

AUGUST 22, 1997

Mr. G. R. Peterson
Site Vice President
Catawba Nuclear Station
Duke Power Company
4800 Concord Road
York, South Carolina 29745-9635

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Docket File ACRS T-2 E26
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SUBJECT: ISSUANCE OF AMENDMENTS - CATAWBA NUCLEAR STATION, UNITS 1 AND 2
(TAC NOS. M98958 AND M98959)

Dear Mr. Peterson:

The Nuclear Regulatory Commission has issued the enclosed Amendment No.161 to Facility Operating License NPF-35 and Amendment No. 153 to Facility Operating License NPF-52 for the Catawba Nuclear Station, Units 1 and 2. The amendments are in response to your application dated June 12, 1997.

The amendments change the name "Duke Power Company" to "Duke Energy Corporation" in the Catawba operating licenses and appendices as a result of Duke Power Company's recent name change.

A copy of the related Safety Evaluation is also enclosed. A Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

ORIGINAL SIGNED BY:

Peter S. Tam, Senior Project Manager
Project Directorate II-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket Nos. 50-413 and 50-414

Enclosures: 1. Amendment No. 161 to NPF-35
2. Amendment No. 153 to NPF-52
3. Safety Evaluation

cc w/encls: See next page

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DOCUMENT NAME: G:\CATAWBA\CAT98958.AMD *See previous concurrence

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| OFFICE | PDII-2/PM | PDII-2/LA | OGC* | PDII-2/D | |
| NAME | P.TAM:cn | L.BERRY | SHOM | H.BERKOW | |
| DATE | 7/25/97 | 8/1/97 | 7/21/97 | 7/12/97 | |
| COPY | YES NO | YES NO | YES NO | YES NO | |

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UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

AUGUST 22, 1997

Mr. G. R. Peterson
Site Vice President
Catawba Nuclear Station
Duke Power Company
4800 Concord Road
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Sincerely,

A handwritten signature in black ink, reading "Peter S. Tam", is written over a large, stylized flourish.

Peter S. Tam, Senior Project Manager
Project Directorate II-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket Nos. 50-413 and 50-414

Enclosures: 1. Amendment No. 161 to NPF-35
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cc w/encls: See next page

Catawba Nuclear Station

cc:

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Catawba Nuclear Station

cc:

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

DUKE POWER COMPANY

NORTH CAROLINA ELECTRIC MEMBERSHIP CORPORATION

SALUDA RIVER ELECTRIC COOPERATIVE, INC.

DOCKET NO. 50-413

CATAWBA NUCLEAR STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 161
License No. NPF-35

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment to the Catawba Nuclear Station, Unit 1 (the facility) Facility Operating License No. NPF-35 filed by the Duke Power Company, acting for itself, North Carolina Electric Membership Corporation and Saluda River Electric Cooperative, Inc. (licensees), dated June 12, 1997, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations as set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is hereby amended by page changes to the license itself, Attachment 1, and Appendix D as indicated in the attachment to this license amendment.
3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Herbert N. Berkow, Director
Project Directorate II-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachments: 1. Facility Operating License No. NPF-35
2. Attachment 1
3. Appendix D

Date of Issuance: August 22, 1997

ATTACHMENT TO LICENSE AMENDMENT NO. 161

FACILITY OPERATING LICENSE NO. NPF-35

DOCKET NO. 50-413

The revised pages are identified by Amendment number and contain vertical lines indicating the areas of change.

Replace Pages 1 thru 11 of Facility Operating License No. NPF-35 with the attached new pages.

Replace page 1 of Attachment 1 of the license.

Replace Appendix D in its entirety with the attached revised Appendix D.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

DUKE ENERGY CORPORATION

NORTH CAROLINA ELECTRIC MEMBERSHIP CORPORATION

SALUDA RIVER ELECTRIC COOPERATIVE, INC.

DOCKET NO. 50-413

CATAWBA NUCLEAR STATION, UNIT 1

FACILITY OPERATING LICENSE

License No. NPF-35

1. The Nuclear Regulatory Commission (the Commission or the NRC) has found that:
 - A. The application for license filed by the Duke Energy Corporation acting for itself and North Carolina Electric Membership Corporation and Saluda River Electric Cooperative, Inc. (the licensees) complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I; and all required notifications to other agencies or bodies have been duly made;
 - B. Construction of the Catawba Nuclear Station, Unit 1 (the facility) has been substantially completed in conformity with Construction Permit No. CPPR-116 and the application, as amended, the provisions of the Act and the regulations of the Commission;
 - C. The facility will operate in conformity with the application, as amended, the provisions of the Act, and the regulations of the Commission (except as exempted from compliance in Section 2.D. below);
 - D. There is reasonable assurance: (i) that the activities authorized by this operating license can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I (except as exempted from compliance in Section 2.D. below);
 - E. Duke Energy Corporation* is technically qualified to engage in the

*Duke Energy Corporation is authorized to act as agent for the North Carolina Electric Membership Corporation and the Saluda River Electric Cooperative, Inc., and has exclusive responsibility and control over the physical construction, operation and maintenance of the facility.

activities authorized by this license in accordance with the Commission's regulations set forth in 10 CFR Chapter I;

