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Mr. William R. McCollum Site Vice President Catawba Nuclear Station Duke Power Company 4800 Concord Road York, South Carolina 29745-9635

SUBJECT:

ISSUANCE OF AMENDMENTS - CATAWBA NUCLEAR STATION, UNITS 1 AND 2,

STEAM GENERATOR PORV STROKE TIMES, (TAC NOS. M91467, M91468)

Dear Mr. McCollum:

The Nuclear Regulatory Commission has issued the enclosed Amendment No. 139 to Facility Operating License NPF-35 and Amendment No. 133 Operating License NPF-52 for the Catawba Nuclear Station, Units 1 and 2. The amendments consist of changes to the Technical Specifications (TS) in response to your application dated October 31, 1994.

The amendments remove the stroke times for the steam generator power operated relief valves from TS Tables 3.6-2a and 3.6-2b.

A copy of the related Safety Evaluation is also enclosed. A Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

Original signed by:

Robert E. Martin, Senior Project Manager Project Directorate II-2 Division of Reactor Projects - I/II Office of Nuclear Reactor Regulation

Docket Nos. 50-413 and 50-414

Enclosures:

1. Amendment No. 139 to NPF-35

2. Amendment No. 133 to NPF-52

3. Safety Evaluation

cc w/encl: See next page

\* See previous concurrence

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WASHINGTON, D.C. 20555-0001

December 18, 1995

Mr. William R. McCollum Site Vice President Catawba Nuclear Station Duke Power Company 4800 Concord Road York, South Carolina 29745-9635

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Robert E. Martin, Senior Project Manager

Project Directorate II-2

Division of Reactor Projects - I/II Office of Nuclear Reactor Regulation

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1. Amendment No. 139 to NPF-35

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3. Safety Evaluation

cc w/encl: See next page

Mr. W. R. McCollum Duke Power Company

cc: Mr. Z. L. Taylor Regulatory Compliance Manager Duke Power Company 4800 Concord Road York, South Carolina 29745

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WASHINGTON, D.C. 20555-0001

#### **DUKE POWER COMPANY**

#### NORTH CAROLINA ELECTRIC MEMBERSHIP CORPORATION

#### SALUDA RIVER ELECTRIC COOPERATIVE, INC.

**DOCKET NO. 50-413** 

#### CATAWBA NUCLEAR STATION, UNIT 1

#### AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 139 License No. NPF-35

- 1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment to the Catawba Nuclear Station, Unit 1 (the facility) Facility Operating License No. NPF-35 filed by the Duke Power Company, acting for itself, North Carolina Electric Membership Corporation and Saluda River Electric Cooperative, Inc. (licensees), dated October 31, 1994 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations as set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is hereby amended by page changes to the Technical Specifications as indicated in the attachment to this license amendment, and Paragraph 2.C.(2) of Facility Operating License No. NPF-35 is hereby amended to read as follows:

#### Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 139 , and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, are hereby incorporated into this license. Duke Power Company shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Leonard A. Wiens, Acting Director

Project Directorate II-2

Division of Reactor Projects - I/II Office of Nuclear Reactor Regulation

Attachment: Technical Specification Changes

Date of Issuance: December 18, 1995



WASHINGTON, D.C. 20555-0001

#### **DUKE POWER COMPANY**

#### NORTH CAROLINA MUNICIPAL POWER AGENCY NO. 1

#### PIEDMONT MUNICIPAL POWER AGENCY

**DOCKET NO. 50-414** 

CATAWBA NUCLEAR STATION, UNIT 2

#### AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 133 License No. NPF-52

- I. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment to the Catawba Nuclear Station, Unit 2 (the facility) Facility Operating License No. NPF-52 filed by the Duke Power Company, acting for itself, North Carolina Municipal Power Agency No. 1 and Piedmont Municipal Power Agency (licensees), dated October 31, 1994 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations as set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is hereby amended by page changes to the Technical Specifications as indicated in the attachment to this license amendment, and Paragraph 2.C.(2) of Facility Operating License No. NPF-52 is hereby amended to read as follows:

#### <u>Technical Specifications</u>

The Technical Specifications contained in Appendix A, as revised through Amendment No. 133, and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, are hereby incorporated into this license. Duke Power Company shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Leonard A. Wiens, Acting Director

Project Directorate II-2

Division of Reactor Projects - I/II Office of Nuclear Reactor Regulation

Attachment: Technical Specification Changes

Date of Issuance: December 18, 1995

#### ATTACHMENT TO LICENSE AMENDMENT NO. 139

#### FACILITY OPERATING LICENSE NO. NPF-35

**DOCKET NO. 50-413** 

<u>and</u>

TO LICENSE AMENDMENT NO. 133

#### FACILITY OPERATING LICENSE NO. NPF-52

**DOCKET NO. 50-414** 

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the areas of change.

Remove Pages	<u>Insert Pages</u>			
3/4 6-27	3/4 6-27			
3/4 6-35	3/4 6-35			

### TABLE 3.6-2a (Continued)

## CONTAINMENT ISOLATION VALVES

BA -	<u>VALVE</u>	NUMBER	<u>FUNCTION</u>	MAXIMUM ISOLATION TIME (s)
SLIND	2.	Phase "B"	Isolation (Continued)	
1 & 2		RN-437B RN-484A RN-487A RN-404B RN-429B RN-432B	Supply to NC Pumps and LCVU Supply Outside Containment Isolation Return from NC Pumps and LCVU Return Inside Containment Isolation Return from NC Pumps and LCVU Return Outside Containment Isolation Supply to Upper Containment Supply Ventilation Units Containment Isolation (Outside) Return from Upper Containment Ventilation Units Containment Isolation (Inside) Return from Upper Containment Ventilation Units Containment Isolation (Outside)	≤ 60 ≤ 60 ≤ 60 ≤ 10 ≤ 10 ≤ 10
3/4 6-27		VI-77B SM-1 # SM-3 #	Instrument Air Containment Outside Isolation  Main Steam 1D Isolation	≤ 10 NA
Amendment Amendment		SM-5 # SM-7 # SM-9 # SM-10 # SM-11 # SM-12 #	Main Steam 1C Isolation Main Steam 1B Isolation Main Steam 1A Isolation Main Steam 1D Isolation Bypass Ctrl. Main Steam 1C Isolation Bypass Ctrl. Main Steam 1B Isolation Bypass Ctrl. Main Steam 1A Isolation Bypass Ctrl.	NA NA NA NA NA NA
ent No. 139 (U		SV-19 # SV-13 # SV-7 # SV-1 # WL-867A** WL-869B**	Main Steam 1A PORV Main Steam 1B PORV Main Steam 1C PORV Main Steam 1D PORV Main Steam 1D PORV Containment Vent Unit Drains Inside Containment Isolation Containment Vent Unit Drains Outside Containment Isolation	NA NA NA NA ≤ 10 ≤ 10

