

March 30, 1993

Docket Nos. 50-413  
and 50-414

Distribution  
See next page

Mr. M. S. Tuckman  
Vice President, Catawba Site  
Duke Power Company  
4800 Concord Road  
York, South Carolina 29745

Dear Mr. Tuckman:

SUBJECT: ISSUANCE OF AMENDMENTS - CATAWBA NUCLEAR STATION, UNITS 1 AND 2  
(TACS M76961 AND M76962)

The Nuclear Regulatory Commission has issued the enclosed Amendment No. 108 to Facility Operating License NPF-35 and Amendment No. 102 to Facility Operating License NPF-52 for the Catawba Nuclear Station, Units 1 and 2. The amendments consist of changes to the Technical Specifications (TSs) in response to your application dated June 7, 1990, as supplemented April 22 and November 4, 1992.

The amendments revise TS 3.7.5b. for the Standby Nuclear Service Water Pond to require an average water temperature less than or equal to 91.5 °F at 568 feet elevation. The TS Bases are also revised to reflect this change.

A copy of the related Safety Evaluation is also enclosed. A Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

/s/

Robert E. Martin, Senior Project Manager  
Project Directorate II-3  
Division of Reactor Projects I/II  
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 108 to NPF-35
2. Amendment No. 102 to NPF-52
3. Safety Evaluation

cc w/enclosures:  
See next page

OFC : PDII-3 LA : PDII-3/PM : OGC : DSSA/SPLB : D: PDII-3 :  
NAME : LBER : RMARTIN : E. Hollen : G. McCracken : D. Matthews :  
DATE : 3/4/93 : 3/4/93 : 3/12/93 : 3/15/93 : 3/23/93 :  
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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555  
March 30, 1993

Docket Nos. 50-413  
and 50-414

Mr. M. S. Tuckman  
Vice President, Catawba Site  
Duke Power Company  
4800 Concord Road  
York, South Carolina 29745

Dear Mr. Tuckman:

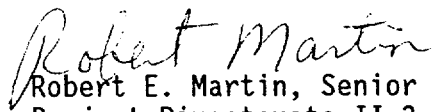
SUBJECT: ISSUANCE OF AMENDMENTS - CATAWBA NUCLEAR STATION, UNITS 1 AND 2  
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3. Safety Evaluation

cc w/enclosures:  
See next page

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Duke Power Company

Catawba Nuclear Station

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DATED: March 30, 1993

AMENDMENT NO. 108 TO FACILITY OPERATING LICENSE NPF-35 - Catawba Nuclear  
Station, Unit 1

AMENDMENT NO. 102 TO FACILITY OPERATING LICENSE NPF-52 - Catawba Nuclear  
Station, Unit 2

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R. Martin 14-H-25

OGC-WF 15-B-18

D. Hagan MNBB 4702

G. Hill (4) P1-37

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ACRS (10) P-135

GPA/PA 17-F-2

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

DUKE POWER COMPANY

NORTH CAROLINA ELECTRIC MEMBERSHIP CORPORATION

SALUDA RIVER ELECTRIC COOPERATIVE, INC.

DOCKET NO. 50-413

CATAWBA NUCLEAR STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 108  
License No. NPF-35

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment to the Catawba Nuclear Station, Unit 1 (the facility) Facility Operating License No. NPF-35 filed by the Duke Power Company, acting for itself, North Carolina Electric Membership Corporation and Saluda River Electric Cooperative, Inc. (licensees) dated June 7, 1990, as supplemented April 22 and November 4, 1992, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations as set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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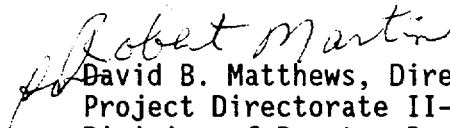
2. Accordingly, the license is hereby amended by page changes to the Technical Specifications as indicated in the attachment to this license amendment, and Paragraph 2.C.(2) of Facility Operating License No. NPF-35 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 108, and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, are hereby incorporated into this license. Duke Power Company shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective within thirty (30) days of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

