January 20, 1995

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Mr. David L. Rehn
Vice President, Catawba Site
Duke Power Company
4800 Concord Road
York, SC 29745

SUBJECT: ISSUANCE OF AMENDMENTS - 4 KV BUS UNDERVOLTAGE TRIP SETPOINT AND ALLOWABLE VALUE CHANGES, CATAWBA NUCLEAR STATION, UNITS 1 AND 2 (TAC NOS. M90333 AND M90334)

Dear Mr. Rehn:

The Nuclear Regulatory Commission has issued the enclosed Amendment No. 127 to Facility Operating License NPF-35 and Amendment No. 121 to Facility Operating License NPF-52 for the Catawba Nuclear Station, Units 1 and 2. The amendments consist of changes to the Technical Specifications (TS) in response to your application dated August 25, 1994, as supplemented on November 16, 1994.

The amendments revise TS Table 3.3-4, by revising the "Trip Setpoint" and "Allowable Value" for the 4 kV bus undervoltage grid degraded voltage relays and the "Allowable Value" for the 4 kV undervoltage loss-of-voltage/loss-of-offsite power relays. This revision was submitted in response to a concern identified by the licensee in their Self-Initiated Technical Audit and during the electrical distribution system functional inspection team findings.

A copy of the related Safety Evaluation is also enclosed. A Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

Original signed by:

Robert E. Martin, Senior Project Manager
Project Directorate II-3
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

250077

Docket Nos. 50-413 and 50-414

Enclosures:
1. Amendment No. 127 to NPF-35
2. Amendment No. 121 to NPF-52
3. Safety Evaluation

cc w/encl: See next page

DOCUMENT NAME: G:\CATAWBA\CAT90333.AMD

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

January 20, 1995

Mr. David L. Rehn
Vice President, Catawba Site
Duke Power Company
4800 Concord Road
York, SC 29745

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Robert E. Martin

Robert E. Martin, Senior Project Manager
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Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket Nos. 50-413 and 50-414

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1. Amendment No. 127 to NPF-35
2. Amendment No. 121 to NPF-52
3. Safety Evaluation

cc w/encl: See next page

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Catawba Nuclear Station

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

DUKE POWER COMPANY

NORTH CAROLINA ELECTRIC MEMBERSHIP CORPORATION

SALUDA RIVER ELECTRIC COOPERATIVE, INC.

DOCKET NO. 50-413

CATAWBA NUCLEAR STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 127
License No. NPF-35

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment to the Catawba Nuclear Station, Unit 1 (the facility) Facility Operating License No. NPF-35 filed by the Duke Power Company, acting for itself, North Carolina Electric Membership Corporation and Saluda River Electric Cooperative, Inc. (licensees), dated August 25, 1994, as supplemented November 16, 1994, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations as set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

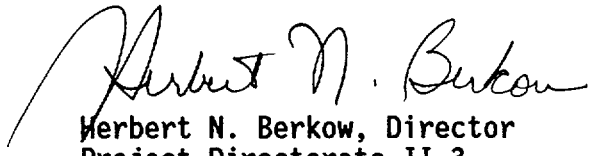
2. Accordingly, the license is hereby amended by page changes to the Technical Specifications as indicated in the attachment to this license amendment, and Paragraph 2.C.(2) of Facility Operating License No. NPF-35 is hereby amended to read as follows:

   Technical Specifications

   The Technical Specifications contained in Appendix A, as revised through Amendment No. 127 , and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, are hereby incorporated into this license. Duke Power Company shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Herbert N. Berkow, Director
Project Directorate II-3
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Technical Specification
Changes

Date of Issuance: January 20, 1995

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

DUKE POWER COMPANY

NORTH CAROLINA MUNICIPAL POWER AGENCY NO. 1

PIEDMONT MUNICIPAL POWER AGENCY

DOCKET NO. 50-414

CATAWBA NUCLEAR STATION, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 121
License No. NPF-52

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment to the Catawba Nuclear Station, Unit 2 (the facility) Facility Operating License No. NPF-52 filed by the Duke Power Company, acting for itself, North Carolina Municipal Power Agency No. 1 and Piedmont Municipal Power Agency (licensees), dated August 25, 1994, as supplemented November 16, 1994, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations as set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is hereby amended by page changes to the Technical Specifications as indicated in the attachment to this license amendment, and Paragraph 2.C.(2) of Facility Operating License No. NPF-52 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 121 , and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, are hereby incorporated into this license. Duke Power Company shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Herbert N. Berkow, Director
Project Directorate II-3
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Technical Specification
Changes

Date of Issuance: January 20, 1995

ATTACHMENT TO LICENSE AMENDMENT NO. 127

FACILITY OPERATING LICENSE NOS. NPF-35

DOCKET NO. 50-413

AND

TO LICENSE AMENDMENT NO. 121

FACILITY OPERATING LICENSE NO. NPF-52

DOCKET NO. 50-414

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the areas of change.