- F. The licensees have satisfied the applicable provisions of 10 CFR Part 140, "Financial Protection Requirements and Indemnity Agreements," of the Commission's regulations;
 - G. The issuance of this license will not be inimical to the common defense and security or to the health and safety of the public;
 - H. After weighing the environmental, economic, technical, and other benefits of the facility against environmental and other costs and considering available alternatives, the issuance of this Facility Operating License No. NPF-35, subject to the conditions for protection of the environment set forth in the Environmental Protection Plan attached as Appendix B, is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied;
 - I. The receipt, possession, and use of source, byproduct and special nuclear material as authorized by this license will be in accordance with the Commission's regulations in 10 CFR Parts 30, 40 and 70.
2. Based on the foregoing findings and the Partial Initial Decisions issued by the Atomic Safety and Licensing Boards dated June 22, September 18, and November 27, 1984, regarding this facility and satisfaction of conditions therein imposed, except as hereinafter set forth, and the Commission's vote on January 17, 1985, Facility Operating License No. NPF-31 issued on December 6, 1984, is superseded by Facility Operating License No. NPF-35 hereby issued to the Duke Energy Corporation, the North Carolina Electric Membership Corporation, and the Saluda River Electric Cooperative, Inc., (the licensees) to read as follows:
- A. This license applies to the Catawba Nuclear Station, Unit 1, a pressurized water reactor and associated equipment (the facility) owned by the Duke Energy Corporation, the North Carolina Electric Membership Corporation, and the Saluda River Electric Cooperative, Inc. The facility is located on the licensees' site in York County, South Carolina, on the shore of Lake Wylie approximately 6 miles north of Rock Hill, South Carolina, and is described in Duke Energy Corporation's Final Safety Analysis Report, as supplemented and amended through Revision No. 11, and in its Environmental Report, as supplemented and amended through Revision No. 6;
 - B. Subject to the conditions and requirements incorporated herein, the Commission hereby licenses:

- (1) Duke Energy Corporation, pursuant to Section 103 of the Act and 10 CFR Part 50, to possess, use, and operate the facility at the designated location in York County, South Carolina, in accordance with the procedures and limitations set forth in the license;
- (2) North Carolina Electric Membership Corporation and Saluda River Electric Cooperative, Inc., to possess the facility at the designated location in York County, South Carolina, in accordance with the procedures and limitations set forth in this license;
- (3) Duke Energy Corporation, pursuant to the Act and 10 CFR Part 70 to receive, possess and use at any time special nuclear material as reactor fuel, in accordance with the limitations for storage and amounts required for reactor operation, as described in the Final Safety Analysis Report, as supplemented and amended through Revision No. 11;
- (4) Duke Energy Corporation, pursuant to the Act and 10 CFR Parts 30, 40 and 70 to receive, possess and use at any time any byproduct, source and special nuclear material as sealed neutron sources for reactor startup, sealed sources for reactor instrumentation and radiation monitoring equipment calibration, and as fission detectors in amounts as required;
- (5) Duke Energy Corporation, pursuant to the Act and 10 CFR Parts 30, 40 and 70, to receive, possess and use in amounts as required any byproduct, source or special nuclear material without restriction to chemical or physical form, for sample analysis or instrument calibration or associated with radioactive apparatus or components; and
- (6) Duke Energy Corporation, pursuant to the Act and 10 CFR Parts 30, 40 and 70, to possess, but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility authorized herein.
- (7) Duke Energy Corporation, pursuant to the Act and 10 CFR Parts 30, 40 and 70, to possess, but not separate, such byproduct and special nuclear materials as may be produced by the operation of McGuire Nuclear Station, Units 1 and 2, and Oconee Nuclear Station, Units 1, 2 and 3.

C. This license shall be deemed to contain and is subject to the conditions specified in the Commission's regulations set forth in 10 CFR Chapter I and is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:

(1) Maximum Power Level

Duke Energy Corporation is authorized to operate the facility at reactor core power levels not in excess of 3411 megawatts thermal (100% power) in accordance with the conditions specified herein and in Attachment 1 to this license. The preoperational tests, startup tests and other items identified in Attachment 1 to this license shall be completed as specified. Attachment 1 is hereby incorporated into this license.

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 161, which are attached hereto, are hereby incorporated into this license. Duke Energy Corporation shall operate the facility in accordance with the Technical Specifications.

(3) Initial Startup Test Program (Section 14, SER, SSER #3)*

Duke Energy Corporation shall conduct those aspects of the post-fuel-loading initial test program described in Chapter 14 of the FSAR, as amended, which are consistent with the limits of this license without making any major modifications unless such modifications have prior NRC approval. Major modifications are defined as:

- (a) elimination of any safety-related test**;
- (b) modification of objectives, test method, or acceptance criteria for any safety-related test;
- (c) performance of any safety-related test at a power level different from that stated in the FSAR by more than 5 percent of rated power;
- (d) failure to satisfactorily complete the entire initial startup test program by the time core burnup equals 120 effective full power days;
- (e) deviation from initial test program administrative procedures or quality assurance controls described in the FSAR; and

*The parenthetical notation following the title of many license conditions denotes the section of the Safety Evaluation Report and/or its supplement; wherein the license condition is discussed.

**Safety-related tests are those tests which verify the design, construction and operation of safety-related systems, structures, and equipment.

- (f) delays in test program in excess of 30 days (14 days if power level exceeds 50 percent), concurrent with power operation. If continued power operation is desired during a delay, Duke Energy Corporation shall provide justification that adequate testing has been performed and evaluated to demonstrate that the facility can be operated at the planned power level with reasonable assurance that the health and safety of the public will not be endangered.

(4) Antitrust Conditions

Duke Energy Corporation shall comply with the antitrust conditions delineated in Appendix C to this license.

(5) Inservice Testing of Pumps and Valves (Section 3.9.6, SSER #2, SSER #4)

Pursuant to 10 CFR Part 50.55a and for the reasons set forth in Section 3.9.6 of SSER #2, the relief identified in the submittals dated March 9, 1983, July 10, 13, 18, 23, 27, October 1, and November 6, 1984, that Duke Energy Corporation has requested from the pump and valve testing requirements of 10 CFR Part 50, Section 50.55a(g)(3) and (g)(4)(i) is granted for that portion of the initial 120-month period until the staff completes its review or until December 1, 1986, whichever is earlier.

(6) Inservice Inspection Program (Section 5.2.4 and 6.6, SSER #2*)

By May 31, 1985, Duke Energy Corporation shall submit the balance of the inservice inspection program as described in its letter dated January 8, 1985, for staff review and approval.

(7) Environmental Equipment Qualification (Section 3.11, SER, SSER #3, SSER #4)

Prior to March 31, 1985, Duke Energy Corporation shall environmentally qualify all electrical equipment as required by 10 CFR 50.49.

