

Docket Nos. 50-413  
and 50-414

Distribution  
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Mr. M. S. Tuckman  
Vice President -  
Nuclear Operations  
Duke Power Company  
P. O. Box 1007  
Charlotte, North Carolina 28201-1007

Dear Mr. Tuckman:

SUBJECT: ISSUANCE OF AMENDMENT NO. 91 TO FACILITY OPERATING LICENSE NPF-35  
AND AMENDMENT NO. 85 TO FACILITY OPERATING LICENSE NPF-52 -  
CATAWBA NUCLEAR STATION, UNITS 1 AND 2 (TACS 66667 AND 66668)

The Nuclear Regulatory Commission has issued the enclosed Amendment No. 91 to Facility Operating License NPF-35 and Amendment No. 85 to Facility Operating License NPF-52 for the Catawba Nuclear Station, Units 1 and 2. The amendments consist of changes to the Technical Specifications (TSs) in response to your application dated October 28, 1987, as supplemented June 18, 1991.

The amendments revise the TSs with four changes that are editorial in nature, corrections of typographical errors, and clarifications. An amendment issued on November 29, 1990, dealt with 13 of the 18 issues in the October 28, 1987 application. One item, number (4) TS 3.4.4 will be dealt with separately in response to another application dated May 9, 1991.

A copy of the related Safety Evaluation is also enclosed. Notice of Issuance of the amendments will be included in the Commission's biweekly Federal Register notice.

Sincerely,

/s/

Robert E. Martin, Project Manager  
Project Directorate II-3  
Division of Reactor Projects I/II  
Office of Nuclear Reactor Regulation

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PDR ADOCK 05000413  
P PDR

Enclosures:

- 1. Amendment No. 91 to NPF-35
- 2. Amendment No. 85 to NPF-52
- 3. Safety Evaluation

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OTS B  
E.T. Tomlinson  
10/9/91

OFC	: PDII-3/LA	: PDII-3/PM	: D: LHEB	: OGC	: PDII-3/A
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DATE	: 10/1/91	: 10/09/91	: 10/10/91	: 10/15/91	: 11/04/91

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555

November 4, 1991

Docket Nos. 50-413  
and 50-414

Mr. M. S. Tuckman  
Vice President -  
Nuclear Operations  
Duke Power Company  
P. O. Box 1007  
Charlotte, North Carolina 28201-1007

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SUBJECT: ISSUANCE OF AMENDMENT NO. 91 TO FACILITY OPERATING LICENSE NPF-35  
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A copy of the related Safety Evaluation is also enclosed. Notice of Issuance of the amendments will be included in the Commission's biweekly Federal Register notice.

Sincerely,

A handwritten signature in cursive script that reads "Robert E. Martin".

Robert E. Martin, Senior Project Manager  
Project Directorate II-3  
Division of Reactor Projects I/II  
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 91 to NPF-35
2. Amendment No. 85 to NPF-52
3. Safety Evaluation

cc w/enclosures:  
See next page

Mr. M. S. Tuckman  
Duke Power Company

Catawba Nuclear Station

cc:

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555

DUKE POWER COMPANY

NORTH CAROLINA ELECTRIC MEMBERSHIP CORPORATION

SALUDA RIVER ELECTRIC COOPERATIVE, INC.

DOCKET NO. 50-413

CATAWBA NUCLEAR STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 91  
License No. NPF-35

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment to the Catawba Nuclear Station, Unit 1 (the facility) Facility Operating License No. NPF-35 filed by the Duke Power Company, acting for itself, North Carolina Electric Membership Corporation and Saluda River Electric Cooperative, Inc. (licensees) dated October 28, 1987, as supplemented June 18, 1991, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations as set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is hereby amended by page changes to the Technical Specifications as indicated in the attachment to this license amendment, and Paragraph 2.C.(2) of Facility Operating License No. NPF-35 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 91 , and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, are hereby incorporated into this license. Duke Power Company shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



David B. Matthews, Director  
Project Directorate II-3  
Division of Reactor Projects-I/II  
Office of Nuclear Reactor Regulation

