

August 15, 1989

Docket Nos. 50-413
and 50-414

Mr. H. B. Tucker, Vice President
Nuclear Production Department
Duke Power Company
422 South Church Street
Charlotte, North Carolina 28242

Dear Mr. Tucker:

SUBJECT: ISSUANCE OF AMENDMENT NO. 68 TO FACILITY OPERATING LICENSE NPF-35
AND AMENDMENT NO. 62 TO FACILITY OPERATING LICENSE NPF-52 - CATAWBA
NUCLEAR STATION, UNITS 1 AND 2 (TACS 66751/66752)

The Nuclear Regulatory Commission has issued the enclosed Amendment No. 68 to Facility Operating License NPF-35 and Amendment No. 62 to Facility Operating License NPF-52 for the Catawba Nuclear Station, Units 1 and 2. These amendments consist of changes to the Technical Specifications (TS) in response to your application dated December 4, 1987, as supplemented December 7, 11, and 29, 1987, March 29, May 4 and 18, June 16, July 1, August 8 and 24, and December 15, 1988, and June 12 and 28, 1989.

The amendments modify the TS by adding requirements for the steam generator power operated relief valves.

A copy of the related safety evaluation supporting Amendment No. 68 to Facility Operating License NPF-35 and Amendment No. 62 to Facility Operating License NPF-52 is enclosed.

Notice of issuance of amendments will be included in the Commission's next bi-weekly Federal Register notice.

Sincerely,

Original signed by:

Kahtan N. Jabbour, Project Manager
Project Directorate II-3
Division of Reactor Projects I/II
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 68 to NPF-35
2. Amendment No. 62 to NPF-52
3. Safety Evaluation

cc w/enclosures:
See next page

OFFICIAL RECORD COPY
[CATAWBA AMEND 66751/66752]

LA:PDII-3

MRood

06/11/89

PM:PDII-3

KJabbour:sa

06/11/89

PDII-3/DRPI/II

DMatthews

~~06/11/89~~

08/4/89

8208230398 890815
PDR ADDCK 05000413
PDC

08/18

Mr. H. B. Tucker
Duke Power Company

Catawba Nuclear Station

cc:

A.V. Carr, Esq.
Duke Power Company
422 South Church-Street
Charlotte, North Carolina 28242

J. Michael McGarry, III, Esq.
Bishop, Cook, Purcell and Reynolds
1400 L Street, N.W.
Washington, D. C. 20005

North Carolina MPA-1
Suite 600
3100 Smoketree Ct.
P.O. Box 29513
Raleigh, North Carolina 27626-0513

Ms. S. S. Kilborn
Area Manager, Mid-South Area
ESSD Projects
Westinghouse Electric Corp.
MNC West Tower - Bay 239
P.O. Box 355
Pittsburgh, Pennsylvania 15230

County Manager of York County
York County Courthouse
York, South Carolina 29745

Richard P. Wilson, Esq.
Assistant Attorney General
S.C. Attorney General's Office
P.O. Box 11549
Columbia, South Carolina 29211

Piedmont Municipal Power Agency
100 Memorial Drive
Greer, South Carolina 29651

Mr. Alan R. Herdt, Chief
Project Branch #3
U.S. Nuclear Regulatory Commission
101 Marietta Street, NW, Suite 2900
Atlanta, Georgia 30323

North Carolina Electric Membership
Corp.
3400 Sumner Boulevard
P.O. Box 27306
Raleigh, North Carolina 27611

Saluda River Electric Cooperative,
Inc.
P.O. Box 929
Laurens, South Carolina 29360

Senior Resident Inspector
Route 2, Box 179N
York, South Carolina 29745

Regional Administrator, Region II
U.S. Nuclear Regulatory Commission
101 Marietta Street, NW, Suite 2900
Atlanta, Georgia 30323

Mr. Heyward G. Shealy, Chief
Bureau of Radiological Health
South Carolina Department of Health
and Environmental Control
2600 Bull Street
Columbia, South Carolina 29201

Ms. Karen E. Long
Assistant Attorney General
N.C. Department of Justice
P.O. Box 629
Raleigh, North Carolina 27602

Mr. Peter G. LeRoy
Nuclear Production Department
Duke Power Company
P.O. Box 33189
Charlotte, North Carolina 28241

DATED: August 15, 1989

AMENDMENT NO. 68 TO FACILITY OPERATING LICENSE NPF-35 - Catawba Nuclear Station, Unit 1
AMENDMENT NO. 62 TO FACILITY OPERATING LICENSE NPF-52 - Catawba Nuclear Station, Unit 2