Remove Pages	Insert Pages
3/4 3-31	3/4 3-31
3/4 3-32	3/4 3-32
3/4 3-33	3/4 3-33
3/4 3-34	3/4 3-34

TABLE 3.3-4 (Continued)
ENGINEERED SAFETY FEATURES ACTUATION SYSTEM INSTRUMENTATION TRIP SETPOINTS

FUNCTIONAL UNIT	TRIP SETPOINT	ALLOWABLE VALUE
8. Auxiliary Feedwater (Continued)		
c. Steam Generator Water Level - Low-Low		
1) Unit 1	≥ 17% of span from 0% to 30% RTP increasing linearly to ≥ 40.0% of span from 30% to 100% RTP	≥ 15.3% of span from 0% to 30% RTP increasing linearly to ≥ 38.3% of span from 30% to 100% RTP
2) Unit 2	≥ 36.8% of narrow range span	≥ 35.1% of narrow range instrument span
d. Safety Injection	See Item 1. above for all Safety Injection Setpoints and Allowable Values.	
e. Loss-of-Offsite Power	≥ 3500 V	≥ 3242 V
f. Trip of All Main Feedwater Pumps	N.A.	N.A.
g. Auxiliary Feedwater Suction Pressure-Low		
1) CAPS 5220, 5221, 5222	≥ 10.5 psig	≥ 9.5 psig
2) CAPS 5230, 5231, 5232	≥ 6.2 psig	≥ 5.2 psig
a. Unit 1	≥ 6.2 psig	≥ 5.2 psig
b. Unit 2	≥ 6.0 psig	≥ 5.0 psig
9. Containment Sump Recirculation		
a. Automatic Actuation Logic and Actuation Relays	N.A.	N.A.
b. Refueling Water Storage Tank Level-Low Coincident With Safety Injection	≥ 177.15 inches	≥ 162.4 inches
	See Item 1. above for all Safety Injection Setpoints and Allowable Values.	

Amendment No. 127 (Unit 1)
Amendment No. 121 (Unit 2)

TABLE 3.3-4 (Continued)
ENGINEERED SAFETY FEATURES ACTUATION SYSTEM INSTRUMENTATION TRIP SETPOINTS

FUNCTIONAL UNIT	TRIP SETPOINT	ALLOWABLE VALUE
10. Loss of Power		
a. 4 kV Bus Undervoltage-Loss of Voltage	≥ 3500 V	≥ 3242 V
b. 4 kV Bus Undervoltage-Grid Degraded Voltage	≥ 3766 V	≥ 3738 V
11. Control Room Area Ventilation Operation		
a. Automatic Actuation Logic and Actuation Relays	N.A.	N.A.
b. Loss-of-Offsite Power	≥ 3500 V	≥ 3242 V
c. Safety Injection	See Item 1. above for all Safety Injection Setpoints and Allowable Values.	
12. Containment Air Return and Hydrogen Skimmer Operation		
a. Manual Initiation	N.A.	N.A.
b. Automatic Actuation Logic and Actuation Relays	N.A.	N.A.
c. Containment Pressure-High-High	≤ 3 psig	≤ 3.2 psig

Amendment No. 127 (Unit 1)
Amendment No. 121 (Unit 2)

TABLE 3.3-4 (Continued)