  
David B. Matthews, Director  
Project Directorate II-3  
Division of Reactor Projects-I/II  
Office of Nuclear Reactor Regulation

Attachment:  
Technical Specification  
Changes

Date of Issuance: March 30, 1993



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

DUKE POWER COMPANY

NORTH CAROLINA MUNICIPAL POWER AGENCY NO. 1

PIEDMONT MUNICIPAL POWER AGENCY

DOCKET NO. 50-414

CATAWBA NUCLEAR STATION, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 102  
License No. NPF-52

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment to the Catawba Nuclear Station, Unit 2 (the facility) Facility Operating License No. NPF-52 filed by the Duke Power Company, acting for itself, North Carolina Municipal Power Agency No. 1 and Piedmont Municipal Power Agency (licensees) dated June 7, 1990, as supplemented April 22 and November 4, 1992, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations as set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is hereby amended by page changes to the Technical Specifications as indicated in the attachment to this license amendment, and Paragraph 2.C.(2) of Facility Operating License No. NPF-52 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 102, and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, are hereby incorporated into this license. Duke Power Company shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective within thirty (30) days of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

*Robert Martin*  
for David B. Matthews, Director  
Project Directorate II-3  
Division of Reactor Projects-I/II  
Office of Nuclear Reactor Regulation

Attachment:  
Technical Specification  
Changes

Date of Issuance: March 30, 1993



ATTACHMENT TO LICENSE AMENDMENT NO. 108

FACILITY OPERATING LICENSE NO. NPF-35

DOCKET NO. 50-413

AND

TO LICENSE AMENDMENT NO. 102

FACILITY OPERATING LICENSE NO. NPF-52

DOCKET NO. 50-414

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the areas of change.

<u>Remove Pages</u>	<u>Insert Pages</u>
3/4 7-13	3/4 7-13
B 3/4 7-3a	B 3/4 7-3a
B 3/4 7-4*	B 3/4 7-4
B 3/4 7-4a*	B 3/4 7-4a

\* overflow pages containing no changes

## PLANT SYSTEMS

### 3/4.7.5 STANDBY NUCLEAR SERVICE WATER POND

#### LIMITING CONDITION FOR OPERATION

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3.7.5 The standby nuclear service water pond (SNSWP) shall be OPERABLE with:

- a. A minimum water level at or above elevation 570 feet Mean Sea Level, USGS datum, and
- b. An average water temperature of less than or equal to 91.5°F at elevation 568 feet in the SNSWP.

APPLICABILITY: MODES 1, 2, 3, and 4.

#### ACTION:

With the requirements of the above specification not satisfied, be in at least HOT STANDBY within 6 hours and in COLD SHUTDOWN within the following 30 hours.

#### SURVEILLANCE REQUIREMENTS

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4.7.5 The SNSWP shall be determined OPERABLE:

- a. At least once per 24 hours by verifying the water level to be within its limit,
- b. At least once per 24 hours during the months of July, August, and September by verifying the water temperature to be within its limit,
- c. At least once per 12 months by visually inspecting the SNSWP dam and verifying no abnormal degradation, erosion, or excessive seepage, and
- d. At least once per 24 hours during the months of July, August and September while the Nuclear Service Water System is aligned to Lake Wylie by recording the water temperature of Lake Wylie, as measured in the discharge path of an operating Nuclear Service Water pump.

## PLANT SYSTEMS

### BASES

#### 3/4.7.5 STANDBY NUCLEAR SERVICE WATER POND

The limitations on the standby nuclear service water pond (SNSWP) level and temperature ensure that sufficient cooling capacity is available to either: (1) provide normal cooldown of the facility, or (2) mitigate the effects of accident conditions within acceptable limits.

The limitations on minimum water level and maximum temperature are based on providing a 30-day cooling water supply to safety-related equipment without exceeding its design basis temperature and is consistent with the recommendations of Regulatory Guide 1.27, "Ultimate Heat Sink for Nuclear Plants," March 1974.