Attachment:  
Technical Specification  
Changes

Date of Issuance: November 4, 1991



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

DUKE POWER COMPANY

NORTH CAROLINA MUNICIPAL POWER AGENCY NO. 1

PIEDMONT MUNICIPAL POWER AGENCY

DOCKET NO. 50-414

CATAWBA NUCLEAR STATION, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 85  
License No. NPF-52

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment to the Catawba Nuclear Station, Unit 2 (the facility) Facility Operating License No. NPF-52 filed by the Duke Power Company, acting for itself, North Carolina Municipal Power Agency No. 1 and Piedmont Municipal Power Agency (licensees) dated October 28, 1987, as supplemented June 18, 1991, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations as set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

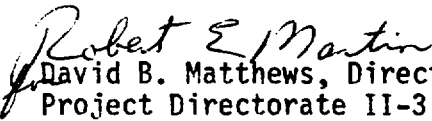
2. Accordingly, the license is hereby amended by page changes to the Technical Specifications as indicated in the attachment to this license amendment, and Paragraph 2.C.(2) of Facility Operating License No. NPF-52 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 85 , and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, are hereby incorporated into this license. Duke Power Company shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

  
for David B. Matthews, Director  
Project Directorate II-3  
Division of Reactor Projects-I/II  
Office of Nuclear Reactor Regulation

Attachment:  
Technical Specification  
Changes

Date of Issuance: November 4, 1991

ATTACHMENT TO LICENSE AMENDMENT NO. 91

FACILITY OPERATING LICENSE NO. NPF-35

DOCKET NO. 50-413

AND

TO LICENSE AMENDMENT NO. 85

FACILITY OPERATING LICENSE NO. NPF-52

DOCKET NO. 50-414

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the areas of change.

Remove Pages

3/4 7-5  
3/4 7-9  
3/4 8-3  
3/4 8-4  
3/4 8-5  
3/4 8-6  
6-2

Insert Pages

3/4 7-5  
3/4 7-9  
3/4 8-3  
3/4 8-4  
3/4 8-5  
3/4 8-6  
6-2



## PLANT SYSTEMS

### SURVEILLANCE REQUIREMENTS (Continued)

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- 3) Verifying that each non-automatic valve in the flow path that is not locked, sealed, or otherwise secured in position is in its correct position;
  - 4) Verifying that each automatic valve in the flow path is in the fully open position whenever the Auxiliary Feedwater System is placed in automatic control or when above 10% RATED THERMAL POWER; and
  - 5) Verifying that the isolation valves in the auxiliary feedwater pump suction lines are open and that power is removed from the valve operators on Valves CA-2, CA-7A, CA-9B, and CA-11A and that the respective circuit breakers are padlocked.
- b. At least once per 18 months during shutdown by:
- 1) Verifying that each automatic valve in the flow path actuates to its correct position upon receipt of an Auxiliary Feedwater Actuation test signal,
  - 2) Verifying that each motor-driven auxiliary feedwater pump starts as designed automatically upon receipt of an Auxiliary Feedwater Actuation test signal,
  - 3) Verifying that the turbine-driven auxiliary feedwater pump steam supply valves open upon receipt of an Auxiliary Feedwater Actuation test signal,\*\* and
  - 4) Verifying that the valve in the suction line of each auxiliary feedwater pump from the Nuclear Service Water System automatically actuates to its full open position within less than or equal to 16 seconds\* on a Loss-of-Suction test signal.

4.7.1.2.2 An auxiliary feedwater flow path to each steam generator shall be demonstrated OPERABLE following each COLD SHUTDOWN of greater than 30 days prior to entering MODE 2 by verifying normal flowpath to each steam generator.

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\*Includes a time delay of up to 6 seconds.

\*\*This surveillance need not be performed on Unit 2 until prior to entering HOT STANDBY following the Unit 2 first refueling.

PLANT SYSTEMS

CONDENSATE STORAGE SYSTEM

LIMITING CONDITION FOR OPERATION

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3.7.1.5 The Condensate Storage System (CSS) (CA Condensate Storage Tank, Upper Surge and Condenser Hotwell) shall be OPERABLE with a contained water volume of at least 225,000 gallons of water.

