

May 27, 1988

Docket Nos.: 50-413
and 50-414

Mr. H. B. Tucker, Vice President
Nuclear Production Department
Duke Power Company
422 South Church Street
Charlotte, North Carolina 28242

Dear Mr. Tucker:

Subject: ISSUANCE OF AMENDMENT NO. 45 TO FACILITY OPERATING LICENSE NPF-35
AND AMENDMENT NO. 38 TO FACILITY OPERATING LICENSE NPF-52 - CATAWBA
NUCLEAR STATION, UNITS 1 AND 2 (TACS 67612/67613)

The Nuclear Regulatory Commission has issued the enclosed Amendment No.45 to Facility Operating License NPF-35 and Amendment No.38 to Facility Operating License NPF-52 for the Catawba Nuclear Station, Units 1 and 2. These amendments consist of changes to the Technical Specifications in response to your application dated March 23, 1988.

The amendments modify the Technical Specifications to cover operation of systems and components associated with the Monitor Tank Building.

A copy of the related safety evaluation supporting Amendment No. 45 to Facility Operating License NPF-35 and Amendment No. 38 to Facility Operating License NPF-52 is enclosed.

Notice of issuance of amendments will be included in the Commission's next bi-weekly Federal Register notice.

Sincerely,

Original signed by:

Kahtan N. Jabbour, Project Manager
Project Directorate II-3
Division of Reactor Projects I/II

Enclosures:

- 1. Amendment No. 45 to NPF-35
- 2. Amendment No. 38 to NPF-52
- 3. Safety Evaluation

cc w/enclosures:
See next page

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5/12/88

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P PDR

Mr. H. B. Tucker
Duke Power Company

Catawba Nuclear Station

cc:

A.V. Carr, Esq.
Duke Power Company
422 South Church Street
Charlotte, North Carolina 28242

North Carolina Electric Membership
Corp.
3400 Sumner Boulevard
P.O. Box 27306
Raleigh, North Carolina 27611

J. Michael McGarry, III, Esq.
Bishop, Liberman, Cook, Purcell
and Reynolds
1200 Seventeenth Street, N.W.
Washington, D. C. 20036

Saluda River Electric Cooperative,
Inc.
P.O. Box 929
Laurens, South Carolina 29360

North Carolina MPA-1
Suite 600
3100 Smoketree Ct.
P.O. Box 29513
Raleigh, North Carolina 27626-0513

Senior Resident Inspector
Route 2, Box 179N
York, South Carolina 29745

S. S. Kilborn
Area Manager, Mid-South Area
ESSD Projects
Westinghouse Electric Corp.
MNC West Tower - Bay 239
P.O. Box 355
Pittsburgh, Pennsylvania 15230

Regional Administrator, Region II
U.S. Nuclear Regulatory Commission,
101 Marietta Street, NW, Suite 2900
Atlanta, Georgia 30323

County Manager of York County
York County Courthouse
York South Carolina 29745

Mr. Heyward G. Shealy, Chief
Bureau of Radiological Health
South Carolina Department of Health
and Environmental Control
2600 Bull Street
Columbia, South Carolina 29201

Richard P. Wilson, Esq.
Assistant Attorney General
S.C. Attorney General's Office
P.O. Box 11549
Columbia, South Carolina 29211

Karen E. Long
Assistant Attorney General
N.C. Department of Justice
P.O. Box 629
Raleigh, North Carolina 27602

Piedmont Municipal Power Agency
100 Memorial Drive
Greer, South Carolina 29651

Spence Perry, Esquire
General Counsel
Federal Emergency Management Agency
Room 840
500 C Street
Washington, D. C. 20472

Mr. Michael Hirsch
Federal Emergency Management Agency
Office of the General Counsel
Room 840
500 C Street, S.W.
Washington, D. C. 20472

Brian P. Cassidy, Regional Counsel
Federal Emergency Management Agency,
Region I
J. W. McCormach POCH
Boston, Massachusetts 02109



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

DUKE POWER COMPANY

NORTH CAROLINA ELECTRIC MEMBERSHIP CORPORATION

SALUDA RIVER ELECTRIC COOPERATIVE, INC.