(8) Fire Protection Program (Section 9.5.1, SER, SSER #2, SSER #3, SSER #4, SSER #5)

Duke Energy Corporation shall implement and maintain in effect all provisions of the approved fire protection program as described in the Final Safety Analysis Report, as amended, for the facility and as approved in the SER through Supplement 6, subject to the following provision:

*Safety evaluation attached to D. Eisenhut letter dated January 17, 1985. To be incorporated in SSER #5.

The licensee may make changes to the approved fire protection program without prior approval of the Commission only if those changes would not adversely affect the ability to achieve and maintain safe shutdown in the event of a fire.

(9) Turbine Missiles (Section 3.5.1.3, SER)

Duke Energy Corporation shall submit for NRC staff approval by December 6, 1987, a turbine system maintenance program based on the manufacturer's calculations of missile generation probabilities acceptable to the NRC staff or volumetrically inspect all low pressure turbine rotors within three years or by the second refueling outage, whichever is later, and thereafter every three years or every other refueling outage until a maintenance program is approved by the staff.

(10) Operating Staff Experience Requirements (Section 13.1.2.3, SSER #3, SSER #4)

Duke Energy Corporation (DEC) shall have a licensed senior operator on each shift who has had at least six months of hot operating experience on a similar type plant, including at least six weeks at power levels greater than 20% of full power, and who has had start-up and shutdown experience. For those shifts where such an individual is not available on the plant staff, an advisor shall be provided who has had at least four years of power plant experience, including two years of nuclear plant experience, and who has had at least one year of experience on shift as a licensed senior operator at a similar type facility. Use of advisors who were licensed only at the RO level will be evaluated on a case-by-case basis. Advisors shall be trained on plant procedures, technical specifications and plant systems, and shall be examined on these topics at a level sufficient to assure familiarity with the plant. For each shift, the remainder of the shift crew shall be trained in the role of the advisors. These advisors shall be retained until the experience levels identified in the first sentence above have been achieved. The NRC shall be notified at least 30 days prior to the date DEC proposes to release the advisors from further service.

- (11) Detailed Control Room Design Review, I.D.1 (Section 18.0, SER, SSER #2)

Duke Energy Corporation shall correct all human engineering deficiencies according to the schedule contained in the letter from Duke Energy Corporation dated February 20, 1984.

- (12) Emergency Response Capabilities (Generic Letter 82-33, Supplement 1 to NUREG-0737).

(a) Deleted

(b) Safety Parameter Display System (SPDS)

Prior to April 1, 1985, Duke Energy Corporation shall have the SPDS operational.

- (13) Anticipatory Reactor Trip, II.K.3.10 (Section 5.2.2, SER)

Prior to exceeding 70% power, Duke Energy Corporation shall complete the described turbine trip tests to verify that PORVs will not be challenged when the anticipatory trip bypass is in effect.

- (14) Hydrogen Control Measure, II.B.7 (Section 6.2.5, Appendix C, SER; Section 6.2.5, SSER #2, SSER #3, SSER #4)

Prior to April 1, 1985, upgraded analyses and tests shall be provided on the following issues and submitted for staff review and approval;

(a) thermal response of the containment atmosphere and essential equipment for a spectrum of accident sequences using revised heat transfer models.

- (b) effects of upper compartment burns on the operation and survival of air return fans and ice condenser doors.
- (c) operability of the glow plug igniter in a spray environment typical of that expected in the upper compartment of the containment.

(15) Instrumentation for Detection of Inadequate Core Cooling, II.F.2
(Section 4.4.3.4, SER, SSER #2)

Prior to startup following the first refueling outage, Duke Energy Corporation shall complete the upgrade of the existing subcooling margin monitor and the existing backup display.

(16) Steam Generator Tube Rupture (Section 15.4.4, SER, SSER #2)

Prior to startup following the second refueling outage, Duke Energy Corporation shall submit for NRC staff review and approval an analysis which demonstrates that the steam generator single-tube rupture analysis presented in the FSAR is the most severe case with respect to the release of fission products and calculated doses. Consistent with the analytical assumptions, Duke Energy Corporation shall propose any necessary changes to Appendix A to this license.

(17) Main Steam Line Break (MSLB) Inside Containment (Section 6.2.1.1, SER, SSER #2, SSER #4)

- (a) Prior to startup following the first refueling outage, Duke Energy Corporation shall submit for NRC review and approval the results of a completed program of tests and analyses to confirm the validity and accuracy of the models and assumptions employed in the revised containment response analysis for MSLB accidents. This program shall include, but not be limited to, the following elements:
 - (i) Hydraulic tests to quantify key parameters related to drain flow model input and assumptions for drain region configurations representative of those in the plant, and uncertainty analysis of measured and computed parameters.
 - (ii) Supplementary tests or analyses to address (i) the applicability of drain flow tests conducted in air to steam environments and (ii) the effect of thermal gradients within the drain flow liquid sheets and droplets.
 - (iii) Revised containment response analyses which incorporate the results of the drain flow test program and address the thermal response to dead-ended compartments.

- (iv) Additional containment response sensitivity analyses to investigate the effect of drain water temperature and flow rate, and uncertainties in parameters determined by test.
- (v) Scaled tests or detailed mass transport analyses for a spectrum of break locations to quantify the impact of break conditions on thermal gradients and non-condensable gas distribution in containment and on ice condenser performance.
- (b) During the interim period of operation, Duke Energy Corporation shall submit to the NRC staff bimonthly reports on the progress of the above confirmatory research program of tests and analyses regarding containment response for MSLB accidents.
- (18) Residual Heat Removal System (Section 5.4.4, SER, SSER #2; Section 15.4.4, SSER #3, SSER #4)*

Prior to startup following the first refueling outage, Duke Energy Corporation shall upgrade the pressurizer power operated relief valves (PORVs) and the steam generator PORVs to safety related.

- (19) Seismic Equipment Qualification (Section 3.10, SSER #2, SSER #3, SSER #4)

A seismic test will be performed utilizing a generic mounting scheme with a GLASTIC pad and fiberglass bushing for electrical isolation to verify the acceptability of the existing mounting. This test will be completed by July 1985.

