

November 25, 1996

Mr. William R. McCollum
Site Vice President
Catawba Nuclear Station
Duke Power Company
4800 Concord Road
York, South Carolina 29745-9635

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SUBJECT: ISSUANCE OF AMENDMENTS - CATAWBA NUCLEAR STATION, UNITS 1 AND 2
(TAC NOS. M95838 AND M95839)

Dear Mr. McCollum:

The Nuclear Regulatory Commission has issued the enclosed Amendment No. 156 to Facility Operating License NPF-35 and Amendment No. 148 to Facility Operating License NPF-52 for the Catawba Nuclear Station, Units 1 and 2. The amendments consist of changes to the Technical Specifications (TS) in response to your application dated June 21, 1996.

The amendments revise Technical Specification (TS) Section 3/4.9.6, "Manipulator Crane," to make the wording consistent with the TS Bases description and consistent with the design of the load handling equipment.

A copy of the related Safety Evaluation is also enclosed. A Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

Original signed by:

Peter S. Tam, Senior Project Manager
Project Directorate II-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket Nos. 50-413 and 50-414

Enclosures: 1. Amendment No. 156 to NPF-35
2. Amendment No. 148 to NPF-52
3. Safety Evaluation

cc w/encl: See next page

DOCUMENT NAME: G:\CATAWBA\CAT95838.AMD

OFFICE	PDII-2/PM	PDII-2/LA	SPLB/SC	OGC	PDII-2/DO
NAME	P.TAM:cn	L.BERRY	G.HUBBARD	APH	H.BERKOW
DATE	10/28/96	10/28/96	10/28/96	11/07/96	11/19/96
COPY	YES NO	YES NO	YES NO	YES NO	YES NO

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

November 25, 1996

Mr. William R. McCollum
Site Vice President
Catawba Nuclear Station
Duke Power Company
4800 Concord Road
York, South Carolina 29745-9635

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Sincerely,

A handwritten signature in cursive script, reading "Peter S. Tam", is positioned above the typed name.

Peter S. Tam, Senior Project Manager
Project Directorate II-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket Nos. 50-413 and 50-414

Enclosures: 1. Amendment No. 156 to NPF-35
2. Amendment No. 148 to NPF-52
3. Safety Evaluation

cc w/encl: See next page

Mr. W. R. McCollum
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cc:

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Catawba Nuclear Station

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

DUKE POWER COMPANY
NORTH CAROLINA ELECTRIC MEMBERSHIP CORPORATION
SALUDA RIVER ELECTRIC COOPERATIVE, INC.
DOCKET NO. 50-413
CATAWBA NUCLEAR STATION, UNIT 1
AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 156
License No. NPF-35

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment to the Catawba Nuclear Station, Unit 1 (the facility) Facility Operating License No. NPF-35 filed by the Duke Power Company, acting for itself, North Carolina Electric Membership Corporation and Saluda River Electric Cooperative, Inc. (licensees), dated June 21, 1996, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations as set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is hereby amended by page changes to the Technical Specifications as indicated in the attachment to this license amendment, and Paragraph 2.C.(2) of Facility Operating License No. NPF-35 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 156, which are attached hereto, are hereby incorporated into this license. Duke Power Company shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Herbert N. Berkow, Director
Project Directorate II-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Technical Specification
Changes

Date of Issuance: November 25, 1996

ATTACHMENT TO LICENSE AMENDMENT NO. 156

FACILITY OPERATING LICENSE NO. NPF-35

DOCKET NO. 50-413

Replace the following page of the Technical Specifications with the enclosed page. The revised page is identified by amendment number and contains vertical lines indicating the areas of change.

Remove Page

Insert Page

3/4 9-9

3/4 9-9

REFUELING OPERATIONS

3/4.9.6 MANIPULATOR CRANE

LIMITING CONDITION FOR OPERATION

3.9.6 The reactor building manipulator crane and an auxiliary hoist shall be used for movement of fuel assemblies or control rods and shall be OPERABLE with:

- a. The manipulator crane used for movement of fuel assemblies having:
 - 1) A minimum capacity of 3250 pounds, and
 - 2) An overload cutoff limit less than or equal to 2900 pounds.
- b. Auxiliary hoists used for latching, unlatching and drag load testing of control rods having:
 - 1) A minimum capacity of 1000 pounds, and
 - 2) A load indicator which shall be used to prevent applying a lifting force in excess of 600 pounds on the core internals.