DOCKET NO. 50-413

CATAWBA NUCLEAR STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 45
License No. NPF-35

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment to the Catawba Nuclear Station, Unit 1 (the facility) Facility Operating License No. NPF-35 filed by the Duke Power Company acting for itself, North Carolina Electric Membership Corporation and Saluda River Electric Cooperative, Inc., (licensees) dated March 23, 1988, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations as set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is hereby amended by page changes to the Technical Specifications as indicated in the attachments to this license amendment, and Paragraph 2.C.(2) of Facility Operating License No. NPF-35 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 45, are hereby incorporated into the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Original signed by:

David B. Matthews, Director
Project Directorate II-3
Division of Reactor Projects-I/II

Attachment:
Technical Specification Changes

Date of Issuance: May 27, 1988

OFFICIAL RECORD COPY

LA:PDII-3
MRobb
5/12/88

PM:PDII-3
KJabbour:pw
5/12/88

OGC
Woodhead
5/12/88

D:PDII-3
DMatthews
5/19/88

KNT

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

DUKE POWER COMPANY

NORTH CAROLINA MUNICIPAL POWER AGENCY NO. 1

PIEDMONT MUNICIPAL POWER AGENCY

DOCKET NO. 50-414

CATAWBA NUCLEAR STATION, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 38
License No. NPF-52

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment to the Catawba Nuclear Station, Unit 2 (the facility) Facility Operating License No. NPF-52 filed by the Duke Power Company acting for itself, North Carolina Municipal Power Agency No. 1 and Piedmont Municipal Power Agency, (licensees) dated March 23, 1988, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations as set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is hereby amended by page changes to the Technical Specifications as indicated in the attachments to this license amendment, and Paragraph 2.C.(2) of Facility Operating License No. NPF-52 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 38, are hereby incorporated into the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Original signed by:

David B. Matthews, Director
Project Directorate II-3
Division of Reactor Projects-I/II

Attachment:
Technical Specification Changes

Date of Issuance: May 27, 1988

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LA:PDII-3
MRood
5/12/88

PM:PDII-3
KJabbour:pw
5/12/88

OGC
5/16/88

D:PDII-3
DMatthews
5/19/88

ATTACHMENT TO LICENSE AMENDMENT NO. 45

FACILITY OPERATING LICENSE NO. NPF-35

DOCKET NO. 50-413

AND

TO LICENSE AMENDMENT NO. 38

FACILITY OPERATING LICENSE NO. NPF-52

DOCKET NO. 50-414

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the areas of change. The corresponding overleaf pages are also provided to maintain document completeness.

<u>Amended Page</u>	<u>Overleaf Page</u>
3/4 3-81	3/4 3-82
3/4 3-83	3/4 3-84
3/4 3-87	3/4 3-88
3/4 3-91	3/4 3-92
3/4 11-2	3/4 11-1
3/4 11-10a (new page)	
5-5	

TABLE 3.3-12

RADIOACTIVE LIQUID EFFLUENT MONITORING INSTRUMENTATION

<u>INSTRUMENT</u>	<u>MINIMUM CHANNELS OPERABLE</u>	<u>ACTION</u>
1. Radioactivity Monitors Providing Alarm and Automatic Termination of Release		
a. Waste Liquid Discharge Monitor (Low Range - EMF-49)	1 per station	40
b. Turbine Building Sump Monitor (Low Range - EMF-31)	1	42
c. Steam Generator Water Sample Monitor (Low Range - EMF-34)	1	43
d. Monitor Tank Building Liquid Discharge Monitor (EMF-57)	1 per station	40
2. Continuous Composite Samplers and Sampler Flow Monitor Conventional Waste Water Treatment Line	1 per station	42
3. Flow Rate Measurement Devices		
a. Waste Liquid Effluent Line	1 per station	41
b. Conventional Waste Water Treatment Line	1 per station	41
c. Low Pressure Service Water Minimum Flow Interlock	1 per station	41
d. Monitor Tank Building Waste Liquid Effluent Line	1 per station	41