2.C. (20) Deleted

- (21) Generic Letter 83-28 (Section 15.6, SSER #4,**)

Duke Energy Corporation shall submit responses to and implement the requirements of Generic Letter 83-28 on a schedule which is consistent with that given in its November 2, and December 31, 1984, letters.

* Requires exemption; see paragraph 2.D

**Safety evaluation attached to D. Eisenhut letter dated January 17, 1985. To be incorporated in SSER #5.

(22) Progress of Offsite Emergency Preparedness (Section 13.3, SER, SSER #1, SSER #2, SSER #3, SSER #4)

In the event that the NRC finds that the lack of progress in completion of the procedures in the Federal Emergency Management Agency's final rule, 44 CFR Part 350, is an indication that a major substantive problem exists in achieving or maintaining an adequate state of preparedness, the provisions of 10 CFR Section 50.54(s)(2) will apply.

(23) Emergency Preparedness Issues (ASLB PID, 9/18/84)

By June 4, 1985, Duke Energy Corporation shall have submitted for staff review and received staff approval on the following items:

1. The Public Information Brochure shall state that high levels of radiation are harmful to health and may be life threatening. Such statements shall be contained within that portion of the brochure that deals with actions to be taken in the event of an emergency.
2. The warning signs and decals shall specify the types of emergencies they cover including nuclear.
3. The warning signs and decals shall notify transients as to where they can obtain local emergency information, as provided in NUREG-0654 Evaluation Criterion II.G.2.
4. The emergency plans shall reflect the kinds of locations within the plume exposure EPZ wherein the warning signs and decals and emergency response information will be placed and the procedures employed to assure that sufficient numbers are being distributed to effectively reach transients, and that the plans are implemented.
5. Comprehensive plans shall provide for early notification to Carowinds of a radiological emergency at Catawba and for evacuation of Carowinds. The plans shall describe the responsibilities of the emergency response organizations of Mecklenburg and York Counties and provide for the coordination of their efforts among themselves and with Carowinds' officials. The plans shall provide for immediate notification of patrons and staff of Carowinds at the time of the precautionary closing of the park, of the cause of the emergency. The means to implement the plans shall be made available.

- D. The facility requires exemptions from certain requirements of Appendices A, E and J to 10 Part CFR 50. These include (a) partial exemption from General Design Criterion 1 of Appendix A, with respect to the upgrade to safety-related of the pressurizer

power operated relief valves (PORVs) and steam generator PORVs until first refueling (Section 5.4.4 of SER and SSER 2, and Section 15.4.4 of SSERs 3 and 4), (b) exemption from the requirements of Appendix E, IV.F, insofar as they may require the active participation of all Crisis Management Center personnel for the Catawba Station emergency preparedness exercises (Section 13.3 of SSER 4), (c) partial exemption from the requirement of paragraph III.D.2(b)(ii) of Appendix J, the testing of containment airlocks at times when the containment integrity is not required (Section 6.2.6 of the SER, and SSERs 3 and 4), (d) exemption from the requirement of paragraph III.A.(d) of Appendix J, insofar as it requires the venting and draining of lines for type A tests (Section 6.2.6 of SSER 3), and (e) partial exemption from the requirements of paragraph III.B of Appendix J, as it relates to bellows testing (Section 6.2.6 of the SER and SSER 3). These exemptions are authorized by law and will not endanger life or property or the common defense and security and are otherwise in the public interest. These exemptions are, therefore, hereby granted pursuant to 10 CFR 50.12. With the granting of these exemptions, the facility will operate, to the extent authorized herein, in conformity with the application, as amended, the provisions of the Act, and the rules and regulations of the Commission.

- E. Duke Energy Corporation shall fully implement and maintain in effect all provisions of the Commission-approved physical security, guard training and qualification, and safeguards contingency plans including amendments made pursuant to provisions of the Miscellaneous Amendments and Search Requirements revisions to 10 CFR 73.55 (51 FR 27817 and 27822) and to the authority of 10 CFR 50.90 and CFR 10 CFR 50.54(p). The plans, which contain safeguards information protected under 10 CFR 73.21, are entitled: "Catawba Nuclear Station Physical Security Plan," with revisions submitted through October 6, 1987; "Catawba Nuclear Station Training and Qualification Plan," with revisions submitted through August 27, 1986; and "Catawba Nuclear Station Safeguards Contingency Plan," with revisions submitted through January 8, 1987. Changes made in accordance with 10 CFR 73.55 shall be implemented in accordance with the schedule set forth therein.

- F. Reporting to the Commission

Duke Energy Corporation shall report any violations of the requirements contained in Section 2, Items C.(1), C.(3) through C.(23) of this license. Initial notification shall be made within

twenty-four (24) hours in accordance with the provisions of 10 CFR 50.72 with written follow-up within 30 days in accordance with the procedures described in 10 CFR 50.73 (b), (c) and (e).

- G. The licensees shall have and maintain financial protection of such type and in such amounts as the Commission shall require in accordance with Section 170 of the Atomic Energy Act of 1954, as amended, to cover public liability claims.
- H. This license is effective as of the date of issuance and shall expire at midnight on December 6, 2024.

FOR THE NUCLEAR REGULATORY COMMISSION

Original Signed By:
Edson G. Case /f/

Harold R. Denton, Director
Office of Nuclear Reactor Regulation

Enclosures:

- 1. Attachment 1
- 2. Appendix A - Technical Specifications
- 3. Appendix B - Environmental Protection Plan
- 4. Appendix C - Antitrust Conditions
- 5. Appendix D - Additional Conditions

Date of Issuance: January 17, 1985

January 17, 1985

ATTACHMENT 1 TO LICENSE NPF-35

Duke Energy Corporation shall comply with the following requirements related to the TDI diesel engines for Catawba Unit 1.