The peak containment pressure analysis assumes that the Nuclear Service Water (RN) flow to the Containment Spray and Component Cooling heat exchangers has a temperature of 92°F. To ensure that this condition is not exceeded, and to ensure that long term RN temperature does not exceed the 100°F design basis of RN components (including a 2.4°F margin described in Section 2.4.4.2 of the Catawba SER, Supplements 1 and 2) a TS limit of 91.5°F is conservatively observed for the SNSWP. This temperature is important in that it, in part, determines the capacity for energy removal from containment. The peak containment pressure occurs when energy addition to containment (core decay heat) is balanced by energy removal from these heat exchangers. This balance is reached far out in time, after the transition from injection to cold leg recirculation and after ice melt. Because of the effectiveness of the ice bed in condensing the steam which passes through it, containment pressure is insensitive to small variations in containment spray temperature prior to ice meltout. Long term equipment qualification of safety related components required to mitigate the accident is based on a continuous, maximum RN supply temperature of 100°F.

To ensure that the RN initial temperature assumptions in the peak containment pressure analysis are met, Lake Wylie temperature is also monitored. During periods of time while Lake Wylie temperature is greater than 92°F, the emergency procedure for transfer of ECCS flow paths to cold leg recirculation directs the operator to align at least one train of containment spray to be cooled by a loop of Nuclear Service Water which is aligned to the SNSWP. Swap-over to the SNSWP is required at 92°F rather than 91.5°F because Lake Wylie is not subject to subsequent heatup due to recirculation, as is the SNSWP; hence the 100°F design basis maximum temperature is not approached.

#### 3/4.7.6 CONTROL ROOM AREA VENTILATION SYSTEM

The OPERABILITY of the Control Room Area Ventilation System ensures that: (1) the ambient air temperature does not exceed the allowable temperature for continuous-duty rating for the equipment and instrumentation cooled by this system, and (2) the control room will remain habitable for operations personnel during and following all credible accident conditions. Operation of the system with the heaters operating to maintain low humidity using automatic control for at least 10 continuous hours in a 31-day period is sufficient to reduce the

3/4.7.6 CONTROL ROOM AREA VENTILATION SYSTEM (Continued)

buildup of moisture on the adsorbers and HEPA filters. The Control Room Area Ventilation System filter units have no bypass line. Either Control Room Area Ventilation System train must operate in the filtered mode continuously. When a train is in operation, its associated heater also runs continuously. The specified laboratory test method, namely, ASTM D3803-89, implies that heaters may be unavailable for controlling the relative humidity of the influent air entering the charcoal adsorber section to less than or equal to 70 percent. This is acceptable, since accident analysis with appropriate adsorber efficiencies for radioiodine in elemental and organic forms based on the above test shows that the control room radiation doses to be within the 10 CFR Part 50, Appendix A, GDC 19 limits during design basis LOCA conditions. However, specifications are included to ensure heater operability and corrective ACTIONS are identified to address the contingency of inoperable heaters; these are in place to increase the safety margin of the filters. The OPERABILITY of this system in conjunction with control room design provisions is based on limiting the radiation exposure to personnel occupying the control room to 5 rems or less whole body, or its equivalent GDC 19 limit. ANSI N510-1980 will be used as a procedural guide for surveillance testing.

The 18-month surveillance to verify a positive pressure of greater than 1/8 inch water gauge, with less than or equal to 4000 cfm of pressurization flow, is to be conducted using only one intake from outside air open. By testing the capability to pressurize the control room using each intake individually, the design basis which assumes reopening of the two intakes following isolation on chlorine, smoke or radiation, is tested.