CATAMBA - UNITS 1 & 2

3/4 3-81

Amendment No. 45 (Unit 1)
Amendment No. 38 (Unit 2)

TABLE 3.3-12 (Continued)

ACTION STATEMENTS

- ACTION 40 - With the number of channels OPERABLE less than required by the Minimum Channels OPERABLE requirement, effluent releases via this pathway may continue for up to 14 days provided that prior to initiating a release:
- a. At least two independent samples are analyzed in accordance with Specification 4.11.1.1.1, and
 - b. At least two technically qualified members of the facility staff independently verify:
 1. The discharge line valving, and
 2. The manual portion of the computer input for the release rate calculations performed on the computer, or the entire release rate calculations if such calculations are performed manually.Otherwise, suspend release of radioactive effluents via this pathway.
- ACTION 41 - With the number of channels OPERABLE less than required by the Minimum Channels OPERABLE requirement, effluent releases via this pathway may continue for up to 30 days provided the flow rate is estimated at least once per 4 hours during actual releases. Pump performance curves generated in place may be used to estimate flow.
- ACTION 42 - With the number of channels OPERABLE less than required by the Minimum Channels OPERABLE requirement, effluent releases via this pathway may continue for up to 30 days provided grab samples are analyzed for radioactivity at a lower limit of detection of no more than 10^{-7} microCurie/ml:
- a. At least once per 12 hours when the specific activity of the secondary coolant is greater than 0.01 microCurie/gram DOSE EQUIVALENT I-131, or
 - b. At least once per 24 hours when the specific activity of the secondary coolant is less than or equal to 0.01 microCurie/gram DOSE EQUIVALENT I-131.
- ACTION 43 - With the number of channels OPERABLE less than required by the Minimum Channels OPERABLE requirement, gaseous effluent releases via the atmospheric vent valves (off-normal mode) may continue provided grab samples of steam generator water are analyzed for radioactivity for up to 30 days at a lower limit of detection of no more than 10^{-7} microCurie/ml:
- a. At least once per 12 hours when the specific activity of the secondary coolant is greater than 0.01 microCurie/gram DOSE EQUIVALENT I-131, or
 - b. At least once per 24 hours when the specific activity of the secondary coolant is less than or equal to 0.01 microCurie/gram DOSE EQUIVALENT I-131.

TABLE 4.3-8

RADIOACTIVE LIQUID EFFLUENT MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS

<u>INSTRUMENT</u>	<u>CHANNEL CHECK</u>	<u>SOURCE CHECK</u>	<u>CHANNEL CALIBRATION</u>	<u>ANALOG CHANNEL OPERATIONAL TEST</u>
1. Radioactivity Monitors Providing Alarm and Automatic Termination of Release				
a. Waste Liquid Discharge Monitor (Low Range - EMF-49)	D	P	R(2)	Q(1)
b. Turbine Building Sump Monitor (Low Range - EMF-31)	D	M	R(2)	Q(1)
c. Steam Generator Water Sample Monitor (Low Range - EMF-34)	D	M	R(2)	Q(1)
d. Monitor Tank Building Liquid Discharge Monitor (EMF-57)	D	P	R(2)	Q(1)
2. Continuous Composite Samplers and Sampler Flow Monitor				
Conventional Waste Water Treatment Line	D(3)	N.A.	R	N.A.
3. Flow Rate Measurement Devices				
a. Waste Liquid Effluent Line	D(3)	N.A.	R	N.A.
b. Conventional Waste Water Treatment Line	D(3)	N.A.	R	N.A.
c. Low Pressure Service Water Minimum Flow Interlock	D(3)	N.A.	R	Q
d. Monitor Tank Building Waste Liquid Effluent Line	D(3)	N.A.	R	Q