APPLICABILITY: MODES 1, 2, and 3. (Unit 2)

ACTION:

With the CSS inoperable, within 4 hours either:

- a. Restore the CSS to OPERABLE status or be in at least HOT STANDBY within the next 6 hours and in HOT SHUTDOWN within the following 6 hours, or
- b. Verify the OPERABILITY of the standby nuclear service water pond as a backup supply to the auxiliary feedwater pumps and restore the CSS to OPERABLE status within 7 days or be in at least HOT STANDBY within the next 6 hours and in HOT SHUTDOWN within the following 6 hours.

SURVEILLANCE REQUIREMENTS

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4.7.1.5 The CSS shall be demonstrated OPERABLE at least once per 12 hours by verifying the contained water volume is within its limits.

## ELECTRICAL POWER SYSTEMS

### LIMITING CONDITION FOR OPERATION

the next 6 hours and in COLD SHUTDOWN within the following 30 hours. Restore both diesel generators to OPERABLE status within 72 hours from time of initial loss or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

- f. With a diesel generator operating at greater than 5750 kW, within 1 hour reduce the diesel generator output to less than or equal to 5750 kW.
- g. With the Cathodic Protection System inoperable, restore the System to OPERABLE status within 10 days or prepare and submit a Special Report pursuant to Specification 6.9.2 outlining the cause of the inoperability and the plans for restoring the System to OPERABLE.

### SURVEILLANCE REQUIREMENTS

4.8.1.1.1 Each of the above required independent circuits between the offsite transmission network and the Onsite Essential Auxiliary Power System shall be:

- a. Determined OPERABLE at least once per 7 days by verifying correct breaker alignments, indicated power availability, and
- b. Demonstrated OPERABLE at least once per 18 months by transferring (manually and automatically) unit power supply from the normal circuit to the alternate circuit.

4.8.1.1.2 Each diesel generator shall be demonstrated OPERABLE:

- a. In accordance with the frequency specified in Table 4.8-1 on a STAGGERED TEST BASIS\*\* by:
  - 1) Verifying the fuel level in the day tank,
  - 2) Verifying the fuel level in the fuel storage tank,
  - 3) Verifying the fuel transfer valve can be operated to allow fuel to be transferred from the storage system to the day tank,
  - 4) Verifying the diesel starts from ambient condition and accelerates to at least 441 rpm in less than or equal to 11 seconds.\* The generator voltage and frequency shall be  $4160 \pm 420$  volts and  $60 \pm 1.2$  Hz within 11 seconds after the start signal. The diesel generator shall be started for this test by using one of the following signals:
    - a) Manual, or
    - b) Simulated loss of offsite power by itself, or

\*The diesel generator start (11 sec.) from ambient conditions shall be performed at least once per 184 days in these surveillance tests. All other engine starts for the purpose of this surveillance testing may be preceded by an engine pre-lube period and/or other warmup procedures recommended by the manufacturer so that mechanical stress and wear on the diesel engine is minimized.

\*\*STAGGERED TEST BASIS is only applicable when both diesel generators are on the same test frequency.

