

November 2, 1995

Mr. William R. McCollum  
Site Vice President  
Catawba Nuclear Station  
Duke Power Company  
4800 Concord Road  
York, South Carolina 29745-9635

Distribution  
Docket File  
PUBLIC G.Hill(4) T-5 C3  
PDII-2 Reading C.Grimes 0-11 F23  
S.Varga ACRS(1) T-2 E26  
R.Crlenjak, RII E.Merschhoff, RII

SUBJECT: ISSUANCE OF AMENDMENTS - CATAWBA NUCLEAR STATION, UNITS 1 AND 2  
REVISIONS TO TS LISTINGS OF COLR METHODOLOGIES (TAC NOS. M93418  
AND M93419)

Dear Mr. McCollum:

The Nuclear Regulatory Commission has issued the enclosed Amendment No. 138 to Facility Operating License NPF-35 and Amendment No. 132 to Facility Operating License NPF-52 for the Catawba Nuclear Station, Units 1 and 2. The amendments consist of changes to the Technical Specifications (TS) in response to your application dated September 1, 1995, as supplemented October 17, 1995.

The amendments revise TS 6.9.1.9 to include references to updated or recently approved methodologies used to calculate cycle-specific limits contained in the Core Operating Limits Report. The subject references have previously been reviewed and approved by the NRC staff. Therefore, these additions to the TS are administrative in nature.

A copy of the related Safety Evaluation is also enclosed. A Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

Original signed by:

Robert E. Martin, Senior Project Manager  
Project Directorate II-2  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Docket Nos. 50-413 and 50-414

Enclosures:

- 1. Amendment No. 138 to NPF-35
- 2. Amendment No. 132 to NPF-52
- 3. Safety Evaluation

cc w/encl: See next page

*\* See previous concurrence.*

DOCUMENT NAME: G:\CATAWBA\CAT93418.AMD

OFFICIAL RECORD COPY

OFFICE	PD22/LA	PD22/PM	SRXB *	OGD	PD22/D
NAME	L. BERRY	R. MARTIN	R. JONES	R. C. VARGA	H. BERKOW
DATE	10/01/95	10/13/95	10/13/95	10/31/95	10/2/95
COPY	YES NO	YES NO	YES NO	YES NO	YES NO

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NAME	L. BERRY <i>LB</i>	R. MARTIN <i>RM</i>	R. JONES	<i>OG</i>	H. BERKOW
DATE	10/01/95	10/13/95	10/13/95	10/31/95	10/2/95
COPY	<input checked="" type="checkbox"/> YES <input type="checkbox"/> NO	<input checked="" type="checkbox"/> YES <input type="checkbox"/> NO	<input type="checkbox"/> YES <input type="checkbox"/> NO	<input type="checkbox"/> YES <input checked="" type="checkbox"/> NO	<input type="checkbox"/> YES <input checked="" type="checkbox"/> NO



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

November 2, 1995

Mr. William R. McCollum  
Site Vice President  
Catawba Nuclear Station  
Duke Power Company  
4800 Concord Road  
York, SC 29745-9635

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Robert E. Martin, Senior Project Manager  
Project Directorate II-2  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Docket Nos. 50-413 and 50-414

Enclosures:

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2. Amendment No. 132 to NPF-52
3. Safety Evaluation

cc w/encl: See next page

Mr. W. R. McCollum  
Duke Power Company

cc:

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Catawba Nuclear Station

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Raleigh, North Carolina 27603-1335



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

DUKE POWER COMPANY  
NORTH CAROLINA ELECTRIC MEMBERSHIP CORPORATION  
SALUDA RIVER ELECTRIC COOPERATIVE, INC.  
DOCKET NO. 50-413  
CATAWBA NUCLEAR STATION, UNIT 1  
AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 138  
License No. NPF-35

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment to the Catawba Nuclear Station, Unit 1 (the facility) Facility Operating License No. NPF-35 filed by the Duke Power Company, acting for itself, North Carolina Electric Membership Corporation and Saluda River Electric Cooperative, Inc. (licensees), dated September 1, 1995, as supplemented October 17, 1995, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations as set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

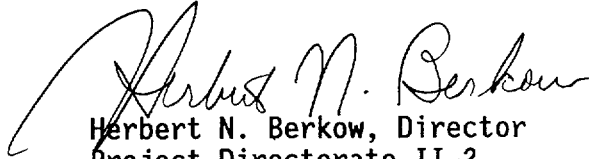
2. Accordingly, the license is hereby amended by page changes to the Technical Specifications as indicated in the attachment to this license amendment, and Paragraph 2.C.(2) of Facility Operating License No. NPF-35 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 138, and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, are hereby incorporated into this license. Duke Power Company shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Herbert N. Berkow, Director  
Project Directorate II-2  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Attachment:  
Technical Specification  
Changes