3/4.7.7 AUXILIARY BUILDING FILTERED EXHAUST SYSTEM

The OPERABILITY of the Auxiliary Building Filtered Exhaust System ensures that radioactive materials leaking from the ECCS equipment within the auxiliary building following a LOCA are filtered prior to reaching the environment. Operation of the system with the heaters operating to maintain low humidity using automatic control for at least 10 continuous hours in a 31-day period is sufficient to reduce the buildup of moisture on the adsorbers and HEPA filters. The specified laboratory test method, namely, ASTM D3803-89, implies that heaters may be unavailable for controlling the relative humidity of the influent air entering the charcoal adsorber section to less than or equal to 70 percent. This is acceptable, since accident analysis with appropriate adsorber efficiencies for radioiodine in elemental and organic forms based on the above test shows that the site boundary radiation doses to be within the 10 CFR Part 100 limits during design basis LOCA conditions. However, specifications are included to ensure heater operability and corrective ACTIONS are identified to address the contingency of inoperable heaters; these are in place to increase the safety margin of the filters. The operation of this system and the resultant effect on offsite dosage calculations was not taken credit for in the safety analyses. However, the operation of this system and the resultant effect on the NRC staff's offsite dose calculations was assumed in the staff's SER, NUREG-0954. ANSI N510-1980 will be used as a procedural guide for surveillance testing.

## PLANT SYSTEMS

### BASES

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#### 3/4.7.8 SNUBBERS

All snubbers are required OPERABLE to ensure that the structural integrity of the Reactor Coolant System and all other safety-related systems is maintained during and following a seismic or other event initiating dynamic loads.

Snubbers are classified and grouped by design and manufacturer but not by size. For example, mechanical snubbers utilizing the same design features of the 2-kip, 10-kip, and 100-kip capacity manufactured by Company "A" are of the same type. The same design mechanical snubbers manufactured by Company "B" for the purposes of this Technical Specification would be of a different type, as would hydraulic snubbers from either manufacturer.

A list of individual snubbers with detailed information of snubber location and size and of system affected shall be available at the plant in accordance with Section 50.71(c) of 10 CFR Part 50. The accessibility of each snubber shall be determined and approved by the Catawba Safety Review Group. The determination shall be based upon the existing radiation levels and the expected time to perform a visual inspection in each snubber location as well as other factors associated with accessibility during plant operations (e.g., temperature, atmosphere, location, etc.) and the recommendations of Regulatory Guides 8.8 and 8.10. The addition or deletions of any hydraulic or mechanical snubber shall be made in accordance with Section 50.59 of 10 CFR Part 50.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
RELATED TO AMENDMENT NO. 108 TO FACILITY OPERATING LICENSE NPF-35  
AND AMENDMENT NO. 102 TO FACILITY OPERATING LICENSE NPF-52  
DUKE POWER COMPANY, ET AL.  
CATAWBA NUCLEAR STATION, UNITS 1 AND 2  
DOCKET NOS. 50-413 AND 50-414

1.0 INTRODUCTION

By letter dated June 7, 1990, as supplemented April 22 and November 4, 1992, the Duke Power Company, et al. (the licensee), submitted a request for changes to the Catawba Nuclear Station, Units 1 and 2, Technical Specifications (TS). The Standby Nuclear Service Water Pond (SNSWP) limiting condition for operation (LCO) would be changed to require an average water temperature less than or equal to 91.5 °F at the 568 foot elevation. The April 22 and November 4, 1992, letters provided clarifying information that did not change the initial proposed no significant hazards consideration determination.

The initial submittal of June 7, 1990, proposed that TS 3.7.5b. be revised to 92 °F at the 563 foot elevation. The staff noted from its earlier review of this issue in the Safety Evaluation Report (NUREG-0954) for issuance of the operating license, that there was a 2.4 °F difference in the NRC staff's calculation of pond temperature and the licensee's calculation. The staff discussed this difference with the licensee and, consequently, the licensee revised its proposed TS 3.7.5b. in its April 22, 1992, submittal to a value of 91.5 °F at 568 feet elevation. This results in a maximum licensee calculated temperature of 97.5 °F, which when combined with the 2.4 °F difference, is less than 100 °F which is the qualification temperature for operation of the Nuclear Service Water (RN) System in this mode.