APPLICABILITY: During movement of fuel assemblies and control rods within the reactor vessel.

ACTION:

With the requirements for crane and/or hoist OPERABILITY not satisfied, suspend use of any inoperable manipulator crane and/or auxiliary hoist from operations involving the movement of fuel assemblies and control rods within the reactor vessel.

SURVEILLANCE REQUIREMENTS

4.9.6.1 Each manipulator crane used for movement of fuel assemblies within the reactor vessel shall be demonstrated OPERABLE within 72 hours prior to the start of such operations by performing a load test of at least 3250 pounds and demonstrating an automatic load cutoff when the crane load exceeds 2850 pounds.

4.9.6.2 Each auxiliary hoist and associated load indicator used for movement of control rods or control rod drag load testing within the reactor vessel shall be demonstrated OPERABLE within 72 hours prior to the start of such operations by performing a load test of at least 1000 pounds.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

DUKE POWER COMPANY

NORTH CAROLINA MUNICIPAL POWER AGENCY NO. 1

PIEDMONT MUNICIPAL POWER AGENCY

DOCKET NO. 50-414

CATAWBA NUCLEAR STATION, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 148
License No. NPF-52

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment to the Catawba Nuclear Station, Unit 2 (the facility) Facility Operating License No. NPF-52 filed by the Duke Power Company, acting for itself, North Carolina Municipal Power Agency No. 1 and Piedmont Municipal Power Agency (licensees), dated June 21, 1996, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations as set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

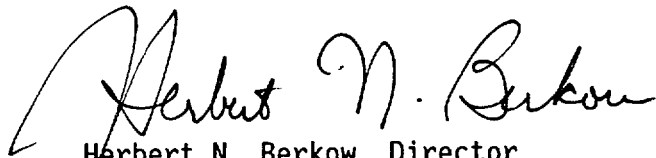
2. Accordingly, the license is hereby amended by page changes to the Technical Specifications as indicated in the attachment to this license amendment, and Paragraph 2.C.(2) of Facility Operating License No. NPF-52 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 148 , which are attached hereto, are hereby incorporated into this license. Duke Power Company shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Herbert N. Berkow, Director
Project Directorate II-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Technical Specification
Changes

Date of Issuance: November 25, 1996

ATTACHMENT TO LICENSE AMENDMENT NO. 148

FACILITY OPERATING LICENSE NO. NPF-52

DOCKET NO. 50-414

Replace the following page of the Technical Specifications with the enclosed page. The revised page is identified by amendment number and contains vertical lines indicating the areas of change.

Remove Page

3/4 9-9

Insert Page

3/4 9-9

REFUELING OPERATIONS

3/4.9.6 MANIPULATOR CRANE

LIMITING CONDITION FOR OPERATION

3.9.6 The reactor building manipulator crane and an auxiliary hoist shall be used for movement of fuel assemblies or control rods and shall be OPERABLE with:

- a. The manipulator crane used for movement of fuel assemblies having:
 - 1) A minimum capacity of 3250 pounds, and
 - 2) An overload cutoff limit less than or equal to 2900 pounds.
- b. Auxiliary hoists used for latching, unlatching and drag load testing of control rods having:
 - 1) A minimum capacity of 1000 pounds, and
 - 2) A load indicator which shall be used to prevent applying a lifting force in excess of 600 pounds on the core internals.

APPLICABILITY: During movement of fuel assemblies and control rods within the reactor vessel.

ACTION:

With the requirements for crane and/or hoist OPERABILITY not satisfied, suspend use of any inoperable manipulator crane and/or auxiliary hoist from operations involving the movement of fuel assemblies and control rods within the reactor vessel.

SURVEILLANCE REQUIREMENTS

4.9.6.1 Each manipulator crane used for movement of fuel assemblies within the reactor vessel shall be demonstrated OPERABLE within 72 hours prior to the start of such operations by performing a load test of at least 3250 pounds and demonstrating an automatic load cutoff when the crane load exceeds 2850 pounds.