DISTRIBUTION:

Docket File

NRC PDR

Local PDR

PD#II-3 R/F

Catawba R/F

S. Varga	14-E-4
G. Lainas	14-H-3
D. Matthews	14-H-25
M. Rood	14-H-25
K. Jabbour	14-H-25
OGC-WF	15-B-18
B. Grimes	9-A-2
E. Jordan	MNBB-3302
W. Jones	P-130A
T. Meek (8)	P1-137
ACRS (10)	P-135
GPA/PA	17-F-2
OC/LFMB	AR-2015
J. Calvo	13-D-18
D. Hagan	MNBB-3302
W. Hodges	8-E-23

DF01
1/1



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

DUKE POWER COMPANY

NORTH CAROLINA ELECTRIC MEMBERSHIP CORPORATION

SALUDA RIVER ELECTRIC COOPERATIVE, INC.

DOCKET NO. 50-413

CATAWBA NUCLEAR STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 68
License No. NPF-35

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment to the Catawba Nuclear Station, Unit 1 (the facility) Facility Operating License No. NPF-35 filed by the Duke Power Company acting for itself, North Carolina Electric Membership Corporation and Saluda River Electric Cooperative, Inc., (licensees) dated December 4, 1987, as supplemented December 7, 11, and 29, 1987, March 29, May 4 and 18, June 16, July 1, August 8 and 24, and December 15, 1988, and June 12 and 28, 1989, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations as set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

8908230403 890815
PDR ADOCK 05000413
P PDC

2. Accordingly, the license is hereby amended by page changes to the Technical Specifications as indicated in the attachments to this license amendment, and Paragraph 2.C.(2) of Facility Operating License No. NPF-35 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 68, are hereby incorporated into the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Original signed by:

David B. Matthews, Director
Project Directorate II-3
Division of Reactor Projects-I/II
Office of Nuclear Reactor Regulation

Attachment:
Technical Specification Changes

Date of Issuance: August 15, 1989

OFFICIAL RECORD COPY

LA:PDII-3
MRood
7/11/89

KWJ
PM:PDII-3
KJabbour:ls
7/11/89

mmh
DEST:SRXB
WHodges
7/13/89

pm
OGC *
SElunk D:PDII-3
7/31/89 *DMatthews*
08/4/89

* insert FR cite
and make sure no
public comments were
received



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

DUKE POWER COMPANY

NORTH CAROLINA MUNICIPAL POWER AGENCY NO. 1

PIEDMONT MUNICIPAL POWER AGENCY

DOCKET NO. 50-414

CATAWBA NUCLEAR STATION, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 62
License No. NPF-52

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment to the Catawba Nuclear Station, Unit 2 (the facility) Facility Operating License No. NPF-52 filed by the Duke Power Company acting for itself, North Carolina Municipal Power Agency No. 1 and Piedmont Municipal Power Agency, (licensees) dated December 4, 1987, as supplemented December 7, 11, and 29, 1987, March 29, May 4 and 18, June 16, July 1, August 8 and 24, and December 15, 1988, and June 12 and 28, 1989, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations as set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is hereby amended by page changes to the Technical Specifications as indicated in the attachments to this license amendment, and Paragraph 2.C.(2) of Facility Operating License No. NPF-52 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 62, are hereby incorporated into the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Original signed by:

David B. Matthews, Director
Project Directorate II-3
Division of Reactor Projects-I/II
Office of Nuclear Reactor Regulation

Attachment:
Technical Specification Changes

Date of Issuance: August 15, 1989

OFFICIAL RECORD COPY

LA:PDII-3
MRood
7/11/89

KWS
PM:PDII-3
KJabbour:
7/11/89

WHodges
DEST:SRXB
WHodges
7/13/89

OGC
SETurk
7/13/89

DMatthews
D:PDII-3
DMatthews
8/4/89

ATTACHMENT TO LICENSE AMENDMENT NO. 68

FACILITY OPERATING LICENSE NO. NPF-35

DOCKET NO. 50-413

AND

TO LICENSE AMENDMENT NO. 62

FACILITY OPERATING LICENSE NO. NPF-52

DOCKET NO. 50-414

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the areas of change. The corresponding overleaf pages are also provided to maintain document completeness.

Amended Page

Overleaf Page

3/4 7-9a (new page)
B 3/4 7-2
B 3/4 7-2a (new page)

3/4 7-9
3/4 7-10
B 3/4 7-1

PLANT SYSTEMS

CONDENSATE STORAGE SYSTEM

LIMITING CONDITION FOR OPERATION

3.7.1.5 The Condensate Storage System (CSS) (CA Condensate Storage Tank, Upper Surge and Condenser Hotwell) shall be OPERABLE with a contained water volume of at least 225,000 gallons of water.