Date of Issuance: November 2, 1995



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

DUKE POWER COMPANY

NORTH CAROLINA MUNICIPAL POWER AGENCY NO. 1

PIEDMONT MUNICIPAL POWER AGENCY

DOCKET NO. 50-414

CATAWBA NUCLEAR STATION, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 132  
License No. NPF-52

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment to the Catawba Nuclear Station, Unit 2 (the facility) Facility Operating License No. NPF-52 filed by the Duke Power Company, acting for itself, North Carolina Municipal Power Agency No. 1 and Piedmont Municipal Power Agency (licensees), dated September 1, 1995, as supplemented October 17, 1995, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations as set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

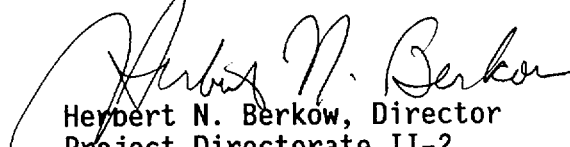
2. Accordingly, the license is hereby amended by page changes to the Technical Specifications as indicated in the attachment to this license amendment, and Paragraph 2.C.(2) of Facility Operating License No. NPF-52 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 132, and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, are hereby incorporated into this license. Duke Power Company shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Herbert N. Berkow, Director  
Project Directorate II-2  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Attachment:  
Technical Specification  
Changes

Date of Issuance: November 2, 1995



ATTACHMENT TO LICENSE AMENDMENT NO. 138

FACILITY OPERATING LICENSE NO. NPF-35

DOCKET NO. 50-413

AND

TO LICENSE AMENDMENT NO. 132

FACILITY OPERATING LICENSE NO. NPF-52

DOCKET NO. 50-414

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the areas of change.

Remove Pages

Insert Pages

6-19  
6-19a  
6-19b

6-19  
6-19a  
6-19b

## ADMINISTRATIVE CONTROLS

### CORE OPERATING LIMITS REPORT (Continued)

10. Accumulator and Refueling Water Storage Tank boron concentration limits for Specifications 3/4.5.1 and 3/4.5.4.
11. Reactor Coolant System and refueling canal boron concentration limits for Specification 3/4.9.1.
12. Standby Makeup Pump water supply boron concentration limit of Specification 4.7.13.3.
13. Spent Fuel Pool boron concentration limit of Specification 3/4.9.12.

The analytical methods used to determine the core operating limits shall be those previously reviewed and approved by NRC in:

1. WCAP-9272-P-A, "WESTINGHOUSE RELOAD SAFETY EVALUATION METHODOLOGY," July 1985 (W Proprietary).  
(Methodology for Specifications 3.1.1.3 - Moderator Temperature Coefficient, 3.1.3.5 - Shutdown Bank Insertion Limit, 3.1.3.6 - Control Bank Insertion Limits, 3.2.1 - Axial Flux Difference, 3.2.2 - Heat Flux Hot Channel Factor, and 3.2.3 - Nuclear Enthalpy Rise Hot Channel Factor.)
2. WCAP-10216-P-A, "RELAXATION OF CONSTANT AXIAL OFFSET CONTROL FQ SURVEILLANCE TECHNICAL SPECIFICATION," June 1983 (W Proprietary).  
(Methodology for Specifications 3.2.1 - Axial Flux Difference (Relaxed Axial Offset Control) and 3.2.2 - Heat Flux Hot Channel Factor (W(Z) surveillance requirements for  $F_0$  Methodology.)
3. WCAP-10266-P-A Rev. 2, "THE 1981 VERSION OF WESTINGHOUSE EVALUATION MODEL USING BASH CODE," March 1987, (W Proprietary).  
(Methodology for Specification 3.2.2 - Heat Flux Hot Channel Factor.)
4. BAW-10168P, Rev. 1, "B&W Loss-of-Coolant Accident Evaluation Model for Recirculating Steam Generator Plants," SER dated January 1991 (B&W Proprietary).  
(Methodology for Specification 3.2.2 - Heat Flux Hot Channel Factor.)

## ADMINISTRATIVE CONTROLS

### CORE OPERATING LIMITS REPORT (Continued)

5. DPC-NE-2011P-A, "Duke Power Company Nuclear Design Methodology for Core Operating Limits of Westinghouse Reactors," March, 1990 (DPC Proprietary).