1. Changes to the maintenance/surveillance program for the TDI diesel engines, as identified in the licensee's submittals of August 1 and September 11, 1986, shall be subject to the provisions of 10 CFR 50.59. The frequency of the major engine overhauls referred to in the license conditions below shall be consistent with Section IV.1. "Overhaul Frequency," in Revision 2 of Appendix II of the Design Review/Quality Revalidation Report which was transmitted by letter dated May 1, 1986, from J. B. George, Owners Group, to H. R. Denton, NRC.
2. Connecting rod assemblies shall be subjected to the following inspections at each major engine overhaul:
 - (a) The surfaces of the rack teeth should be inspected for signs of fretting. If fretting has occurred, it should be subject to an engineering evaluation for appropriate corrective action.
 - (b) All connecting rod bolts should be lubricated in accordance with the engine manufacturer's instructions and torqued to the specifications of the manufacturer. The lengths of the two pairs of bolts above the crankpin should be measured ultrasonically before and after tensioning.
 - (c) The lengths of the two pairs of bolts above the crankpin should be remeasured ultrasonically before detensioning and disassembly of the bolts. If bolt tension is less than 93% of the value at installation, the cause should be determined, appropriate corrective action should be taken, and the interval between checks of bolt tension should be reevaluated.
 - (d) All connecting rod bolts should be visually inspected for thread damage (e.g., galling), and the two pairs of connecting rod bolts above the crankpin should be inspected by magnetic particle testing to verify the continued absence of cracking. All washers used with the bolts should be examined visually for signs of galling or cracking, and replaced if damaged.
 - (e) A visual inspection should be performed of all external surfaces of the link rod box to verify the absence of any signs of service-induced stress.
 - (f) All of the bolt holes in the link rod box should be inspected for thread damage (e.g., galling) or other signs of abnormalities. In addition, the bolt holes subject to the highest stresses (e.g., the pair immediately above the crankpin) should be examined with an appropriate nondestructive method to verify the continued absence of cracking. Any indications should be recorded for engineering evaluation and appropriate corrective action.

APPENDIX D

ADDITIONAL CONDITIONS

FACILITY OPERATING LICENSE NO. NPF-35

Duke Energy Corporation shall comply with the following conditions on the schedules noted below:

| <u>Amendment Number</u> | <u>Additional Condition</u> | <u>Implementation Date</u> |
|-----------------------------|---|--|
| 159 | This amendment requires the licensee to incorporate in the Updated Final Safety Analysis Report (UFSAR) certain changes to the description of the facility. Implementation of this amendment is the incorporation of these changes as described in the licensee's application dated March 7, 1997, as supplemented by letters dated April 2, 10, 16, 22, and 28, 1997, and evaluated in the staff's Safety Evaluation dated April 29, 1997. | Next update of the UFSAR |
| 159 | This amendment requires the licensee to use administrative controls, as described in the licensee's letter of March 7, 1997, and evaluated in the staff's safety evaluation dated April 29, 1997, to restrict the dose-equivalent iodine levels to 0.46 microCurie per gram (in lieu of the limit in TS Section 3.4.8.a), and to 26 microCurie per gram (in lieu of the limit of TS Figure 3.4-1), until this license condition is removed by a future amendment. | Immediately upon issuance of the amendment |



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

DUKE POWER COMPANY

NORTH CAROLINA MUNICIPAL POWER AGENCY NO. 1

PIEDMONT MUNICIPAL POWER AGENCY

DOCKET NO. 50-414

CATAWBA NUCLEAR STATION, UNIT 2

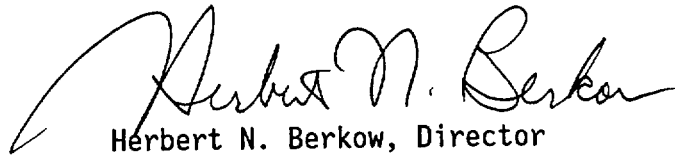
AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 153
License No. NPF-52

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment to the Catawba Nuclear Station, Unit 2 (the facility) Facility Operating License No. NPF-52 filed by the Duke Power Company, acting for itself, North Carolina Municipal Power Agency No. 1 and Piedmont Municipal Power Agency (licensees), dated May 27, 1997, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations as set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is hereby amended by page changes to the license itself, Attachment 1, and Appendix D as indicated in the attachment to this license amendment.
3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Herbert N. Berkow, Director
Project Directorate II-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachments: 1. Facility Operating License No. NPF-52
2. Attachment 1
3. Appendix D

Date of Issuance: August 22, 1997

ATTACHMENT TO LICENSE AMENDMENT NO. 153

FACILITY OPERATING LICENSE NO. NPF-52

DOCKET NO. 50-414

The revised pages are identified by Amendment number and contain vertical lines indicating the areas of change.

Replace Facility Operating License No. NPF-52 in its entirety with the attached new license.

Replace page 1 of Attachment 1 of the license.

Replace Appendix D in its entirety with the attached revised Appendix D.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

DUKE ENERGY CORPORATION

NORTH CAROLINA MUNICIPAL POWER AGENCY NO. 1

PIEDMONT MUNICIPAL POWER AGENCY

DOCKET NO. 50-414

CATAWBA NUCLEAR STATION, UNIT 2

FACILITY OPERATING LICENSE

License No. NPF-52

1. The Nuclear Regulatory Commission (the Commission or the NRC) has found that:
 - A. The application for license filed by the Duke Energy Corporation acting for itself, North Carolina Municipal Power Agency No. 1 and Piedmont Municipal Power Agency (the licensees) complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I; and all required notifications to other agencies or bodies have been duly made;
 - B. Construction of the Catawba Nuclear Station, Unit 2 (the facility) has been substantially completed in conformity with Construction Permit No. CPPR-117 and the application, as amended, the provisions of the Act and the regulations of the Commission;
 - C. The facility will operate in conformity with the application, as amended, the provisions of the Act, and the regulations of the Commission (except as exempted from compliance in Section 2.D. below);
 - D. There is reasonable assurance: (i) that the activities authorized by this operating license can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I (except as exempted from compliance in Section 2.D. below);
 - E. Duke Energy Corporation* is technically qualified to engage in the

*Duke Energy Corporation is authorized to act as agent for the North Carolina Municipal Power Agency No. 1 and Piedmont Municipal Power Agency, and has exclusive responsibility and control over the physical construction, operation, and maintenance of the facility.