ENGINEERED SAFETY FEATURES ACTUATION SYSTEM INSTRUMENTATION TRIP SETPOINTS

FUNCTIONAL UNIT	TRIP SETPOINT	ALLOWABLE VALUE
13. Annulus Ventilation Operation		
a. Manual Initiation	N.A.	N.A.
b. Automatic Actuation Logic and Actuation Relays	N.A.	N.A.
c. Safety Injection	See Item 1. above for all Safety Injection Setpoints and Allowable Values.	
14. Nuclear Service Water Operation		
a. Manual Initiation	N.A.	N.A.
b. Automatic Actuation Logic and Actuation Relays	N.A.	N.A.
c. Loss-of-Offsite Power	≥ 3500 V	≥ 3242 V
d. Containment Spray	See Item 2. above for all Containment Spray Setpoints and Allowable Values.	
e. Phase "B" Isolation	See Item 3.b. above for all Phase "B" Isolation Setpoints and Allowable Values.	
f. Safety Injection	See Item 1. above for all Safety Injection Setpoints and Allowable Values.	
g. Suction Transfer-Low Pit Level	≥El. 554.4 ft.	≥El. 552.9 ft.
15. Emergency Diesel Generator Operation (Diesel Building Ventilation Operation, Nuclear Service Water Operation)		
a. Manual Initiation	N.A.	N.A.

Amendment No. 127 (Unit 1)
Amendment No. 121 (Unit 2)

TABLE 3.3-4 (Continued)

ENGINEERED SAFETY FEATURES ACTUATION SYSTEM INSTRUMENTATION TRIP SETPOINTS

FUNCTIONAL UNIT	TRIP SETPOINT	ALLOWABLE VALUE
15. Emergency Diesel Generator Operation (Diesel Building Ventilation Operation, Nuclear Service Water Operation) (Continued)		
b. Automatic Actuation Logic and Actuation Relays	N.A.	N.A.
c. Loss-of-Offsite Power	≥ 3500 V	≥ 3242 V
d. Safety Injection	See Item. 1 above for all Safety Injection Setpoints and Allowable Values.	
16. Auxiliary Building Filtered Exhaust Operation		
a. Manual Initiation	N.A.	N.A.
b. Automatic Actuation Logic and Actuation Relays	N.A.	N.A.
c. Safety Injection	See Item 1. above for all Safety Injection Setpoints and Allowable Valu	
17. Diesel Building Ventilation Operation		
a. Manual Initiation	N.A.	N.A.
b. Automatic Actuation Logic and Actuation Relays	N.A.	N.A.
c. Emergency Diesel Generator Operation	See Item 15. above for all Emergency Diesel Generator Operation Setpoints and Allowable Values.	

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 127 TO FACILITY OPERATING LICENSE NPF-35

AND AMENDMENT NO. 121 TO FACILITY OPERATING LICENSE NPF-52

DUKE POWER COMPANY, ET AL.

CATAWBA NUCLEAR STATION, UNITS 1 AND 2

DOCKET NOS. 50-413 AND 50-414

1.0 INTRODUCTION

By letter dated August 25, 1994, as supplemented November 16, 1994, Duke Power Company, et al. (the licensee), submitted a request for changes to the Catawba Nuclear Station, Units 1 and 2, Technical Specifications (TS). The requested changes would change TS Table 3.3-4, "Engineered Safety Features Actuation System Instrumentation Trip Setpoints" by revising the "Trip Setpoint" and "Allowable Value" for the 4 kV bus undervoltage grid degraded voltage relays and the "Allowable Value" for the 4 kV undervoltage loss of voltage/loss-of-offsite power relays. This revision was submitted in response to a concern identified by the licensee in their Self-Initiated Technical Audit (SITA) and during the electrical distribution system functional inspection (EDSFI) team findings. The November 16, 1994, letter provided clarifying information that did not change the scope of the August 25, 1994, application and the initial proposed no significant hazards consideration determination.

The two trains of the 4160 volt ac essential auxiliary power system are each provided with two levels of undervoltage protection. The first level detects a loss of voltage. The relays will drop out if the voltage falls below 3500 volts for more than 10 cycles. This 3500 volt setpoint, identified as "Loss of Offsite Power" or "Loss of Power/4KV Bus Undervoltage-Loss of Voltage," appears in five functional units in Table 3.3-4. It did not require modification for the reasons discussed below. However, the allowable value for this parameter did require a change from 3200 volts to 3242 volts. The second level of protection provides degraded voltage detection and alarms and drops out after established time delays. This parameter, identified as "Loss of Power/4KV Bus Undervoltage-Grid Degraded Voltage," appears once in Table 3.3-4. Its setpoint was changed from 3685 to 3766 volts and its allowable value was changed from 3611 to 3738 volts.

As noted above, the "Trip Setpoint" for the 4 kV loss-of-voltage/loss-of-offsite power does not require any change. This is because the existing TS trip setpoint is already at a more conservative value than required.