CATAMBA - UNITS 1 & 2

3/4 3-83

Amendment No. 45 (Unit 1)
Amendment No. 38 (Unit 2)

TABLE 4.3-8 (Continued)

TABLE NOTATIONS

- (1) The ANALOG CHANNEL OPERATIONAL TEST shall also demonstrate that automatic isolation of this pathway and control room alarm annunciation occur if any of the following conditions exist:
 - a. Instrument indicates measured levels above the Alarm/Trip Setpoint,
or
 - b. Circuit failure (Alarm only), or
 - c. Instrument indicates a downscale failure (Alarm only).
- (2) The initial CHANNEL CALIBRATION shall be performed using one or more of the reference standards certified by the National Bureau of Standards (NBS) or using standards that have been obtained from suppliers that participate in measurement assurance activities with NBS. These standards shall permit calibrating the system over its intended range of energy and measurement range. For subsequent CHANNEL CALIBRATION, sources that have been related to the initial calibration shall be used.
- (3) CHANNEL CHECK shall consist of verifying indication of flow during periods of release. CHANNEL CHECK shall be made at least once per 24 hours on days on which continuous, periodic, or batch releases are made.

TABLE 3.3-13 (Continued)

RADIOACTIVE GASEOUS EFFLUENT MONITORING INSTRUMENTATION

	<u>INSTRUMENT</u>	<u>MINIMUM CHANNELS OPERABLE</u>	<u>APPLICABILITY</u>	<u>ACTION</u>
5.	Containment Purge System			
	Noble Gas Activity Monitor - Providing Alarm and Automatic Termination of Release (Low Range - EMF-39)	1	***	48
6.	Containment Air Release and Addition System			
	Noble Gas Activity Monitor - Providing Alarm (Low Range - EMF-39)	1	*	45
7.	Monitor Tank Building HVAC			
a.	Noble Gas Activity Monitor - Providing Alarm (EMF-58)	1 per station	***	47
b.	Monitor Tank Building Effluent Flow Rate Measuring Device	1 per station	***	46

CATAMBA - UNITS 1 & 2

3/4 3-87

Amendment No. 45 (Unit 1)
Amendment No. 38 (Unit 2)

TABLE 3.3-13 (Continued)

TABLE NOTATIONS

- * At all times except when the isolation valve is closed and locked.
- ** During WASTE GAS HOLDUP SYSTEM operation.
- *** At all times.

ACTION STATEMENTS

- ACTION 45 - With the number of channels OPERABLE less than required by the Minimum Channels OPERABLE requirement, the contents of the tank(s) may be released to the environment for up to 14 days provided that prior to initiating the release either:
- a. Vent system noble gas activity monitor providing alarm and automatic termination of release (Low Range - EMF-36) has at least one channel OPERABLE, or
 - b. At least two independent samples of the tank's contents are analyzed, and at least two technically qualified members of the facility staff independently verify:
 - 1. The discharge valve lineup, and
 - 2. The manual portion of the computer input for the release rate calculations performed on the computer, or the entire release rate calculations if such calculations are performed manually.
- Otherwise, suspend release of radioactive effluents via this pathway.
- ACTION 46 - With the number of channels OPERABLE less than required by the Minimum Channels OPERABLE requirement, effluent releases via this pathway may continue for up to 30 days provided the flow rate is estimated at least once per 4 hours.
- ACTION 47 - With the number of channels OPERABLE less than required by the Minimum Channels OPERABLE requirement, effluent releases via this pathway may continue for up to 30 days provided grab samples are taken at least once per 12 hours and these samples are analyzed for radioactivity within 24 hours.
- ACTION 48 - With the number of channels OPERABLE less than required by the Minimum Channels OPERABLE requirement, immediately suspend PURGING of radioactive effluents via this pathway.

