive gaseous and liquid effluents, in the calculation of gaseous and liquid effluent monitoring alarm and trip setpoints, and in the conduct of the radiological environmental monitoring program.

Licensee initiated changes to the ODCM:

- a. Shall be documented and records of reviews performed shall be retained. This documentation shall contain:
 1. sufficient information to support the change(s) together with the appropriate analyses or evaluations justifying the change(s), and
 2. a determination that the change(s) do not adversely impact the accuracy or reliability of effluent, dose, or setpoint calculations;
- b. Shall become effective after the approval of the Station Manager; and
- c. Shall be submitted to the NRC in the form of a complete, legible copy of the entire ODCM as a part of or concurrent with the Radioactive Effluent Release Report for the period of the report in which any change in the ODCM was made. Each change shall be identified by markings in the margin of the affected pages, clearly indicating the area of the page that was changed, and shall indicate the date (i.e., month and year) the change was implemented.

5.5.2 Containment Leakage Rate Testing Program

A program shall be established to implement the leakage rate testing of the containment as required by 10 CFR 50.54(o) and 10 CFR 50, Appendix J, Option B, for Type A testing, as modified by approved exemptions. This program shall be in accordance with the guidelines contained in Regulatory Guide 1.163, "Performance-Based Containment Leak-Test Program," dated September 1995, except that the containment visual examinations required by Regulatory Position C.3 shall be conducted 3 times every 10 years, including during each shutdown for SR 3.6.1.1 Type A test, prior to initiating the Type A test.

(continued)

B 3.6 CONTAINMENT SYSTEMS

B 3.6.16 Reactor Building

BASES

BACKGROUND The reactor building is a concrete structure that surrounds the steel containment vessel. Between the containment vessel and the reactor building inner wall is an annular space that collects containment leakage that may occur following a loss of coolant accident (LOCA). This space also allows for periodic inspection of the outer surface of the steel containment vessel.

The Annulus Ventilation System (AVS) establishes a negative pressure in the annulus between the reactor building and the steel containment vessel under post-accident conditions. Filters in the system then control the release of radioactive contaminants to the environment. The reactor building is required to be OPERABLE to ensure retention of containment leakage and proper operation of the AVS. To ensure the retention of containment leakage within the reactor building:

- a. The door in each access opening is closed except when the access opening is being used for normal transit entry and exit, and
- b. The sealing mechanism associated with each penetration (e.g., welds, bellows, or O-rings) is OPERABLE.

APPLICABLE SAFETY ANALYSES The design basis for reactor building OPERABILITY is a LOCA. Maintaining reactor building OPERABILITY ensures that the release of radioactive material from the containment atmosphere is restricted to those leakage paths and associated leakage rates assumed in the accident analyses.

The reactor building satisfies Criterion 3 of 10 CFR 50.36 (Ref. 1).

LCO Reactor building OPERABILITY must be maintained to ensure proper operation of the AVS and to limit radioactive leakage from the containment to those paths and leakage rates assumed in the accident analyses.

BASES

APPLICABILITY

Maintaining reactor building OPERABILITY prevents leakage of radioactive material from the reactor building. Radioactive material may enter the reactor building from the containment following a LOCA. Therefore, reactor building OPERABILITY is required in MODES 1, 2, 3, and 4 when a steam line break, LOCA, or rod ejection accident could release radioactive material to the containment atmosphere.

In MODES 5 and 6, the probability and consequences of these events are low due to the Reactor Coolant System temperature and pressure limitations in these MODES. Therefore, reactor building OPERABILITY is not required in MODE 5 or 6.

ACTIONS

A.1

In the event reactor building OPERABILITY is not maintained, reactor building OPERABILITY must be restored within 24 hours. Twenty-four hours is a reasonable Completion Time considering the limited leakage design of containment and the low probability of a Design Basis Accident occurring during this time period.

B.1 and B.2

If the reactor building cannot be restored to OPERABLE status within the required Completion Time, the plant must be brought to a MODE in which the LCO does not apply. To achieve this status, the plant must be brought to at least MODE 3 within 6 hours and to MODE 5 within 36 hours. The allowed Completion Times are reasonable, based on operating experience, to reach the required plant conditions from full power conditions in an orderly manner and without challenging plant systems.