## ELECTRICAL POWER SYSTEMS

### SURVEILLANCE REQUIREMENTS (Continued)

- c) Simulated loss of offsite power in conjunction with an ESF Actuation test signal, or
  - d) An ESF Actuation test signal by itself.
- 5) Verifying the generator is synchronized, loaded to greater than or equal to 5600 kW but less than or equal to 5750 kW in less than or equal to 60 seconds, and operates for at least 60 minutes, and
- 6) Verifying the diesel generator is aligned to provide standby power to the associated emergency busses.
- b. At least once per 31 days and after each operation of the diesel where the period of operation was greater than or equal to 1 hour by checking for and removing accumulated water from the day tank;
- c. At least once per 31 days by checking for and removing accumulated water from the fuel oil storage tanks;
- d. By verifying that the Cathodic Protection System is OPERABLE by verifying:
- 1) At least once per 60 days that cathodic protection rectifiers are OPERABLE and have been inspected in accordance with the manufacturer's inspection procedures, and
  - 2) At least once per 12 months that adequate protection from corrosion is provided in accordance with manufacturer's inspection procedures.
- e. By sampling new fuel oil in accordance with ASTM-D4057 prior to addition to storage tanks and:
- 1) By verifying in accordance with the tests specified in ASTM-D975-81 prior to addition to the storage tanks that the sample has:
    - a) An API Gravity of within 0.3 degrees at 60°F, or a specific gravity of within 0.0016 at 60/60°F, when compared to the supplier's certificate, or an absolute specific gravity at 60/60°F of greater than or equal to 0.83 but less than or equal to 0.89, or an API gravity of greater than or equal to 27 degrees but less than or equal to 39 degrees;
    - b) A kinematic viscosity at 40°C of greater than or equal to 1.9 centistokes, but less than or equal to 4.1 centistokes (alternatively, Saybolt viscosity, SUS at 100°F of greater than or equal to 32.6, but less than or equal to 40.1), if gravity was not determined by comparison with the supplier's certification;
    - c) A flash point equal to or greater than 125°F; and
    - d) A clear and bright appearance with proper color when tested in accordance with ASTM-D4176-82.
  - 2) By verifying within 30 days of obtaining the sample that the other properties specified in Table 1 of ASTM-D975-81 are met when tested in accordance with ASTM-D975-81 except that the

## ELECTRICAL POWER SYSTEMS

### SURVEILLANCE REQUIREMENTS (Continued)

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analysis for sulfur may be performed in accordance with ASTM-D1552-79 or ASTM-D2622-82.

- f. At least once every 31 days by obtaining a sample of fuel oil in accordance with ASTM-D2276-78, and verifying that total particulate contamination is less than 10 mg/liter when checked in accordance with ASTM-D2276-78, Method A;
- g. At least once per 18 months by:
  - 1) Subjecting the diesel to an inspection, during shutdown, in accordance with procedures prepared in conjunction with its manufacturer's recommendations for this class of standby service;\*\*\*
  - 2) Verifying the generator capability to reject a load of greater than or equal to 825 kW while maintaining voltage at  $4160 \pm 420$  volts and frequency at  $60 \pm 1.2$  Hz;
  - 3) Verifying the generator capability to reject a load of greater than or equal to 5600 kW but less than or equal to 5750 kW without tripping. The generator speed shall not exceed 500 rpm during and following the load rejection;
  - 4) Simulating a loss-of-offsite power by itself,\*\* during shutdown, and:
    - a) Verifying deenergization of the emergency busses and load shedding from the emergency busses, and
    - b) Verifying the diesel starts on the auto-start signal, energizes the emergency busses with permanently connected loads within 11 seconds, energizes the auto-connected emergency (accident) loads through the load sequencer and operates for greater than or equal to 5 minutes while its generator is loaded with the emergency loads. After energization, the steady-state voltage and frequency of the emergency busses shall be maintained at  $4160 \pm 420$  volts and  $60 \pm 1.2$  Hz during this test.
  - 5) Verifying that on an ESF Actuation test signal, without loss-of-offsite power, the diesel generator starts on the auto-start signal and operates on standby for greater than or equal to 5 minutes. The generator voltage and frequency shall be at  $4160 \pm 420$  volts and  $60 \pm 1.2$  Hz within 11 seconds after the auto-start signal; the steady-state generator voltage and frequency shall be maintained within these limits during this test;
  - 6) Simulating a loss-of-offsite power in conjunction with an ESF Actuation test signal, during shutdown, and

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\*\*This surveillance need not be performed until prior to entering HOT SHUTDOWN following the Unit 1 first refueling.

\*\*\*This surveillance need not be performed on Unit 2 until prior to entering HOT SHUTDOWN following the Unit 2 first refueling.

