

Docket Nos.: 50-413
and 50-414

June 1, 1988

Mr. H. B. Tucker, Vice President
Nuclear Production Department
Duke Power Company
422 South Church Street
Charlotte, North Carolina 28242

Dear Mr. Tucker:

SUBJECT: ISSUANCE OF AMENDMENT NO. 46 TO FACILITY OPERATING LICENSE NPF-35
AND AMENDMENT NO. 39 TO FACILITY OPERATING LICENSE NPF-52 - CATAWBA
NUCLEAR STATION, UNITS 1 AND 2 (TACS 68251/68252)

The Nuclear Regulatory Commission has issued the enclosed Amendment No. 46 to Facility Operating License NPF-35 and Amendment No. 39 to Facility Operating License NPF-52 for the Catawba Nuclear Station, Units 1 and 2. These amendments consist of changes to the Technical Specifications (TS) in response to your application dated May 26, 1988, as supplemented May 27, 1988. The changes were approved by telephone on May 27, 1988, and confirmed by letter on that same date.

The amendments modify the total number of channels for the PORV block valve position indicator from 2 per valve to 1 per valve for Catawba Unit 2 only.

A copy of the related safety evaluation supporting Amendment No. 46 to Facility Operating License NPF-35 and Amendment No. 39 to Facility Operating License NPF-52 is enclosed.

Notice of issuance of amendments and opportunity for hearing will be included in the Commission's next bi-weekly Federal Register notice.

Sincerely,

D. B. Matthews for

8806070118 880601
PDR ADOCK 05000413
P PDR

Gus C. Lainas, Assistant Director
for Region II Reactors
Division of Reactor Projects I/II
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 46 to NPF-35
2. Amendment No. 39 to NPF-52
3. Safety Evaluation

cc w/enclosures:
See next page

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Mr. H. B. Tucker
Duke Power Company

Catawba Nuclear Station

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DATED: June 1, 1988

AMENDMENT NO. TO FACILITY OPERATING LICENSE NPF-35 - Catawba Nuclear Station, Unit 1
AMENDMENT NO. TO FACILITY OPERATING LICENSE NPF-52 - Catawba Nuclear Station, Unit 2

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

DUKE POWER COMPANY

NORTH CAROLINA ELECTRIC MEMBERSHIP CORPORATION

SALUDA RIVER ELECTRIC COOPERATIVE, INC.

DOCKET NO. 50-413

CATAWBA NUCLEAR STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 46
License No. NPF-35

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment to the Catawba Nuclear Station, Unit 1 (the facility) Facility Operating License No. NPF-35 filed by the Duke Power Company acting for itself, North Carolina Electric Membership Corporation and Saluda River Electric Cooperative, Inc., (licensees) dated May 26, 1988, as supplemented May 27, 1988, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations as set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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PDR ADOCK 05000413
P PDR

2. Accordingly, the license is hereby amended by page changes to the Technical Specifications as indicated in the attachments to this license amendment, and Paragraph 2.C.(2) of Facility Operating License No. NPF-35 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 46, are hereby incorporated into the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment was effective as of May 27, 1988.

FOR THE NUCLEAR REGULATORY COMMISSION

D. B. Matthews for

Gus C. Lainas, Assistant Director
for Region II Reactors
Division of Reactor Projects I/II
Office of Nuclear Reactor Regulation

Attachment:
Technical Specification Changes

Date of Issuance: June 1, 1988

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KJabbour:pw
5/31/88

J. Goldberg
6/1/88

D. Matthews
6/1/88

By phone
L. Reyes
6/1/88

G. Lainas
6/1/88



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

DUKE POWER COMPANY

NORTH CAROLINA MUNICIPAL POWER AGENCY NO. 1

PIEDMONT MUNICIPAL POWER AGENCY

DOCKET NO. 50-414

CATAWBA NUCLEAR STATION, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 39
License No. NPF-52

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment to the Catawba Nuclear Station, Unit 2 (the facility) Facility Operating License No. NPF-52 filed by the Duke Power Company acting for itself, North Carolina Municipal Power Agency No. 1 and Piedmont Municipal Power Agency, (licensees) dated May 26, 1988, as supplemented May 27, 1988, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations as set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is hereby amended by page changes to the Technical Specifications as indicated in the attachments to this license amendment, and Paragraph 2.C.(2) of Facility Operating License No. NPF-52 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 39, are hereby incorporated into the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment was effective as of May 27, 1988.

FOR THE NUCLEAR REGULATORY COMMISSION

D. B. Matthews for

Gus C. Lainas, Assistant Director
for Region II Reactors
Division of Reactor Projects I/II
Office of Nuclear Reactor Regulation

Attachment:
Technical Specification Changes

Date of Issuance: June 1, 1988

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6/1/88

ATTACHMENT TO LICENSE AMENDMENT NO. 46

FACILITY OPERATING LICENSE NO. NPF-35

DOCKET NO. 50-413

AND

TO LICENSE AMENDMENT NO. 39

FACILITY OPERATING LICENSE NO. NPF-52

DOCKET NO. 50-414

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised page is identified by Amendment number and contains a vertical line indicating the area of change. The corresponding overleaf page is also provided to maintain document completeness.

Amended Page

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Overleaf Page

3/4 3-65

INSTRUMENTATION

ACCIDENT MONITORING INSTRUMENTATION

LIMITING CONDITION FOR OPERATION

3.3.3.6 The accident monitoring instrumentation channels shown in Table 3.3-10 shall be OPERABLE.

APPLICABILITY: MODES 1, 2, and 3.