TABLE 4.3-9 (Continued)

RADIOACTIVE GASEOUS EFFLUENT MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS

<u>INSTRUMENT</u>	<u>CHANNEL CHECK</u>	<u>SOURCE CHECK</u>	<u>CHANNEL CALIBRATION</u>	<u>ANALOG CHANNEL OPERATIONAL TEST</u>	<u>MODES FOR WHICH SURVEILLANCE IS REQUIRED</u>
4. Vent System (Continued)					
c. Particulate Sampler (EMF-35)	W	N.A.	N.A.	N.A.	*
d. Flow Rate Monitor	D	N.A.	R	N.A.	*
e. Sampler Flow Rate Monitor	D	N.A.	R	N.A.	*
5. Containment Purge System					
Noble Gas Activity Monitor - Providing Alarm and Automatic Termination of Release (Low Range - EMF-39)	D	P	R(3)	Q(1)	***
6. Containment Air Release and Addition System					
Noble Gas Activity Monitor- Providing Alarm (Low Range - EMF-39)	D	P	R(3)	Q(1)	*
7. Monitor Tank Building HVAC					
a. Noble Gas Activity Monitor - Providing Alarm (EMF-58)	D	M	R(3)	Q(2)	***
b. Discharge Flow Instrumentation	D	N.A.	R	N.A.	***

CATAMBA - UNITS 1 & 2

3/4 3-91

Amendment No. 45 (Unit 1)
Amendment No. 38 (Unit 2)

TABLE 4.3-9 (Continued)

TABLE NOTATIONS

- * At all times except when the isolation valve is closed and locked.
 - ** During WASTE GAS HOLDUP SYSTEM operation.
 - *** At all times.
- (1) The ANALOG CHANNEL OPERATIONAL TEST shall also demonstrate that automatic isolation of this pathway and control room alarm annunciation occur if any of the following conditions exists:
 - a. Instrument indicates measured levels above the Alarm/Trip Setpoint, or
 - b. Circuit failure (Alarm only), or
 - c. Instrument indicates a downscale failure (Alarm only).
 - (2) The ANALOG CHANNEL OPERATIONAL TEST shall also demonstrate that control room alarm annunciation occurs if any of the following conditions exists:
 - a. Instrument indicates measured levels above the Alarm Setpoint, or
 - b. Circuit failure, or
 - c. Instrument indicates a downscale failure.
 - (3) The initial CHANNEL CALIBRATION shall be performed using one or more of the reference standards certified by the National Bureau of Standards (NBS) or using standards that have been obtained from suppliers that participate in measurement assurance activities with NBS. These standards shall permit calibrating the system over its intended range of energy and measurement range. For subsequent CHANNEL CALIBRATION, sources that have been related to the initial calibration shall be used.
 - (4) The CHANNEL CALIBRATION shall include the use of standard gas samples in accordance with the manufacturer's recommendations. In addition, a standard gas sample of nominal four volume percent hydrogen, balance nitrogen, shall be used in the calibration to check linearity of the hydrogen analyzer.
 - (5) The CHANNEL CALIBRATION shall include the use of standard gas samples in accordance with the manufacturer's recommendations. In addition, a standard gas sample of nominal four percent oxygen, balance nitrogen, shall be used in the calibration to check linearity of the oxygen analyzer.

3/4.11 RADIOACTIVE EFFLUENTS

3/4.11.1 LIQUID EFFLUENTS

CONCENTRATION

LIMITING CONDITION FOR OPERATION

3.11.1.1 The concentration of radioactive material released in liquid effluents to UNRESTRICTED AREAS (see Figure 5.1-3) shall be limited to the concentrations specified in 10 CFR Part 20, Appendix B, Table II, Column 2 for radionuclides other than dissolved or entrained noble gases. For dissolved or entrained noble gases, the concentration shall be limited to 2×10^{-4} microCurie/ml total activity.

APPLICABILITY: At all times.

ACTION:

With the concentration of radioactive material released in liquid effluents to UNRESTRICTED AREAS exceeding the above limits, immediately restore the concentration to within the above limits.

SURVEILLANCE REQUIREMENTS

4.11.1.1.1 Radioactive liquid wastes shall be sampled and analyzed according to the sampling and analysis program of Table 4.11-1.