activities authorized by this license in accordance with the Commission's regulations set forth in 10 CFR Chapter I;

- F. The licensees have satisfied the applicable provisions of 10 CFR Part 140, "Financial Protection Requirements and Indemnity Agreements," of the Commission's regulations;
 - G. The issuance of this license will not be inimical to the common defense and security or to the health and safety of the public;
 - H. After weighing the environmental, economic, technical, and other benefits of the facility against environmental and other costs and considering available alternatives, the issuance of this Facility Operating License No. NPF-52, subject to the conditions for protection of the environment set forth in the Environmental Protection Plan attached as Appendix B, is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied;
 - I. The receipt, possession, and use of source, byproduct and special nuclear material as authorized by this license will be in accordance with the Commission's regulations in 10 CFR Parts 30, 40, and 70.
2. Based on the foregoing findings and the July 26, 1985, and the November 21, 1985, affirmations by the Atomic Safety and Licensing Appeal Board of the Partial Initial Decisions issued by the Atomic Safety and Licensing Boards dated June 22, September 18, and November 27, 1984, regarding this facility and satisfaction of conditions therein imposed, and pursuant to approval by the Nuclear Regulatory Commission at a meeting held on May 14, 1986, Facility Operating License No. NPF-48, issued on February 24, 1986, is superseded by Facility Operating License No. NPF-52, hereby issued to the Duke Energy Corporation, the North Carolina Municipal Power Agency No. 1 and Piedmont Municipal Power Agency (the licensees) to read as follows:
- A. This license applies to the Catawba Nuclear Station, Unit 2, a pressurized water reactor and associated equipment (the facility) owned by the North Carolina Municipal Power Agency No. 1 and Piedmont Municipal Power Agency and operated by Duke Energy Corporation. The facility is located on the licensees' site in York County, South Carolina, on the shore of Lake Wylie approximately 6 miles north of Rock Hill, South Carolina, and is described in Duke Energy Corporation's Final Safety Analysis Report, as supplemented and amended, and in its Environmental Report, as supplemented and amended;
 - B. Subject to the conditions and requirements incorporated herein, the Commission hereby licenses:

- (1) Duke Energy Corporation, pursuant to Section 103 of the Act and 10 CFR Part 50, to possess, use, and operate the facility at the designated location in York County, South Carolina, in accordance with the procedures and limitations set forth in this license;
- (2) North Carolina Municipal Power Agency No. 1 and Piedmont Municipal Power Agency, pursuant to the Act and 10 CFR Part 50, to possess the facility at the designated location in York County, South Carolina, in accordance with the procedures and limitations set forth in this license;
- (3) Duke Energy Corporation, pursuant to the Act and 10 CFR Part 70, to receive, possess, and use at any time special nuclear material as reactor fuel, in accordance with the limitations for storage and amounts required for reactor operation, as described in the Final Safety Analysis Report, as supplemented and amended;
- (4) Duke Energy Corporation, pursuant to the Act and 10 CFR Parts 30, 40, and 70 to receive, possess, and use at any time any byproduct, source and special nuclear material as sealed neutron sources for reactor startup, sealed sources for reactor instrumentation and radiation monitoring equipment calibration, and as fission detectors in amounts as required;
- (5) Duke Energy Corporation, pursuant to the Act and 10 CFR Parts 30, 40, and 70, to receive, possess, and use in amounts as required any byproduct, source or special nuclear material without restriction to chemical or physical form, for sample analysis or instrument calibration or associated with radioactive apparatus or components;
- (6) Duke Energy Corporation, pursuant to the Act and 10 CFR Parts 30, 40 and 70, to possess, but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility authorized herein; and
- (7) Duke Energy Corporation, pursuant to the Act and 10 CFR Parts 30, 40, and 70, to possess, but not separate, such byproduct and special nuclear materials as may be produced by the operation of McGuire Nuclear Station, Units 1 and 2, and Oconee Nuclear Station, Units 1, 2, and 3.

C. This license shall be deemed to contain and is subject to the conditions specified in the Commission's regulations set forth in 10 CFR Chapter I and is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:

(1) Maximum Power Level

Duke Energy Corporation is authorized to operate the facility at reactor core power levels not in excess of 3411 megawatts thermal (100 percent power) in accordance with the conditions specified herein.

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 153, which are attached hereto, are hereby incorporated into this license. Duke Energy Corporation shall operate the facility in accordance with the Technical Specifications.

(3) Initial Startup Test Program (Section 14, SER, SSER #3, SSER #4, SSER #5, SSER #6)*

Any changes to the Initial Test Program described in Section 14 of the FSAR made in accordance with the provisions of 10 CFR 50.59 shall be reported in accordance with 50.59(b) within one month of such change.

(4) Antitrust Conditions

Duke Energy Corporation shall comply with the antitrust conditions delineated in Appendix C to this license.

(5) Inservice Inspection Program (Sections 5.2.4 and 6.6, SSER #2, SSER #5)

By August 24, 1986, Duke Energy Corporation shall submit the balance of the inservice inspection program as described in its letter dated January 8, 1985, for staff review and approval.

(6) Fire Protection Program (Section 9.5.1, SER, SSER #2, SSER #3, SSER #4, SSER #5)

Duke Energy Corporation shall implement and maintain in effect all provisions of the approved fire protection program as described in the Final Safety Analysis Report, as amended, for the facility and as approved in the SER through Supplement 6, subject to the following provision below:

The licensee may make changes to the approved fire protection program without prior approval of the Commission only if those changes would not adversely affect the ability to achieve and maintain safe shutdown in the event of a fire.

*The parenthetical notation following the title of many license conditions denotes the section of the Safety Evaluation Report and/or its supplements wherein the license condition is discussed.

- (7) Detailed Control Room Design Review, I.D.1 (Section 18, SER, SSER #2, SSER #5)

Duke Energy Corporation shall correct all human engineering deficiencies according to the schedule contained in its letter dated March 28, 1984.

- (8) Emergency Response Capabilities (Generic Letter 82-33, Supplement 1 to NUREG-0737).