4.9.6.2 Each auxiliary hoist and associated load indicator used for movement of control rods or control rod drag load testing within the reactor vessel shall be demonstrated OPERABLE within 72 hours prior to the start of such operations by performing a load test of at least 1000 pounds.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 156 TO FACILITY OPERATING LICENSE NPF-35
AND AMENDMENT NO. 148 TO FACILITY OPERATING LICENSE NPF-52

DUKE POWER COMPANY, ET AL.

CATAWBA NUCLEAR STATION, UNITS 1 AND 2

DOCKET NOS. 50-413 AND 50-414

1.0 INTRODUCTION

By letter dated June 21, 1996, Duke Power Company, et al. (the licensee), requested license amendments to Facility Operating Licenses NPF-35 and NPF-52 for the Catawba Nuclear Station, Units 1 and 2, respectively. The licensee proposed changes to Technical Specification (TS) 3/4.9.6, "Manipulator Crane," primarily to make the wording consistent with the TS Bases description and consistent with the design of the load handling equipment. One of the proposed changes would revise Limiting Condition for Operation (LCO) 3.9.6b, which currently specifies that the auxiliary hoist shall have (1) a minimum capacity of 610 pounds, and (2) a load indication which shall be used to prevent lifting loads in excess of 600 pounds. The proposed change would specify that auxiliary hoists shall have (1) a minimum capacity of 1000 pounds, and (2) a load indicator which shall be used to prevent applying a lifting force in excess of 600 pounds on the core internals.

2.0 EVALUATION

LCO 3.9.6b.2) currently specifies that the auxiliary hoist used for latching and unlatching drive rods shall have a load indicator which shall be used to prevent "lifting loads" in excess of 600 pounds. The term "lifting loads" has the potential to create some confusion as to whether or not the lifting tool, drive rod, and control rod weights are included in the 600 pound value. The proposed change from "lifting loads" to "lifting force" should alleviate this potential confusion.

The intent of this 600-pound requirement, as stated in the Bases for TS 3/4.9.6, is to ensure the core internals and reactor vessel are protected from excessive lifting forces in the event they are inadvertently engaged during lifting operations. The specific component of concern is the guide tube, which is part of the reactor vessel upper internal structure. The 600-pound "lifting load" limit in the specification refers to the dynamic force that may be inadvertently applied to the guide tube in the event of a jammed rod

control cluster. It does not include loads registered on the indicator by the static weights of the latching tool, drive rod, or control rod. Therefore, the actual limiting load reading on the indicator will be 600 pounds greater than the combined values of the static weight of the latching tool, drive rod, and control rod. The staff has determined that the proposed change is basically a clarification to prevent a misinterpretation of the existing LCO and more accurately reflects actual plant operations and design. Therefore, the change is acceptable.

The increase in the minimum auxiliary hoist capacity as specified in LCO 3.9.6b.1) from 610 pounds to 1000 pounds was proposed to more accurately reflect the actual minimum capacity required and retained by the hoists. A change to Surveillance Requirement (SR) 4.9.6.2 is also proposed such that the minimum required load test is at least 1000 pounds in lieu of at least 610 pounds as specified in the current SR. The staff has determined that this change is conservative in that it provides more safety margin (the hoists are actually rated at 3000 pounds). It is also more consistent with actual plant design and existing test procedures, which actually test the hoists to greater than 1000 pounds. Therefore, the proposed changes are acceptable.

The other proposed wording changes to TS 3/4.9.6 are purely administrative in nature to more accurately reflect the terminology and design of the manipulator crane and auxiliary hoists at Catawba, Units 1 and 2. They are, therefore, acceptable.

Based on the above evaluation, the staff concludes that the proposed changes to LCO 3.9.6 and SR 4.9.6.2 are necessary to more accurately reflect the actual plant design, are basically administrative in nature, and will help prevent potential misinterpretation of the specifications. The proposed changes are also consistent with the corresponding basis of the subject specifications (Bases Sections 3/4.9.6). The staff, therefore, concludes that the proposed changes are acceptable.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the South Carolina State Official was notified of the proposed issuance of the amendment. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

The amendments change requirements with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and change surveillance requirements. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration, and there has been no public comment on such finding (61 FR

55031, dated October 23, 1996). Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

5.0 CONCLUSION

The staff has concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: William T. LeFave

Date: November 25, 1996