## TABLE 3.6-2b (Continued)

## UNIT 2 CONTAINMENT ISOLATION VALVES

VBA - (	<u>VALVE</u>	NUMBER	<u>FUNCTION</u>	MAXIMUM ISOLATION TIME (s)
SLIND	2.	Phase "B"	Isolation (Continued)	
		RN-437B	Supply to NC Pumps and LCVU Supply Outside Containment Isolation	≤ 60
Qο		RN-484A	Return from NC Pumps and LCVU Return Inside Containment Isolation	≤ 60
2		RN-487A	Return from NC Pumps and LCVU Return Outside Containment Isolation	≤ 60
		RN-404B	Supply to Upper Containment Supply Ventilation Units Containment Isolation (Outside)	≤ 10
		RN-429B	Return from Upper Containment Ventilation Units Containment Isolatic (Inside)	on ≤ 10
		RN-432B	Return from Upper Containment Ventilation Units Containment Isolatic (Outside)	on ≤ 10
3/4 6		VI-77B	Instrument Air Containment Outside Isolation	≤ 10
6-35		SM-1 #	Main Steam 2D Isolation	NA
•		SM-3 #	Main Steam 2C Isolation	NA
		SM-5 #	Main Steam 2B Isolation	NA
		SM-7 #	Main Steam 2A Isolation	NA
		SM-9 #	Main Steam 2D Isolation Bypass Ctrl.	NA
<b>⊳</b> >>		SM-10 #	Main Steam 2C Isolation Bypass Ctrl.	NA
m me		SM-11 #	Main Steam 2B Isolation Bypass Ctrl.	NA
Amendment		SM-12 #	Main Steam 2A Isolation Bypass Ctrl.	NA
n in		SV-19 #	Main Steam 2A PORV	NA
z		SV-13 #	Main Steam 2B PORV	NA
No. 139		SV-7 #	Main Steam 2C PORV	NA
$\frac{1}{\omega} \frac{1}{\omega}$		SV-1 #	Main Steam 2D PORV	NA
သဏ်		WL-867A**	Containment Vent Unit Drains Inside Containment Isolation	≤ 10
<u> </u>		WL-869B**	Containment Vent Unit Drains Outside Containment Isolation	≤ 10



WASHINGTON, D.C. 20555-0001

# SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION RELATED TO AMENDMENT NO. 139 TO FACILITY OPERATING LICENSE NPF-35 AND AMENDMENT NO. 133 TO FACILITY OPERATING LICENSE NPF-52

DUKE POWER COMPANY, ET AL.

#### CATAWBA NUCLEAR STATION, UNITS 1 AND 2

DOCKET NOS. 50-413 AND 50-414

#### 1.0 INTRODUCTION

By letter dated October 31, 1994, Duke Power Company, et al. (the licensee), submitted a request for changes to the Catawba Nuclear Station, Units 1 and 2, Technical Specifications (TS). The requested changes would remove the steam generator power operated relief valve (PORV) stroke times from TS Tables 3.6-2a and 3.6-2b.

#### 2.0 EVALUATION

The proposed changes would revise the lists of containment isolation valves by changing the maximum opening time limitation from "<5" seconds to "NA" for the valves SV-19, SV-13, SV-7, and SV-1.

The licensee's technical justification for the changes is based on the following: (a) the valves' protective action functions are initiated by signals other than containment isolation signals, as identified in Final Safety Analysis Report (FSAR) Table 3.104, Note 8, and (b) credit for the operation of these valves is not taken in the dose analyses.

The function of the above valves is to prevent lifting of the safety valves during mild pressure transients, assist reseating actuated safety valves, provide a means for plant cooldown when the steam dump system is not operable, and to provide a safety-grade means of achieving reactor coolant system (RCS) cooldown to decay heat removal (ND) system initiation temperature in compliance with Branch Technical Position (BTP) RSB 5-1.

Since the steam and feedwater isolation functions are safety-related functions for which the protective actions must be completed within an assumed time period, it is appropriate that the TS include operability criteria which ensure that the protective actions can be accomplished within time limits assumed in the FSAR analyses. Such assurance is provided elsewhere in the TS in Table 3.3-5 "Engineered Safety Features Response Times" which specifies, for both feedwater and steamline isolation features, maximum protective action time limits that identify the maximum acceptable total time allotted for an entire protective action, encompassing the time from which a monitored parameter exceeds its setpoint, to the time at which the actuated valve is closed (Reference TS Section 1.13 "Definition of Engineered Safety Features

Response Time"). Since the requirements established by TS Table 3.3-5 ensure the timely actuation of the valves listed above, and the subject valves serve no other safety function, deletion of the closure time specification from TS isolation valve Tables 3.6-2a and 3.6-2b is acceptable.

Based on the results of the staff's review, it is concluded that the proposed TS changes are acceptable.

#### 3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the South Carolina State official was notified of the proposed issuance of the amendments. The State official had no comments.

#### 4.0 ENVIRONMENTAL CONSIDERATION

The amendments change requirements with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration, and there has been no public comment on such finding (60 FR 8745, dated February 15, 1995). Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

#### 5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: R. E. Martin

Date: December 18, 1995