(a) Deleted

- (b) Safety Parameter Display System (SPDS) (Section 18, SSER #5)

Prior to December 8, 1989, Duke Energy Corporation shall add to the existing SPDS and have operational the following SPDS parameters: (a) residual heat removal flow, (b) containment isolation status, (c) stack radiation measurements, and (d) steam generator or steamline radiation. The actual value of these and all other SPDS variables should be displayed for operator viewing in easily and rapidly accessible display formats.

- (9) Anticipatory Reactor Trip, II.K.3.10 (Section 5.2.2, SER)

Prior to exceeding 70% power, Duke Energy Corporation shall complete the described turbine trip tests to verify that PORVs will not be challenged when the anticipatory trip bypass is in effect.

- (10) Steam Generator Tube Rupture (Section 15.4.4, SER, SSER #2)

Prior to startup following the first refueling outage of Catawba Unit 2, Duke Energy Corporation shall submit for NRC staff review and approval an analysis which demonstrates that the steam generator single-tube rupture analysis presented in the FSAR is the most severe case with respect to the release of fission products and calculated doses. Consistent with the analytical assumptions, Duke Energy Corporation shall propose any necessary changes to Appendix A to this license.

- (11) Deleted

- (12) Generic Letter 83-28 (Section 15.6, SSER #4, SSER #5)

Duke Energy Corporation shall submit responses to and implement the guidance of Generic Letter 83-28 on a schedule which is consistent with that given in its November 2 and December 31, 1984, letters.

(13) Additional Conditions

The Additional Conditions contained in Appendix D, as revised through Amendment No. 153, are hereby incorporated into this license. Duke Energy Corporation shall operate the facility in accordance with the Additional Conditions.

- D. The facility requires exemptions from certain requirements of Appendix J to 10 CFR Part 50, as delineated below, and pursuant to evaluations contained in the referenced SER and SSERs. These include (a) partial exemption from the requirement of paragraph III.D.2(b)(ii) of Appendix J, the testing of containment airlocks at times when the containment integrity is not required (Section 6.2.6 of SSER #5), (b) exemption from the requirement of paragraph III.A.1(d) of Appendix J, insofar as it requires the venting and draining of lines for type A tests (Section 6.2.6 of SSER #5), and (c) partial exemption from the requirements of paragraph III.B of Appendix J, as it relates to bellows testing (Section 6.2.6 of the SER and SSER #5). These exemptions are authorized by law, will not present an undue risk to the public health and safety, and are consistent with the common defense and security; and certain special circumstances, as discussed in Section 6.2.6 of SSER #5, are present. These exemptions are, therefore, hereby granted pursuant to 10 CFR 50.12. With the granting of these exemptions, the facility will operate, to the extent authorized herein, in conformity with the application, as amended, the provisions of the Act, and the rules and regulations of the Commission. In addition, two exemptions were previously granted pursuant to 10 CFR 50.12. A partial exemption from those portions of General Design Criterion 4 of Appendix A to 10 CFR 50 which require protection of structures, systems and components important to safety against dynamic effects associated with postulated reactor coolant system pipe breaks was granted on April 23, 1985, for a period ending with the completion of the second refueling outage for Catawba Unit 2 or the adoption of the proposed rulemaking for modification of GDC-4 whichever occurs first. Effective May 12, 1986, GDC-4 has been modified to exclude from the design basis the protection of structures, systems and components against the dynamic effects associated with postulated pipe ruptures of primary coolant loop piping in PWRs when analyses demonstrate the probability of rupture of such piping to be extremely low under design basis conditions (51 FR 12502 April 11, 1986). As a result of this final rule and Duke Energy Corporation's demonstration in accordance with the rule, the previously granted specific partial exemption will no longer be required, on the rule's effective date, and terminate by its own terms. Furthermore, an exemption from the requirements of Appendix E, IV.F, insofar as they may require the active participation of all Crisis Management Center personnel for the Catawba Station emergency preparedness exercises (Section 13.3 of SSER #4), was granted on January 17, 1985, by the issuance of Facility Operating License No. NPF-35 for Catawba Nuclear Station, Unit 1.

E. Duke Energy Corporation shall fully implement and maintain in effect all provisions of the Commission-approved physical security, guard training and qualification, and safeguards contingency plans including amendments made pursuant to provisions of the Miscellaneous Amendments and Search Requirements revisions to 10 CFR 73.55 (51 FR 27817 and 27822) and to the authority of 10 CFR 50.90 and CFR 10 CFR 50.54(p). The plans, which contain safeguards information protected under 10 CFR 73.21, are entitled: "Catawba Nuclear Station Physical Security Plan," with revisions submitted through October 6, 1987; "Catawba Nuclear Station Training and Qualification Plan," with revisions submitted through August 27, 1986; and Catawba Nuclear Station Safeguards Contingency Plan," with revisions submitted through January 8, 1987. Changes made in accordance with 10 CFR 73.55 shall be implemented in accordance with the schedule set forth therein.

F. Reporting to the Commission

Except as otherwise provided in the Technical Specifications or Environmental Protection Plan, Duke Energy Corporation shall report any violations of the requirements contained in Section 2.C of this license in the following manner: initial notification shall be made within twenty-four (24) hours to the NRC Operations Center via the Emergency Notification System with written follow-up within 30 days in accordance with the procedures described in 10 CFR 50.73 (b), (c), and (e).

G. The licensees shall have and maintain financial protection of such type and in such amounts as the Commission shall require in accordance with Section 170 of the Atomic Energy Act of 1954, as amended, to cover public liability claims.

H. This license is effective as of the date of issuance and shall expire at midnight on February 24, 2026.

FOR THE NUCLEAR REGULATORY COMMISSION

Original Signed By:

Harold R. Denton, Director
Office of Nuclear Reactor Regulation

Enclosures:

1. Attachment 1
2. Appendix A - Technical Specifications
3. Appendix B - Environmental Protection Plan
4. Appendix C - Antitrust Conditions
5. Appendix D - Additional Conditions

Date of Issuance: May 15, 1986

Amendment No. 153

May 15, 1986

ATTACHMENT 1 TO LICENSE NPF-52

TDI DIESEL ENGINES REQUIREMENTS

Duke Energy Corporation shall comply with the following requirements related to the TDI diesel engines for Catawba Unit 2.