The licensee's original calculation for establishing these parameters did not account for all variables in determining the relay settings. The consideration of those variables is required in order to determine the

appropriate relay Trip Setpoint and Allowable Value. The new calculation includes allowance for these variables. The proposed revision is a result of the licensee's SITA and the EDSFI team evaluation.

2.0 EVALUATION

The staff has evaluated the proposed revisions submitted by the licensee as follows:

Revision to Existing TS Table 3.3-4

The licensee has proposed to revise the "Engineered Safety Features Actuation Instrumentation Trip Setpoints," TS Table 3.3-4 for the 4 kV essential power system which supplies power to those Class 1E loads required to safely shut down the unit following a design basis accident. The system is divided into two completely redundant and independent trains. Each redundant 4 kV essential bus is provided with two levels of undervoltage protection to monitor bus voltage. Each level is provided with a separate set of three undervoltage relays which are utilized in a two-out-of-three logic scheme. The proposed changes are as follows:

Item 8.e: Loss of Offsite Power: Under "ALLOWABLE VALUE" change "≥ 3200" volts to "≥ 3242" volts.

Item 10.a: 4 kV Bus Undervoltage Loss of Voltage: Under "ALLOWABLE VALUE" change "≥ 3200" volts to "≥ 3242" volts.

Item 10.b: 4 kV Undervoltage Grid Degraded Voltage: Under "TRIP SETPOINT" change "≥ 3685" volts to "≥ 3766" volts, and under "ALLOWABLE VALUE" change "≥ 3611" volts to "≥ 3738" volts.

Item 11.b: Loss of Offsite Power: Under "ALLOWABLE VALUE" change "≥ 3200" volts to "≥ 3242" volts.

Item 14.c: Loss of Offsite Power: Under "ALLOWABLE VALUE" change "≥ 3200" volts to "≥ 3242" volts.

Item 15.c: Loss of Offsite Power: Under "ALLOWABLE VALUE" change "≥ 3200" volts to "≥ 3242" volts.

The first level of undervoltage relays detects a loss of voltage on the 4 kV essential bus. The TS trip setpoints for the loss of voltage/loss of offsite power do not require any changes because the existing TS trip setpoints are already at a more conservative value than required by the calculation. The allowable value for the loss-of-voltage/loss-of-offsite power was revised to ≥ 3242 volts to account for instrument errors which were not considered in the licensee's original calculation.

The second level of undervoltage protection provides degraded voltage protection. The EDSFI team evaluation identified that the reset-to-dropout ratio of the degraded voltage relays was high, approximately 105 percent. This wide reset band could allow unnecessary actuation of the undervoltage protection system. In response to the EDSFI team concerns, the licensee replaced the degraded voltage relays with a model that has a narrower tolerance band and better pickup-to-dropout ratio. Also, the licensee's original calculation was non-conservative in that the voltage drop analysis only went down to the 4 kV level rather than the 120 volt level. Also, the calculation did not account for all possible errors in determining the relay settings. The allowance for error in the calculation is required in order to determine the minimum allowable value and the relay trip setpoint. The new calculation includes all possible errors, tolerances, and drift considerations in the determination of the trip setpoint of the relay. The licensee also did a new voltage drop analysis that went down to the 120 volt level on all essential buses. The calculation specifies a trip setpoint of $\geq$ 3766 volts (approximately 90.5% of nominal bus voltage) with a revised allowable value of $\geq$ 3738 volts (89.9% of nominal bus voltage).

Branch Technical Position (BTP) PSB-1, "Adequacy of Station Electrical Distribution System Voltages," provided the criteria for determining adequate degraded voltage protection. The licensee conducted an engineering review and reanalysis of the Electrical Distribution System, which involved adequately addressing the EDSFI results and the requirements of BTP PSB-1, in order to establish the technical specification trip setpoint and allowable value for grid degraded (undervoltage) relays. The proposed revision does not change the design, function, or method of operation of Class 1E equipment at Catawba, Units 1 and 2.

This proposed TS modification requires no change to the actual field settings because the current settings of the affected relays are in conformance with the proposed values.

Based on the above evaluation, the staff finds the proposed changes to Catawba, Units 1 and 2, TS to be acceptable.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the South Carolina State official was notified of the proposed issuance of the amendments. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

The amendments change requirements with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the

amendments involve no significant hazards consideration, and there has been no public comment on such finding (59 FR 51619 dated October 12, 1994). Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: Charles R. Thomas

Date: January 20, 1995