APPLICABILITY: MODES 1, 2, and 3. (Unit 2)

ACTION:

With the CSS inoperable, within 4 hours either:

- a. Restore the CSS to OPERABLE status or be in at least HOT STANDBY within the next 6 hours and in HOT SHUTDOWN within the following 6 hours, or
- b. Demonstrate the OPERABILITY of the standby nuclear service water pond as a backup supply to the auxiliary feedwater pumps and restore the CSS to OPERABLE status within 7 days or be in at least HOT STANDBY within the next 6 hours and in HOT SHUTDOWN within the following 6 hours.

SURVEILLANCE REQUIREMENTS

4.7.1.5 The CSS shall be demonstrated OPERABLE at least once per 12 hours by verifying the contained water volume is within its limits.

PLANT SYSTEMS

STEAM GENERATOR POWER OPERATED RELIEF VALVES

LIMITING CONDITION FOR OPERATION

3.7.1.6 Three steam generator power-operated relief valves (PORVs) and associated remote manual controls, including the safety-related gas supply systems, shall be OPERABLE.

APPLICABILITY: MODES 1, 2, 3, and 4.*

ACTION:

- a. With one less than the required steam generator PORVs OPERABLE, restore the inoperable steam generator PORV to OPERABLE status within 7 days; or be in at least HOT STANDBY within the next 6 hours and in HOT SHUTDOWN within the following 6 hours and place the required Residual Heat Removal loop in operation for decay heat removal.
- b. With two less than the required steam generator PORVs OPERABLE, restore at least one of the inoperable steam generator PORVs to OPERABLE status within 72 hours or be in at least HOT STANDBY within the next 6 hours and in HOT SHUTDOWN within the following 6 hours and place the required Residual Heat Removal loop in operation for decay heat removal.

SURVEILLANCE REQUIREMENTS

4.7.1.6 Each steam generator PORV and associated remote manual controls including the safety-related gas supply systems shall be demonstrated OPERABLE:

- a. At least once per 24 hours by verifying that at least one of the two nitrogen bottles associated with each PORV has a pressure greater than or equal to 2100 psig, and
- b. At least once per 18 months and prior to startup following any refueling shutdown by verifying that all steam generator PORVs will operate through one cycle of full travel using remote manual controls and safety-related gas supply.

*When steam generators are being used for decay heat removal.

PLANT SYSTEMS

3/4.7.2 STEAM GENERATOR PRESSURE/TEMPERATURE LIMITATION

LIMITING CONDITION FOR OPERATION

3.7.2 The temperatures of both the reactor and secondary coolants in the steam generators shall be greater than 70°F when the pressure of either coolant in the steam generator is greater than 200 psig.

APPLICABILITY: At all times.

ACTION:

With the requirements of the above specification not satisfied:

- a. Reduce the steam generator pressure of the applicable side to less than or equal to 200 psig within 30 minutes, and
- b. Perform an engineering evaluation to determine the effect of the overpressurization on the structural integrity of the steam generator. Determine that the steam generator remains acceptable for continued operation prior to increasing its temperatures above 200°F.

SURVEILLANCE REQUIREMENTS

4.7.2 The pressure in each side of the steam generator shall be determined to be less than 200 psig at least once per hour when the temperature of either the reactor or secondary coolant is less than 70°F.

3/4.7 PLANT SYSTEMS

BASES

3/4.7.1 TURBINE CYCLE

3/4.7.1.1 SAFETY VALVES

The OPERABILITY of the main steam line Code safety valves ensures that the Secondary System pressure will be limited to within 110% (1304 psig) of its design pressure of 1185 psig during the most severe anticipated system operational transient. The maximum relieving capacity is associated with a Turbine trip from valve wide-open condition coincident with an assumed loss of condenser heat sink (i.e., no steam bypass to the condenser).

The specified valve lift settings and relieving capacities are in accordance with the requirements of Section III of the ASME Boiler and Pressure Code, 1971 Edition. The total relieving capacity for all valves on all of the steam lines is 16.85×10^6 lbs/h which is 105% of the total secondary steam flow of 16.05×10^6 lbs/h at 100% RATED THERMAL POWER. A minimum of two OPERABLE safety valves per steam generator ensures that sufficient relieving capacity is available for the allowable THERMAL POWER restriction in Table 3.7-1.