1. Changes to the maintenance/surveillance program for the TDI diesel engines, as identified in the licensee's submittals of August 1 and September 11, 1986, shall be subject to the provisions of 10 CFR 50.59. The frequency of the major engine overhauls referred to in the license conditions below shall be consistent with Section IV.1. "Overhaul Frequency," in Revision 2 of Appendix II of the Design Review/Quality Revalidation Report which was transmitted by letter dated May 1, 1986, from J.B. George, Owners Group, to H.R. Denton, NRC.
2. Connecting rod assemblies shall be subjected to the following inspections at each major engine overhaul:
 - (a) The surfaces of the rack teeth should be inspected for signs of fretting. If fretting has occurred, it should be subject to an engineering evaluation for appropriate corrective action.
 - (b) All connecting rod bolts should be lubricated in accordance with the engine manufacturer's instructions and torqued to the specifications of the manufacturer. The lengths of the two pairs of bolts above the crankpin should be measured ultrasonically before and after tensioning.
 - (c) The lengths of the two pairs of bolts above the crankpin should be remeasured ultrasonically before detensioning and disassembly of the bolts. If bolt tension is less than 93% of the value at installation, the cause should be determined, appropriate corrective action should be taken, and the interval between checks of bolt tension should be reevaluated.
 - (d) All connecting rod bolts should be visually inspected for thread damage (e.g., galling), and the two pairs of connecting rod bolts above the crankpin should be inspected by magnetic particle testing to verify the continued absence of cracking. All washers used with the bolts should be examined visually for signs of galling or cracking, and replaced if damaged.
 - (e) A visual inspection should be performed of all external surfaces of the link rod box to verify the absence of any signs of service-induced stress.

APPENDIX D

ADDITIONAL CONDITIONS

FACILITY OPERATING LICENSE NO. NPF-52

Duke Energy Corporation shall comply with the following conditions on the schedules noted below:

| <u>Amendment Number</u> | <u>Additional Condition</u> | <u>Implementation Date</u> |
|-----------------------------|---|--|
| 151 | This amendment requires the licensee to incorporate in the Updated Final Safety Analysis Report (UFSAR) certain changes to the description of the facility. Implementation of this amendment is the incorporation of these changes as described in the licensee's application dated March 7, 1997, as supplemented by letters dated April 2, 10, 16, 22, and 28, 1997, and evaluated in the staff's Safety Evaluation dated April 29, 1997. | Next update of the UFSAR |
| 151 | This amendment requires the licensee to use administrative controls, as described in the licensee's letter of March 7, 1997, and evaluated in the staff's safety evaluation dated April 29, 1997, to restrict the dose-equivalent iodine levels to 0.46 microCurie per gram (in lieu of the limit in TS Section 3.4.8.a), and to 26 microCurie per gram (in lieu of the limit of TS Figure 3.4-1), until this license condition is removed by a future amendment. | Immediately upon issuance of the amendment |



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 161 TO FACILITY OPERATING LICENSE NPF-35
AND AMENDMENT NO. 153 TO FACILITY OPERATING LICENSE NPF-52

DUKE ENERGY CORPORATION, ET AL.
CATAWBA NUCLEAR STATION, UNITS 1 AND 2
DOCKET NOS. 50-413 AND 50-414

1.0 INTRODUCTION

By letter dated June 12, 1997, Duke Power Company, et al. (DPC or the licensee), submitted a request to amend the operating licenses for Catawba, McGuire, and Oconee nuclear stations to reflect the licensee's name change from "Duke Power Company" to "Duke Energy Corporation."

2.0 DISCUSSION AND EVALUATION

The licensee has recently undergone significant growth through a consolidation with PanEnergy Corporation, a natural gas company. In a June 5, 1997, letter and accompanying safety evaluation, the staff separately analyzed this consolidation, and found that it did not raise any regulatory concerns or require NRC approval under 10 CFR 50.80. The licensee now seeks approval of amendments to the licenses to reflect the licensee's decision to change its name to Duke Energy Corporation. The proposed amendments do not involve any substantive changes in the licenses or alter any responsibilities or obligations of the licensee, and, thus, are administrative in nature.

The staff reviewed the Catawba Units 1 and 2 operating licenses and determined that the name change needs to be effected as follows:

- The operating licenses - All references to "Duke Power company" will be changed to "Duke Energy Corporation."
- Appendix A, Technical Specifications - The only references to "Duke Power company" are made in the context of documents issued by the licensee as "Duke Power Company" (multiple examples in Section 6.9.1.9). None of these will be changed unless the referenced documents are revised by the licensee in the future.
- Appendix B, Environmental Protection Plan - This was deleted by Amendment No. 149 (for Unit 1) and No. 143 (for Unit 2). Thus, no name change is necessary.

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- Appendix C, Antitrust Conditions -- The licensee's name does not appear in this. Thus, no name change is necessary.
- Appendix D, Additional Conditions -- This was introduced by Amendment 159 (for Unit 1) and 151 (for Unit 2). The references to "Duke Power Company" will be changed to "Duke Energy Corporation."
- Attachment 1, requirements regarding the diesel generators -- The single reference to "Duke Power Company" will be changed to "Duke Energy Corporation."

Given that all the changes delineated above are purely administrative in that no responsibilities or obligations of the licensee will be altered, and no plant equipment or operations will be affected. The changes, therefore, are acceptable.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the South Carolina State official was notified of the proposed issuance of the amendments. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

Pursuant to 10 CFR 51.21, 51.32, and 51.35, an Environmental Assessment and Finding of No Significant Impact was published in the Federal Register on July 31, 1997 (62 FR 41102).

Accordingly, based on the Environmental Assessment, the Commission has determined that issuance of this amendment will not have a significant effect on the quality of the human environment.

5.0 CONCLUSION

The staff has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: Peter S. Tam

Date: August 22, 1997