**SURVEILLANCE
REQUIREMENTS**

SR 3.6.16.1

Maintaining reactor building OPERABILITY requires maintaining the door in each access opening closed, except when the access opening is being used for normal transit entry and exit. The 31 day Frequency of this SR is based on engineering judgment and is considered adequate in view of the other indications of door status that are available.

BASES

SURVEILLANCE REQUIREMENTS (continued)

SR 3.6.16.2

The ability of a AVS train to produce the required negative pressure ≥ 0.5 inch water gauge during the test operation within 1 minute provides assurance that the building is adequately sealed. The negative pressure prevents leakage from the building, since outside air will be drawn in by the low pressure. The negative pressure must be established within the time limit to ensure that no significant quantity of radioactive material leaks from the reactor building prior to developing the negative pressure.

The AVS trains are tested every 18 months on a STAGGERED TEST BASIS to ensure that in addition to the requirements of LCO 3.6.10, "Annulus Ventilation System," either AVS train will perform this test. The 18 month Frequency is based on the need to perform this Surveillance under the conditions that apply during a plant outage.

SR 3.6.16.3

This SR would give advance indication of gross deterioration of the concrete structural integrity of the reactor building. The Frequency is based on engineering judgment, and is the same as that for containment visual inspections performed in accordance with SR 3.6.1.1.

REFERENCES

1. 10 CFR 50.36, Technical Specifications, (c)(2)(ii).



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 178 TO FACILITY OPERATING LICENSE NPF-35
AND AMENDMENT NO. 170 TO FACILITY OPERATING LICENSE NPF-52

DUKE ENERGY CORPORATION, ET AL.

CATAWBA NUCLEAR STATION, UNITS 1 AND 2

DOCKET NOS. 50-413 AND 50-414

1.0 INTRODUCTION

By letter dated February 18, 1999, Duke Energy Corporation, et al. (DEC, the licensee), submitted a request for changes to the Catawba Nuclear Station, Units 1 and 2 Technical Specifications (TSs). The requested changes would revise Surveillance Requirement (SR) 3.6.16.1, SR 3.6.16.3, and Administrative Controls 5.5.2. The licensee found that the revised requirements would provide scheduling flexibility. The staff's review of DEC's proposed revision is set forth below.

2.0 DISCUSSION AND EVALUATION

2.1 Surveillance Requirements and Administrative Controls

2.1.1 Surveillance Requirement 3.6.16.1

This SR currently requires, on a 31-day frequency:

"Verify each door in each access opening is closed, except when the access opening is being used for normal transit entry and exit; then at least one door shall be closed."

The wording pertains to airlock design (two doors) for entry into the reactor building. The licensee stated that there are five openings into each Catawba reactor building; each of these openings has only a single door.

The licensee proposed to revise this SR to state:

"Verify the door in each access opening is closed, except when the access opening is being used for normal transit entry and exit."

The staff reviewed the licensee's submitted information and agreed that errors exist in the current TS as described above. The current requirement is inconsistent with the plant as originally designed and built. The licensee's proposed change would correct the error and is, therefore, acceptable.

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2.1.2 Surveillance Requirement 3.6.16.3

This SR requires that reactor building structural integrity be verified by performing a visual inspection of the exposed interior and exterior surfaces of the reactor building once every 40 months and during shutdown for SR 3.6.1.1 (i.e., Type A tests). SR 3.0.2 allows SR intervals be extended to 1.25 the specified value. Thus, SR 3.6.16.3 would have to be done within 40 to 50 months of the last surveillance.

The licensee stated that the current interval for Unit 1 would expire soon, leading to the possibility of an unnecessary shutdown in order to perform SR 3.6.16.3. The licensee observed that the Catawba TS before conversion to the Improved Technical Specification (ITS) format did not have the 40-to-50-month interval specified. Instead, the old TS (Section 4.5.1.7) specified that a structural integrity inspection be performed at the same time as the Type A test (i.e., every 10 years), and two additional structural integrity inspections be performed during shutdowns between Type A tests at approximately equal intervals.

