

April 2, 1993

Docket Nos. 50-413
and 50-414

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Mr. M. S. Tuckman
Vice President, Catawba Site
Duke Power Company
4800 Concord Road
York, South Carolina 29745

Dear Mr. Tuckman:

SUBJECT: CORRECTION TO CATAWBA LICENSE AMENDMENTS 107 AND 101

The Nuclear Regulatory Commission issued Amendments 107 and 101 to Facility Operating Licenses NPF-35 and NPF-52 for Catawba Nuclear Station, Units 1 and 2, Technical Specifications (TS), dated March 23, 1993.

Due to an earlier TS Amendment (Amendments 103 and 97), a majority of the TS pages were deleted which changed the page numbering. When the amendment application was submitted for Nos. 107 and 101, page 6-19b was included which had been previously changed to page number 6-19a as a result of Amendments 103 and 97.

A revised TS page 6-19a is enclosed reflecting the correct page number for Amendments 107 and 101. No other changes were made.

We apologize for any inconvenience this may have caused.

Sincerely,

/s/

Robert E. Martin, Senior Project Manager
Project Directorate II-3
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosure:
TS page 6-19a

cc w/enclosure:
See next page

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UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

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Vice President, Catawba Site
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4800 Concord Road
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A revised TS page 6-19a is enclosed reflecting the correct page number for Amendments 107 and 101. No other changes were made.

We apologize for any inconvenience this may have caused.

Sincerely,

A handwritten signature in cursive script that reads "Robert E. Martin".

Robert E. Martin, Senior Project Manager
Project Directorate II-3
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosure:
TS page 6-19a

cc w/enclosure:
See next page

Mr. M. S. Tuckman
Duke Power Company

Catawba Nuclear Station

cc:

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ADMINISTRATIVE CONTROLS

CORE OPERATING LIMITS REPORT (Continued)

9. DPC-NE-3000P-A, Rev. 1, "Thermal-Hydraulic Transient Analysis Methodology," November 1991.

(Modeling used in the system thermal-hydraulic analyses)

10. DPC-NE-1004A, "Nuclear Design Methodology Using CASMO-3/SIMULATE-3P," November 1992.

(Methodology for Specification 3.1.1.3 - Moderator Temperature Coefficient.)

The core operating limits shall be determined so that all applicable limits (e.g., fuel thermal-mechanical limits, core thermal-hydraulic limits, ECCS limits, nuclear limits such as shutdown margin, and transient and accident analysis limits) of the safety analysis are met.

The CORE OPERATING LIMITS REPORT, including any mid-cycle revisions or supplements thereto, shall be provided upon issuance, for each reload cycle, to the NRC in accordance with 10 CFR 50.4.