4.11.1.1.2 The results of the radioactivity analyses shall be used in accordance with the methodology and parameters in the ODCM to assure that the concentrations at the point of release are maintained within the limits of Specification 3.11.1.1.

TABLE 4.11-1

RADIOACTIVE LIQUID WASTE SAMPLING AND ANALYSIS PROGRAM

LIQUID RELEASE TYPE	SAMPLING FREQUENCY	MINIMUM ANALYSIS FREQUENCY	TYPE OF ACTIVITY ANALYSIS	LOWER LIMIT OF DETECTION (LLD) ⁽¹⁾ ($\mu\text{Ci/ml}$)	
1. Batch Waste Release Tanks ⁽²⁾ Any tank which discharges liquid wastes by either liquid effluent monitor, EMF-49 or EMF-58	P Each Batch	P Each Batch	Principal Gamma Emitters ⁽³⁾	5×10^{-7}	
			I-131	1×10^{-6}	
	P One Batch/M	M	Dissolved and Entrained Gases (Gamma Emitters)		1×10^{-5}
				P Each Batch	M Composite ⁽⁴⁾
	Gross Alpha	1×10^{-7}			
	P Each Batch	Q Composite ⁽⁴⁾	Sr-89, Sr-90	5×10^{-8}	
			Fe-55	1×10^{-6}	
	2. Continuous Releases ⁽⁵⁾ Conventional Waste Water Treatment Line	Continuous ⁽⁶⁾	W Composite ⁽⁶⁾	Principal Gamma Emitters ⁽³⁾	5×10^{-7}
I-131				1×10^{-6}	
M Grab Sample		M	Dissolved and Entrained Gases (Gamma Emitters)		1×10^{-5}
				Continuous ⁽⁶⁾	M Composite ⁽⁶⁾
Gross Alpha		1×10^{-7}			
Continuous ⁽⁶⁾		Q Composite ⁽⁶⁾	Sr-89, Sr-90	5×10^{-8}	
			Fe-55	1×10^{-6}	

Table 4.11-2 (Continued)

RADIOACTIVE GASEOUS WASTE SAMPLING AND ANALYSIS PROGRAM

GASEOUS RELEASE TYPE	SAMPLING FREQUENCY	MINIMUM ANALYSIS FREQUENCY	TYPE OF ACTIVITY ANALYSIS	LOWER LIMIT OF DETECTION (LLD) ⁽¹⁾ ($\mu\text{Ci/ml}$)
6. Waste Monitor Tank Building Ventilation Exhaust	W Grab Sample	W	Principal Gamma Emitters ⁽²⁾	1×10^{-4}
			H-3 (oxide)	1×10^{-6}
	Continuous ⁽⁶⁾	W Charcoal Sample	I-131	1×10^{-11}
			I-133	1×10^{-9}
	Continuous ⁽⁶⁾	W Particulate Sample	Principal Gamma Emitters ⁽²⁾	1×10^{-10}
	Continuous ⁽⁶⁾	M Composite Particulate Sample	Gross Alpha	1×10^{-11}
Continuous ⁽⁶⁾	Q Composite Particulate Sample	Sr-89, Sr-90	1×10^{-11}	

CATAMBA - UNITS 1 & 2

3/4 11-10a

Amendment No. 45 (Unit 1)
Amendment No. 38 (Unit 2)

SITE BOUNDARY/
PERIMETER FENCE

NSW
DAM

NUCLEAR SERVICE
WATER POND

2500 FT. R.
EXCLUSION
BOUNDARY

INTAKE STRUCTURE

MONITOR TANK BUILDING
(GASEOUS RELEASE POINT)

REACTOR
BLDG.

STATION VENTS
(GASEOUS RELEASE
POINTS EL. 718.75)

SWITCH
YARD

AUX.
BLDG.