The licensee observed that under the current wording, and if the Type A test did not happen to fall within the 40-to-50-month interval, a total of four structural integrity inspections could be required during a 10-year interval. The licensee also observed that there are now three separate requirements pertaining to visual examination of the steel containment vessel and the reactor building: Appendix J of 10 CFR Part 50, Option B; Section XI of the ASME Code; and SR 3.6.16.3. In order to meet all these requirements, the licensee would like to see more flexibility in SR 3.6.16.3. In particular, the licensee plans to schedule structural integrity inspections concurrently with examinations required by other requirements.

Accordingly, the licensee proposed to revise the frequency requirement of SR 3.6.16.3 to "3 times every 10 years, coinciding with containment visual examinations required by SR 3.6.1.1." This would essentially revise the frequency requirement back to what it was before implementation of the ITS.

The staff agrees that the revised wording will provide more flexibility in scheduling without decreasing the number of structural integrity inspections (three per 10 years). The proposed revision would have no impact on the quality or the safety functions of the affected components. Therefore, the proposed revision of SR 3.6.16.3 is acceptable.

2.1.3 Administrative Control 5.5.2

This requires a program for leakage rate testing of the containment as required by 10 CFR 50.54(o) and 10 CFR Part 50, Appendix J, Option B, for Type A testing. This also requires that the guidelines contained in Regulatory Guide (RG) 1.163, "Performance-Based Containment Leak-Test Program," be followed.

The licensee proposed to add the following to Administrative Control 5.5.2:

“Except that the containment visual examinations required by Regulatory Position C.3 shall be conducted 3 times every 10 years, including during each shutdown for SR 3.6.1.1 Type A test, prior to initiating the Type A test.”

The licensee pointed out that Regulatory Position C.3 of RG 1.163 states that these containment visual examinations should be conducted during two other refueling outages. The licensee stated that performance of these examinations during operation or shutdown has no impact on the quality of these examinations, provided all accessible interior and exterior surfaces are examined. The additional wording would provide the licensee with scheduling flexibility.

The staff agrees that the additional wording will provide more flexibility in scheduling without decreasing the number of containment visual examinations (three per 10 years). The purpose of Regulatory Position C.3 of RG 1.163 was to specify that visual examinations were to be conducted at a certain frequency. The phrase “during two other refueling outages” was used because the staff assumed the examinations would probably be done during refueling outages. However, this was by no means intended to be an essential condition for the examination. Thus the revision would have no impact on the quality or safety functions of the affected components. Therefore, the proposed revision of Administrative Control 5.5.2 is acceptable.

2.2 Technical Specification Bases Document

The TS Bases is a licensee-controlled document, and is not part of the TS (10 CFR 50.36(a)). However, the staff reviewed the licensee’s proposed changes as supplemental information for the changes to the TS. The staff finds the proposed changes to the Bases document acceptable as summarized below.

2.2.1 Bases for SR 3.6.16.1

The licensee proposed to revise the bases for SR 3.6.16.1 to reflect the correction described above. The staff evaluated the revision of SR 3.6.16.1 in Section 2.1.1 above. In addition, the licensee pointed out that there is no control room indication of reactor building door status; instead, indication is by alarm station methods which are available to Security. The licensee thus proposed to delete the phrase “to the operator” which implies, incorrectly, that there is indication to the operator in the control room.

2.2.2 Bases for SR 3.6.16.3

The licensee proposed to revise the bases for SR 3.6.16.3 to reflect the revised frequency requirement. The staff evaluated the revision of SR 3.6.16.3 in Section 2.1.2 above.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the South Carolina State official, Mr. Virgil Autrey, was notified of the proposed issuance of the amendments. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

The amendments change surveillance requirements. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration, and there has been no public comment on such finding (64 FR 11961 dated March 10, 1999). Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributors: James C. Pulsipher
Peter S. Tam

Date: April 9, 1999