## ELECTRICAL POWER SYSTEMS

### SURVEILLANCE REQUIREMENTS (Continued)

- a) Verifying deenergization of the emergency busses and load shedding from the emergency busses;\*\*
  - b) Verifying the diesel starts on the auto-start signal, energizes the emergency busses with permanently connected loads within 11 seconds, energizes the auto-connected emergency (accident) loads through the load sequencer and operates for greater than or equal to 5 minutes while its generator is loaded with the emergency loads. After energization, the steady-state voltage and frequency of the emergency busses shall be maintained at  $4160 \pm 420$  volts and  $60 \pm 1.2$  Hz during this test;\*\* and
  - c) Verifying that all automatic diesel generator trips, except engine overspeed, low-low lube oil pressure, generator differential, and the 2 out of 3 voltage controlled overcurrent relay scheme, are automatically bypassed upon loss of voltage on the emergency bus concurrent with a Safety Injection Actuation signal.
- 7) Verifying, during shutdown, the diesel generator operates for at least 24 hours. The diesel generator shall be loaded to greater than or equal to 5600 kW but less than or equal to 5750 kW. The generator voltage and frequency shall be  $4160 \pm 420$  volts and  $60 \pm 1.2$  Hz within 11 seconds after the start signal; the steady-state generator voltage and frequency shall be maintained within these limits during this test.
  - 8) Verifying, during shutdown, that the auto-connected loads to each diesel generator do not exceed 5750 kW;\*\*
  - 9) Verifying, during shutdown, the diesel generator's capability to:\*\*
    - a) Synchronize with the offsite power source while the generator is loaded with its emergency loads upon a simulated restoration of offsite power,
    - b) Transfer its loads to the offsite power source, and
    - c) Be restored to its standby status.
  - 10) Verifying, during shutdown, that with the diesel generator operating in a test mode, connected to its bus, a simulated Safety Injection signal overrides the test mode by: (1) returning the diesel generator to standby operation, and (2) automatically energizing the emergency loads with offsite power;\*\*

\*\*This surveillance need not be performed until prior to entering HOT SHUTDOWN following the Unit 1 first refueling.

## ADMINISTRATIVE CONTROL

### UNIT STAFF (Continued)

- c. A Health Physics Technician\* shall be on site when fuel is in either reactor;
- d. All CORE ALTERATIONS shall be observed and directly supervised by either a licensed Senior Operator or licensed Senior Operator Limited to Fuel Handling who has no other concurrent responsibilities during this operation; and
- e. (Deleted)
- f. Administrative procedures shall be developed and implemented to limit the working hours of unit staff who perform safety-related functions (e.g., licensed Senior Operators, licensed Operators, health physicists, auxiliary operators, and key maintenance personnel).

Adequate shift coverage shall be maintained without routine heavy use of overtime. The objective shall be to have operating personnel work a normal 12-hour day with alternating 48-hour and 36-hour work weeks while the unit is operating. However, in the event that unforeseen problems require substantial amounts of overtime to be used, or during extended periods of shutdown for refueling, major maintenance, or major plant modification, on a temporary basis the following guidelines shall be followed:

- 1) An individual should not be permitted to work more than 16 hours straight, excluding shift turnover time.
- 2) An individual should not be permitted to work more than 16 hours in any 24-hour period, nor more than 24 hours in any 48-hour period, nor more than 72 hours in any 7-day period, all excluding shift turnover time.
- 3) A break of at least 8 hours should be allowed between work periods, including shift turnover time.
- 4) Except during extended shutdown periods, the use of overtime should be considered on an individual basis and not for the entire staff on a shift.

Any deviation from the above guidelines shall be authorized by the Station Manager or his designee, or higher levels of management, in accordance with established procedures and with documentation of the basis for granting the deviation. Controls shall be included in the procedures such that individual overtime shall be reviewed monthly by the Station Manager or his designee to assure that excessive hours have not been assigned. Routine deviation from the above guidelines is not authorized.

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\*The Health Physics Technician may be less than the minimum requirements for a period of time not to exceed 2 hours, in order to accommodate unexpected absence, provided immediate action is taken to fill the required positions.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
RELATED TO AMENDMENT NO. 91 TO FACILITY OPERATING LICENSE NPF-35  
AND AMENDMENT NO. 85 TO FACILITY OPERATING LICENSE NPF-52

DUKE POWER COMPANY, ET AL.