ACCESS ROAD

TRAINING
CENTER

COOLING TOWER
YARD

DISCHARGE
STRUCTURE
(LIQUID RELEASE
POINT)

CO RD 1132

FIGURE 5-1-4

UNRESTRICTED AREA AND SITE BOUNDARY FOR RADIOACTIVE GASEOUS EFFLUENTS



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 45 TO FACILITY OPERATING LICENSE NPF-35
AND AMENDMENT NO. 38 TO FACILITY OPERATING LICENSE NPF-52

DUKE POWER COMPANY, ET AL.

CATAWBA NUCLEAR STATION, UNITS 1 AND 2

DOCKET NOS. 50-413 AND 50-414

I. INTRODUCTION

By letter dated March 23, 1988, Duke Power Company, et al., (the licensee) proposed amendments to the operating licenses for Catawba Nuclear Station, Units 1 and 2, which would revise Technical Specification (TS) Tables 3.3-12, 3.3-13, 4.3-8, 4.3-9, 4.11-1 and 4.11-2 to add TS requirements to cover operation of systems and components associated with the Monitor Tank Building (MTB) which is being constructed at Catawba Nuclear Station. Also, TS Figure 5.1-4 "Unrestricted Area and Site Boundary for Radioactive Gaseous Effluent" will be revised to show the MTB as a potential release point.

II. EVALUATION

At the present time, Catawba does not have the capability to process large volumes of liquid radwaste due to restrictions on releases and release rates. This is particularly true for peak load conditions associated with routine plant operations such as during refueling outages.

The MTB and associated components, including additional tankage, will increase process rates and ensure segregation for the various liquid waste streams. By providing a piping arrangement and process area to accommodate portable temporary equipment, the facility will provide surge capacity and processing flexibility to incorporate such future problems as load cycling, ice condenser ice melt and potential volume reduction requirements.

The MTB includes many ALARA design features that will reduce the maintenance and operations dose currently received. Its primary functions are to provide additional processing capacity for high radwaste inventories during normal operation, primary to secondary leaks, and contaminated powdex processing.

The MTB and associated trenches do not house any equipment which is important to safety and being a remote facility, cannot adversely affect any equipment which is important to safety. An accident or malfunction within the facility can, however, result in a radioactive release to the environment. The most severe consequences would be those following a tank failure.

The accident which is already analyzed in the FSAR is the failure of the refueling water storage tank (RWST) which results in the release of 395,000 gallons of contaminated water directly to Lake Wylie. Since the total volume

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of all MTB tankage is much less than that of the RWST and since the radionuclide concentrations of liquids within the MTB will be less than those assumed in the RWST analysis, the consequences of the MTB accident will be much less severe than the RWST accident. The releases resulting from the postulated RWST failure were determined to be within the limits of 10 CFR 20, Appendix B.

Accidents and malfunctions within the MTB will, therefore, not affect the safe operation or shutdown of the plant and will not adversely affect the health and safety of the public.

Based on our evaluation, we conclude that the licensee's proposed changes to the Radiological Effluent Technical Specifications (RETS) meet the intent of the NRC staff's model, NUREG-0472, "Standard Radiological Effluent Technical Specifications for Pressurized Water Reactors," Revision 2, dated February 1, 1980, and are therefore acceptable.

III. ENVIRONMENTAL CONSIDERATION

Pursuant to 10 CFR 51.32, the Commission has determined that the issuance of these amendments will have no significant impact on the environment (53 FR19059).

IV. CONCLUSION

The Commission issued a Notice of Consideration of Issuance of Amendments and Opportunity for Hearing which was published in the Federal Register (53 FR 14874) on April 26, 1988. The Commission consulted with the state of South Carolina. No public comments were received, and the state of South Carolina did not have any comments.

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: K. Jabbour, PD#II-3/DRP-I/II
W. Meinke, RPB/DREP

Dated: May 27, 1988

UNITED STATES NUCLEAR REGULATORY COMMISSIONDUKE POWER COMPANY, et al.DOCKET NOS. 50-413 AND 50-414NOTICE OF ISSUANCE OF AMENDMENTS TOFACILITY OPERATING LICENSES

The U.S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 45 to Facility Operating License No. NPF-35 and Amendment No. 38 to Facility Operating License No. NPF-52 issued to Duke Power Company, et al., (the licensee) which revised the Technical Specifications for operation of the Catawba Nuclear Station, Units 1 and 2, (the facility) located in York County, South Carolina. The amendments were effective as of the date of issuance.