STARTUP and/or POWER OPERATION is allowable with safety valves inoperable within the limitations of the ACTION requirements on the basis of the reduction in Secondary Coolant System steam flow and THERMAL POWER required by the reduced Reactor trip settings of the Power Range Neutron Flux channels. The Reactor Trip Setpoint reductions are derived on the following bases:

For four loop operation

$$SP = \frac{(X) - (Y)(V)}{X} \times (109)$$

Where:

- SP = Reduced Reactor Trip Setpoint in percent of RATED THERMAL POWER,
- V = Maximum number of inoperable safety valves per steam line,
- 109 = Power Range Neutron Flux-High Trip Setpoint for four loop operation,
- X = Total relieving capacity of all safety valves per steam line in lbs/hour, and
- Y = Maximum relieving capacity of any one safety valve in lbs/hour

PLANT SYSTEMS

BASES

3/4.7.1.2 AUXILIARY FEEDWATER SYSTEM

The OPERABILITY of the Auxiliary Feedwater System ensures that the Reactor Coolant System can be cooled down to less than 350°F from normal operating conditions in the event of a feedwater line break accident with a worst case single active failure.

The Auxiliary Feedwater System is capable of delivering a total feedwater flow of at least 492 gpm at a pressure of 1210 psig to the entrance of at least two of the steam generators. This capacity is sufficient to ensure that adequate feedwater flow is available to remove decay heat and reduce the Reactor Coolant System temperature to less than 350°F when the Residual Heat Removal System may be placed into operation.

3/4.7.1.3 SPECIFIC ACTIVITY

The limitations on Secondary Coolant System specific activity ensure that the resultant offsite radiation dose will be limited to a small fraction of 10 CFR Part 100 dose guideline values in the event of a steam line rupture. This dose also includes the effects of a coincident 1 gpm reactor to secondary tube leak in the steam generator of the affected steam line. These values are consistent with the assumptions used in the safety analyses.

3/4.7.1.4 MAIN STEAM LINE ISOLATION VALVES

The OPERABILITY of the main steam line isolation valves ensures that no more than one steam generator will blow down in the event of a steam line rupture. This restriction is required to: (1) minimize the positive reactivity effects of the Reactor Coolant System cooldown associated with the blowdown, and (2) limit the pressure rise within containment in the event the steam line rupture occurs within containment. The OPERABILITY of the main steam isolation valves within the closure times of the Surveillance Requirements are consistent with the assumptions used in the safety analyses.

3/4.7.1.5 CONDENSATE STORAGE SYSTEM

The OPERABILITY of the Condensate Storage System with the minimum water volume ensures that sufficient water is available to maintain the Reactor Coolant system at HOT STANDBY conditions for 2 hours followed by approximately 5 hours cooldown with steam discharge to the atmosphere concurrent with total loss-of-offsite power. The contained water volume limit includes an allowance for water not usable because of tank discharge line location or other physical characteristics.

3/4.7.1.6 STEAM GENERATOR POWER OPERATED RELIEF VALVES

The Surveillance Requirement for the Main Steam power-operated relief valves (PORVs) nitrogen supplies ensures that the PORVs will be available to mitigate the consequences of a steam generator tube rupture accident

PLANT SYSTEMS

BASES

STEAM GENERATOR POWER OPERATED RELIEF VALVES (Continued)

concurrent with loss of offsite power. This assumes that the PORV on the ruptured steam generator is unavailable, and that the other two are used to cool the Reactor Coolant System inventory to less than the saturation temperature of the ruptured steam generator.

Concurrent with the requirement that a specific number of PORVs be OPERABLE is the requirement that the associated PORV block valves upstream be open or OPERABLE. Should an associated PORV block valve be closed and inoperable, the PORV downstream of that block valve should also be considered inoperable and the applicable ACTION statement shall be entered until such time that the block valve is opened or returned to OPERABLE status.

Additionally, if a PORV is inoperable and open, then the requirements of Technical Specification 3.6.3, Containment Isolation Valves, would apply in addition to Technical Specification 3.7.1.6.

3/4.7.2 STEAM GENERATOR PRESSURE/TEMPERATURE LIMITATION

The limitation on steam generator pressure and temperature ensures that the pressure-induced stresses in the steam generators do not exceed the maximum allowable fracture toughness stress limits. The limitations of 70°F and 200 psig are based on a steam generator RT_{NDT} of 60°F and are sufficient to prevent brittle fracture.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 68 TO FACILITY OPERATING LICENSE NPF-35
AND AMENDMENT NO. 62 TO FACILITY OPERATING LICENSE NPF-52

DUKE POWER COMPANY, ET AL.