CATAWBA NUCLEAR STATION, UNITS 1 AND 2

DOCKET NOS. 50-413 AND 50-414

1.0 INTRODUCTION

By letter dated October 28, 1987, as supplemented June 18, 1991, the Duke Power Company, et al. (the licensee), submitted a request for changes to the Catawba Nuclear Station, Units 1 and 2, Technical Specifications (TS). The licensee's application dated October 28, 1987, proposed 18 changes of an administrative and clarifying nature to the TS. Thirteen of these changes were dealt with in Amendment Nos. 80 and 74 to Facility Operating License Nos. NPF-35 and NPF-56, respectively, dated November 29, 1990. One change, number (4), will be dealt with separately in response to your application dated May 9, 1991. Therefore, this amendment will deal with changes (5) TS 3.7.1.5, (6) TS 4.7.1.2.2, (8) TS 4.8.1.1.2a, and (11) TS 6.2.2.f.

2.0 EVALUATION

Change number (5) in the October 28, 1987 application would change the word "Demonstrate" in TS 3.7.1.5b. to "Verify." The licensee's basis for this change is that the term "verify" would provide a clarification that operability for this TS could be correctly determined through verification of water temperature, level, etc. The term "demonstrate" would imply that a physical action or test must be performed and this would have no physical meaning in determining the operability of the standby nuclear service water pond.

The NRC staff finds that usage of the term "verify" for this action statement would be consistent with similar usage elsewhere in the Catawba TS and with the Standard Technical Specifications and is acceptable.

Change number (6) would insert the word "path" into TS 4.7.1.2.2 to clarify that the purpose is to verify that the flowpath is not blocked and not to verify the design flow rate. The flowrates are verified by TS 4.7.1.2.1. This change is made to satisfy a licensee commitment which was accepted by the NRC staff as discussed in Section 10.4.9 of the Catawba Safety Evaluation Report (NUREG-0954). This change satisfies that commitment and is acceptable.

Change number (8) would add a footnote to TS 4.8.1.1.2a. that would clarify that a STAGGERED TEST BASIS for testing of the diesel generators (DGs) would be applicable only when both DGs are on the same test frequency. This would



recognize the situation wherein one DG, due to the number of failures experienced, could be on a different test frequency (once per 7 days) than the other DG (once per 31 days). In such a case a STAGGERED TEST BASIS requirement would have no meaning. In addition, the staff has determined that the intent of the term "STAGGERED TEST BASIS" is to ensure that two or more DG's are not tested simultaneously. Therefore, the proposed footnote to TS 4.8.1.1.2 is acceptable.

Change number (11) would modify TS 6.2.2f. by changing the language defining the length of a work week for operating personnel while the unit is operating from a "nominal 40-hour week" to a "normal 12-hour day with alternating 48-hour and 36-hour work weeks." The licensee states that the change more accurately reflects the 12-hour shifts which the operators currently work. The change is acceptable since the alternating 48- and 36-hour weeks are within the scope of the guidance provided in Generic Letter 82-12 for a nominal 40-hour work week and other limits, as recommended by GL 82-12, are maintained.

Part of this change, to increase the maximum number of hours that could be worked in a 48-hour period from 24 to 28, was withdrawn by the licensee's letter dated June 18, 1991. The withdrawal of this part is acceptable to the NRC staff.

### 3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the South Carolina State official was notified of the proposed issuance of the amendments. The State official had no comments.

### 4.0 ENVIRONMENTAL CONSIDERATION

The amendments change requirements with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and change surveillance requirements. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration, and there has been no public comment on such finding (54 FR 47601 and 56 FR 41579). Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

### 5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: R. Martin, PDII-3/NRR

Date: November 04, 1991

DATED: November 4, 1991

AMENDMENT NO. 91 TO FACILITY OPERATING LICENSE NPF-35 - Catawba Nuclear Station, Unit 1  
AMENDMENT NO. 85 TO FACILITY OPERATING LICENSE NPF-52 - Catawba Nuclear Station, Unit 2

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