(Methodology for Specifications 2.2.1 - Reactor Trip System Instrumentation Setpoints, 3.1.3.5 - Shutdown Rod Insertion Limits, 3.1.3.6 - Control Bank Insertion Limits, 3.2.1 - Axial Flux Difference, 3.2.2 - Heat Flux Hot Channel Factor, and 3.2.3 - Nuclear Enthalpy Rise Hot Channel Factor.)

6. DPC-NE-3001P-A, "Multidimensional Reactor Transients and Safety Analysis Physics Parameter Methodology," November 1991 (DPC Proprietary).

(Methodology for Specification 3.1.1.3 - Moderator Temperature Coefficient, 3.1.3.5 - Shutdown Rod Insertion Limits, 3.1.3.6 - Control Bank Insertion Limits, 3.2.1 - Axial Flux Difference, 3.2.2 - Heat Flux Hot Channel Factor, and 3.2.3 - Nuclear Enthalpy Rise Hot Channel Factor.)

7. DPC-NF-2010A, "Duke Power Company McGuire Nuclear Station Catawba Nuclear Station Nuclear Physics Methodology for Reload Design," June 1985

(Methodology for Specification 3.1.1.3 - Moderator Temperature Coefficient, Specification 4.7.13.3 - Standby Makeup Pump Water Supply Boron Concentration, and Specification 3.9.1 - RCS and Refueling Canal Boron Concentration, and Specification 3.9.12 - Spent Fuel Pool Boron Concentration.)

8. DPC-NE-3002A, "FSAR Chapter 15 System Transient Analysis Methodology," November 1991.

(Methodology used in the system thermal-hydraulic analyses which determine the core operating limits)

9. DPC-NE-3000P-A, "Thermal-Hydraulic Transient Analysis Methodology," August 1994.

(Modeling used in the system thermal-hydraulic analyses)

## ADMINISTRATIVE CONTROLS

### CORE OPERATING LIMITS REPORT (Continued)

10. DPC-NE-1004A, "Design Methodology Using CASMO-3/Simulate-3P," November 1992.  
(Methodology for Specification 3.1.1.3 - Moderator Temperature Coefficient.)
11. DPC-NE-2004P-A, "Duke Power Company McGuire and Catawba Nuclear Stations Core Thermal-Hydraulic Methodology using VIPRE-01," December 1991 (DPC Proprietary).  
(Methodology for Specifications 2.2.1 - Reactor Trip System Instrumentation Setpoints, 3.2.1 - Axial Flux Difference (AFD), and 3.2.3 - Nuclear Enthalpy Rise Hot Channel Factor  $F_{\Delta H}(X,Y)$ .)
12. DPC-NE-2001P-A, Rev. 1, "Fuel Mechanical Reload Analysis Methodology for Mark-BW Fuel," October 1990 (DPC Proprietary).  
(Methodology for Specification 2.2.1 - Reactor Trip System Instrumentation Setpoints.)
13. DPC-NE-2005P-A, "Thermal Hydraulic Statistical Core Design Methodology," February 1995 (DPC Proprietary).  
(Methodology for Specification 2.2.1 - Reactor Trip System Instrumentation Setpoints, Specifications 3.2.1 - Axial Flux Difference, and 3.2.3 - Nuclear Enthalpy Rise Hot Channel Factor)
14. BAW-10162P-A, TACO3 Fuel Pin Thermal Analysis Computer Code, B&W Fuel Company, November 1989.  
(Methodology used for Specification 2.2.1 - Reactor Trip System Instrumentation Setpoints)
15. BAW-10183P, Fuel Rod Gas Pressure Criterion, B&W Fuel Company, May 1994.  
(Used for Specification 2.2.1, Reactor Trip System Instrumentation Setpoints)

The core operating limits shall be determined so that all applicable limits (e.g., fuel thermal-mechanical limits, core thermal-hydraulic limits, ECCS limits, nuclear limits such as shutdown margin, and transient and accident analysis limits) of the safety analysis are met.