The application for the amendments complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendments.

Notice of Consideration of Issuance of Amendments and Opportunity for Hearing in connection with this action was published in the FEDERAL REGISTER on April 26, 1988 (53 FR 14874). No request for a hearing or petition for leave to intervene was filed following this notice.

The commission has prepared an Environmental Assessment and Finding of No Significant Impact (53 FR 19059) related to the action and has concluded that an environmental impact statement is not warranted because there will be no significant environmental impact attributable to the action.

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For further details with respect to the action see (1) the application for amendments dated March 23, 1988, (2) Amendment No. 45 to License No. NPF-35 and Amendment No. 38 to License No. NPF-52 and (3) the Commission's related Safety Evaluation and Environmental Assessment. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., and at the York County Library, 138 East Black Street, Rock Hill, South Carolina 29730. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Director, Division of Reactor Projects I/II.

Dated at Rockville, Maryland this 27th day of May 1988.

FOR THE NUCLEAR REGULATORY COMMISSION

Original signed by:

Kahtan N. Jabbour, Project Manager
Project Directorate II-3
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

LA:PDII-3
MRood
5/12/88

KNJ
K. JABBOUR
PM:PDII-3
Hood:pw
5/14/88

OGC-WF
w/ change to PEN
5/16/88

D:PDII-3
DMatthews
5/19/88

May 27, 1988

DOCKET NO. S. 50-413
50-414

MEMORANDUM FOR: Rules and Procedures Branch
Division of Rules and Records
Office of Administration

FROM: Office of Nuclear Reactor Regulation

SUBJECT: Catawba Nuclear Station, Units 1 and 2 (Duke Power Company, et al)

One signed original of the *Federal Register* Notice identified below is enclosed for your transmittal to the Office of the Federal Register for publication. Additional conformed copies (5) of the Notice are enclosed for your use.

- Notice of Receipt of Application for Construction Permit(s) and Operating License(s).
- Notice of Receipt of Partial Application for Construction Permit(s) and Facility License(s); Time for Submission of Views on Antitrust Matters.
- Notice of Consideration of Issuance of Amendment to Facility Operating License.
- Notice of Receipt of Application for Facility License(s); Notice of Availability of Applicant's Environmental Report; and Notice of Consideration of Issuance of Facility License(s) and Notice of Opportunity for Hearing.
- Notice of Availability of NRC Draft/Final Environmental Statement.
- Notice of Limited Work Authorization.
- Notice of Availability of Safety Evaluation Report.
- Notice of Issuance of Construction Permit(s).
- Notice of Issuance of Facility Operating License(s) ~~and~~ Amendment(s).
- Order.
- Exemption.
- Notice of Granting Exemption.
- Environmental Assessment.
- Notice of Preparation of Environmental Assessment.
- Other: _____

Office of Nuclear Reactor Regulation

Enclosure:
As stated

Contact: Marilee Rood
Phone: 21487

OFFICE	PD II-3					
NAME	MRood					
DATE	5/27/88					

DATED: May 27, 1988

AMENDMENT NO.45 TO FACILITY OPERATING LICENSE NPF-35 - Catawba Nuclear Station, Unit 1
AMENDMENT NO.38 TO FACILITY OPERATING LICENSE NPF-52 - Catawba Nuclear Station, Unit 2

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May 27, 1988

DOCKET NO. 8. 50-413
50-414

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Division of Rules and Records
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Office of Nuclear Reactor Regulation

Enclosure:
As stated

Contact: Marilee Rood
Phone: 21487

OFFICE ▶	PD II-3						
SURNAME ▶	MRood						
DATE ▶	5/27/88						