The proposed change reflects a more conservative approach to ensuring that the SNSWP can meet its design basis safety function, which is to provide an adequate source of cooling water to dissipate waste heat rejected during a unit LOCA plus a unit cooldown. The previous TS required that the temperature for the SNSWP, which serves as the ultimate heat sink in the event of the loss of Lake Wylie, be monitored at the intake pipe level (540 feet). This is not conservative because that monitoring point is near the bottom of the SNSWP, where cooler water exists. There is the opportunity for warmer, less dense water to exist at levels above the monitoring point. It is assumed in the Final Safety Analysis Report (FSAR) analysis that the entire pond is at the same initial temperature as measured at the indicated level in the pond with

no stratification. Therefore, the licensee concludes that it is not conservative for the LCO to specify a maximum allowed SNSWP temperature which is measured at the intake level.

## 2.0 EVALUATION

The proposed SNSWP change, as revised, requires that the average water temperature be less than or equal to 91.5 °F at an elevation of 568 feet in the SNSWP. This reflects a change from 86.5 °F at an elevation of 540 feet, and offers a more conservative approach in ensuring that the SNSWP can perform its design basis function as the ultimate heat sink, when required.

The licensee performed an analysis to qualify the SNSWP at a more conservative (91.5 °F) initial temperature and evaluate its impact on station structures, systems, and equipment. Preliminary steps in the analysis included verifying that the FSAR and SNSWP input parameters (i.e., area, volume) remained valid; and performing temperature surveys during summer months to come up with a reasonably conservative SNSWP temperature. Westinghouse then performed a containment analysis to determine the effects of the higher water temperature on peak containment pressure during a LOCA. Results of this analysis were reported in the initial June 7, 1990, application based on the 92 °F limit then proposed. Two input parameters--the initial SNSWP temperature and the assumed temperature of the Refueling Water Storage Tank (RWST)--were changed to reflect this proposed change and to more closely represent the TS requirements for the RWST temperature. The resulting peak analyzed pressure was 14.05 psig (which is below the assumed containment peak design pressure of 14.7 psig). This was reached 12.5 hours from the start of the LOCA sequence. The licensee stated that the water temperature is conservatively assumed to remain at 92 °F for the first 12.5 hours and that the volume of water needed to supply the RN system for those 12.5 hours is available below the 557 feet elevation. This demand volume determination was revised in the April 22, 1992, submittal to be consistent with the FSAR 9.2.5 methodology in which two trains of essential components are assumed to be shut down after 4 hours.

The licensee stated that, following the initial turnover of the SNSWP volume through the RN system assuming an initial unstratified temperature of 91.5 °F, the predicted peak SNSWP temperature is 97.5 °F. When the 2.4 °F difference between the DPC analytical model results and the NRC staff's independent analysis is added to this, the predicted maximum pond temperature remains at or below the 100 °F limit. This is the SNSWP temperature for which safety-related components were designed and assured makeup demands were based. Records show that the highest surface water temperatures in the Catawba SNSWP observed under warm weather conditions were below the value of the revised TS limit.

The licensee also stated that test acceptance criteria have been implemented for the major RN system heat exchangers to ensure that the actual LOCA heat rejection rates used in the Westinghouse containment peak pressure analysis can be attained with margin for fouling. Equipment qualification analyses for long-term containment temperatures have been revised to account for 92 °F initial SNSWP temperature followed by 100 °F RN cooling water temperature.

Based on its review of the licensee's proposed TS changes related to the SNSWP, the staff concludes that the licensee's analysis adequately demonstrates that the SNSWP will continue to meet its design basis safety function and satisfy the requirements of General Design Criterion 44. The staff, therefore, finds the proposed TS changes acceptable.

### 3.0 CONSULTATION

In accordance with the Commission's regulations, the South Carolina State official was notified of the proposed issuance of the amendments. The State official had no comments.

### 4.0 ENVIRONMENTAL CONSIDERATION

The amendments change requirements with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and change surveillance requirements. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration, and there has been no public comment on such finding (55 FR 28474 dated July 11, 1990). Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

### 5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributors: R. Martin  
D. Roberts

Date: March 30, 1993