The CORE OPERATING LIMITS REPORT, including any mid-cycle revisions or supplements thereto, shall be provided upon issuance, for each reload cycle, to the NRC in accordance with 10 CFR 50.4.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
RELATED TO AMENDMENT NO. 138 TO FACILITY OPERATING LICENSE NPF-35  
AND AMENDMENT NO. 132 TO FACILITY OPERATING LICENSE NPF-52  
DUKE POWER COMPANY, ET AL.  
CATAWBA NUCLEAR STATION, UNITS 1 AND 2  
DOCKET NOS. 50-413 AND 50-414

1.0 INTRODUCTION

By letter dated September 1, 1995, as supplemented by letter dated October 17, 1995, Duke Power Company (DPC), et al. (the licensee), submitted a request for changes to the Catawba Nuclear Station, Units 1 and 2, Technical Specifications (TS). The requested changes would revise TS 6.9.1.9 to include references to updated or recently approved methodologies used to calculate cycle-specific limits contained in the Core Operating Limits Report. The subject references have previously been reviewed and approved by the NRC staff. Therefore, these additions to the TS are administrative in nature. The October 17, 1995, letter provided clarifying information that did not change the scope of the September 1, 1995, application and the initial proposed no significant hazards consideration determination.

2.0 EVALUATION

Specifically, the amendments:

- (1) Propose to change report number "DPC-NF-2010P-A" to "DPC-NF-2010A." This eliminates the "P" designator for proprietary information. This is appropriate since DPC-NF-2010-A is not a proprietary report.
- (2) Add three additional reports to the TS 6.9.1.9 list of approved reports as follows:
  - Item number 13 - DPC-NE-2005P-A, as approved.

The staff issued its evaluation of the Duke Power Company Topical Report DPC-NE-2005P-A, "Thermal Hydraulic Statistical Core Design Methodology" by letter from G. M. Holahan, NRC, to H. B. Tucker, DPC, dated February 27, 1995. This report documents the development of core thermal-hydraulic analysis based upon the statistical core design methodology using the VIPRE-01 computer code for the Catawba, McGuire and Oconee Nuclear Stations. The staff concluded that the report is acceptable for referencing in license applications to the extent specified and under the limitations delineated in the report and the associated NRC evaluation. On these bases, the staff finds this proposed revision to TS 6.9.1.9, to add report DPC-NE-2005P-A, as approved, to be acceptable.

- Items number 14 and number 15 - BAW-10162P-A and BAW-10183P.

The staff issued its evaluation of the Babcock & Wilcox (B&W) Topical Report BAW-10162P, "TACO3 - Fuel Pin Thermal Analysis Computer Code" by letter from A. C. Thadani, NRC, to J. H. Taylor, B&W, dated August 14, 1989. The revised TACO3 code, addressed by this evaluation, was developed to provide predictions of the thermal and mechanical performance of pressurized water reactor fuel rods experiencing variable power histories up to a particular burnup level. The staff's review concluded that the Topical Report provided an acceptable basis for changes to the B&W TACO3 computer code.

The staff issued its evaluation of the B&W Topical Report BAW-10183P, "Fuel Rod Gas Pressure Criterion" by letter dated February 22, 1994, from A. C. Thadani, NRC, to J. H. Taylor, B&W. The BAW-10183P report describes a fuel rod gas pressure criterion that the B&W Fuel Company (BWFC) would apply to existing fuel designs to allow the rod pressure to exceed system pressure under certain conditions. The staff's review concluded that the Topical Report provides an acceptable basis for the fuel rod gas pressure criterion for licensing applications.

By letter dated May 4, 1994, DPC requested that NRC review and approve the transfer of the fuel performance code TACO3 from BWFC to DPC for reload licensing applications. The transfer includes the approved Topical Reports BAW-10162P and BAW-10183P, as approved. The NRC staff concluded, in a letter from H. N. Berkow, NRC, to M. S. Tuckman, DPC, dated April 3, 1995, that DPC has the technical capability to perform TACO3 analyses for reload licensing applications and therefore, the use of TACO3 by DPC for the Catawba, McGuire, and Oconee Nuclear Stations is acceptable.

The proposed revision in DPC's application of September 1, 1995, adds the references for the two methodology reports, BAW-10162P-A and BAW-10183P, as approved, to TS 6.9.1.9. On the basis of its review of the licensee's submittals summarized above, the NRC staff finds that this proposed revision to TS 6.9.1.9 is acceptable.

The inclusion of these approved methodologies ensures that the core operating limits shall be determined so that all applicable limits of the safety analysis are met. Therefore, the proposed TS changes are acceptable.

### 3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the South Carolina State official was notified of the proposed issuance of the amendments. The State official had no comments.

### 4.0 ENVIRONMENTAL CONSIDERATION

The amendments change recordkeeping, reporting, or administrative procedures or requirements. Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22d(c)(10). Pursuant to 10

CFR 51.22(b), no environmental impact statement or environmental assessment need to be prepared in connection with the issuance of the amendments.

#### 5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: Robert E. Martin

Date: November 2, 1995