CATAWBA NUCLEAR STATION, UNITS 1 AND 2

DOCKET NOS. 50-413 AND 50-414

1.0 INTRODUCTION

Catawba Unit 1 License Condition 2.C.(16) states that: "Prior to startup following the second refueling outage, Duke Power Company shall submit for NRC staff review and approval an analysis which demonstrates that the steam generator single-tube rupture analysis presented in the FSAR is the most severe case with respect to the release of fission products and calculated doses. Consistent with the analytical assumptions, Duke Power Company shall propose any necessary changes to Appendix A to this license." Catawba Unit 2 License Condition 2.C.(10) requires actions identical to those required by License Condition 2.C.(16) for Unit 1. However, Unit 2 is required to complete these actions prior to its startup following the first refueling outage while Unit 1 is required to complete them prior to its startup following the second refueling outage.

By letters dated December 4, 1987, and supplemented December 7, 11, and 29, 1987, March 29, May 4 and 18, June 16, July 1, August 8 and 24, and December 15, 1988, and June 12 and 28, 1989, Duke Power Company (the licensee) submitted the required steam generator tube rupture (SGTR) analysis and its associated Technical Specification (TS).

Because the June 28, 1989, submittal clarified and corrected certain aspects of the request, the substance of the changes noticed in the Federal Register and the proposed no significant hazards consideration were not affected.

2.0 EVALUATION

The NRC staff is currently reviewing two separate concerns related to License Conditions 2.C.(16) and 2.C.(10) of Operating License Nos. NPF-35 and NPF-52, respectively. The first concern is the SGTR accident analysis. Two scenarios were analyzed as requested by the staff: the scenario most conducive to steam generator overfill, and the scenario that maximizes offsite dose. With regard to the SGTR analysis, the staff is currently examining assumptions being used for operator action times used in the analysis. The second concern of the license conditions is the proposed TS

for operability of steam generator (SG) power-operated relief valves (PORVS). The staff is handling the two concerns separately and will address the acceptability of the SGTR analysis in future correspondence.

The Catawba-Plant design includes four SG PORVs, one for each SG. The operability of at least three of the four PORVs ensures that reactor decay heat can be dissipated to the atmosphere in the event of a SGTR coincident with a loss of offsite power. Reactor coolant system (RCS) cooling can subsequently be performed through the residual heat removal system (RHR). For RCS cooldown to RHR system initiation, only one PORV is required. Three operable PORVs are adequate, assuming that one of the operable PORVs is on the faulted SG, and thus unavailable for heat removal, and that one PORV fails to function in accordance with single failure assumptions.

Each SG PORV is equipped with an upstream electric motor operated block valve whose primary purpose is to allow isolation for repair or maintenance. The block valve can be used to isolate a partially or fully stuck open PORV. The motor operators fail "as is" and can be controlled from the control room. The power supplies and controls for the SG PORVs are Class 1E.

The Action statements included in the TS are consistent with those already accepted by the NRC staff. The proposed requirements state that three of the four SG PORVs shall be operable in Modes 1, 2, and 3. Also, in Mode 4 three of the four PORVs are required to be operable only when the SGs are being used for decay heat removal. If only two PORVs are operable, the Action statement allows 7 days to return one of the two inoperable valves to operable status. If only one is operable, the Action statement allows 72 hours to return at least one to operable status. The restoration time periods provided in the TS Action statements are based on the low likelihood of having a SGTR event coincident with a loss of offsite power during the time period that one (or more) of the required PORVs is (or are) out of service.

The proposed TS requirements are consistent with those already accepted by the NRC staff. They constitute additional limitations on facility operations, and satisfy, in part, the specific requirements of License Conditions 2.C.(16) and 2.C.(10) of Facility Operating Licenses NPF-35 and NPF-52, respectively. Based on its review, the staff concludes that the proposed TS has no adverse impact on safety and would not pose an undue risk to public health and safety. Therefore, it is acceptable.

3.0 ENVIRONMENTAL CONSIDERATION

These amendments involve a change to a requirement with respect to the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite

and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that these amendments involve no significant hazards consideration and there has been no public comment on such finding. Accordingly, these amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of these amendments.

4.0 CONCLUSION

The Commission made a proposed determination that the amendments involve no significant hazards consideration which was published in the Federal Register (54 FR29404) on July 12, 1989. The Commission consulted with the state of South Carolina. No public comments were received, and the state of South Carolina did not have any comments.

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributors: K. Jabbour, PDII-3/DRP-I/II
K. Desai, SRXB/DEST

Dated: August 15, 1989