ACTION:

- a. With the number of OPERABLE accident monitoring instrumentation channels less than the Total Number of Channels shown in Table 3.3-10, restore the inoperable channel(s) to OPERABLE status within 7 days, or be in at least HOT STANDBY within the next 6 hours and in HOT SHUTDOWN within the following 6 hours.
- b. With the number of OPERABLE accident monitoring instrumentation channels except the unit vent-high-high range area monitor, the steam relief valve exhaust radiation monitor, the containment atmosphere-high range radiation monitor, and the reactor coolant radiation level less than the Minimum Channels OPERABLE requirements of Table 3.3-10, restore the inoperable channel(s) to OPERABLE status within 48 hours or be in at least HOT STANDBY within the next 6 hours and in HOT SHUTDOWN within the following 6 hours.
- c. With the number of OPERABLE Channels for the unit vent-high-high range area monitor, or the steam relief valve exhaust radiation monitor, or the containment atmosphere-high range radiation monitor, or the reactor coolant radiation level less than required by the Minimum Channels OPERABLE requirements, initiate an alternate method of monitoring the appropriate parameter(s) within 72 hours, and either restore the inoperable channel(s) to OPERABLE status within 7 days or prepare and submit a Special Report to the Commission pursuant to Specification 6.9.2 within 14 days that provides actions taken, cause of the inoperability, and the plans and schedule for restoring the channels to OPERABLE status.
- d. The provisions of Specification 3.0.4 are not applicable.

SURVEILLANCE REQUIREMENTS

4.3.3.6 Each accident monitoring instrumentation channel shall be demonstrated OPERABLE by performance of the CHANNEL CHECK and CHANNEL CALIBRATION operations at the frequencies shown in Table 4.3-7.

TABLE 3.3-10

ACCIDENT MONITORING INSTRUMENTATION

<u>INSTRUMENT</u>	<u>TOTAL NO. OF CHANNELS</u>	<u>MINIMUM CHANNELS OPERABLE</u>
1. Containment Pressure	2	1
2. Reactor Coolant Outlet Temperature - T _{HOT} (Wide Range)	2	1
3. Reactor Coolant Inlet Temperature - T _{COLD} (Wide Range)	2	1
4. Reactor Coolant Pressure - Wide Range	2	1
5. Pressurizer Water Level	2	1
6. Steam Line Pressure	2/steam generator	1/steam generator
7. Steam Generator Water Level - Narrow Range	2/steam generator	1/steam generator
8. Refueling Water Storage Tank Water Level	2	1
9. Auxiliary Feedwater Flow Rate	2/steam generator	1/steam generator
10. Reactor Coolant System Subcooling Margin Monitor	1	1
11. PORV Position Indicator*	2/Valve	1/Valve
12. PORV Block Valve Position Indicator**		
a. Unit 1	2/Valve	1/Valve
b. Unit 2	1/Valve	1/Valve
13. Pressurizer Safety Valve Position Indicator	1/Valve	1/Valve
14. Containment Sump Water Level (Wide Range)	2	1

CATAMBA - UNITS 1 & 2

3/4 3-66

Amendment No. 46 (Unit 1)
 Amendment No. 39 (Unit 2)
 Effective Date: May 27, 1988



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 46 TO FACILITY OPERATING LICENSE NPF-35
AND AMENDMENT NO. 39 TO FACILITY OPERATING LICENSE NPF-52

DUKE POWER COMPANY, ET AL.

CATAWBA NUCLEAR STATION, UNITS 1 AND 2

DOCKET NOS. 50-413 AND 50-414

I. INTRODUCTION

By letters dated May 26 and 27, 1988, Duke Power Company, et al. (the licensee), proposed amendments to the operating licenses for Catawba Nuclear Station, Units 1 and 2, which would revise Technical Specifications (TS) Table 3.3-10. The change is to reduce the total number of channels for the power operated relief valve (PORV) block valve position indicator from 2/valve to 1/valve. The change is applicable to Catawba Unit 2 only, however, Unit 1 is included administratively because the TSs for both units are combined in one document.

The licensee requested that the proposed amendments be granted on an emergency basis. Catawba Units 1 and 2 were placed in a seven day Action Statement (Action a. of TS 3.3.3.6) at 1650 hours on Friday, May 20, 1988, when the existence of a second independent channel of position indication was called into question. Subsequently, a second position indication per valve was identified and appropriate surveillances were performed. Two of the three non-safety position indicators at Catawba Unit 2 did not pass the surveillance test. These non-safety indicators cannot be repaired or replaced with the unit on-line.

On May 27, 1988, at 1355, a trip occurred at Catawba Unit 2. Subsequently, the licensee supplemented its May 26, 1988 letter to inform the staff that absent the approval of this emergency TS change, the restart of Unit 2 would be delayed approximately 1 to 2 days.

Telephone authorization was granted on an emergency basis on May 27, 1988, and confirmed by letter dated May 27, 1988.

II. EVALUATION

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The licensee stated in its May 26, 1988, submittal that each of the three pressurizer PORVs has an associated motor operated block valve which is safety related. In accordance with NUREG-0737, Item II.G.1, they are capable of being supplied from either the offsite power source or the emergency power source when offsite power is not available. One channel of safety related position indication is provided for each block valve by means of control room indicating lights associated with the valve control switch. Each block valve has a second non-safety related position indication channel which was provided in the design for maintenance and test purposes but was not intended to be used as accident monitoring instrumentation nor relied upon during an accident. No credit was taken for this non-safety indication in the design of the system or in any accident analyses.

Regulatory Guide 1.97, "Instrumentation For Light-Water-Cooled Nuclear Power Plants to Assess Plant and Environs Conditions During and Following an Accident," Revision 2, December 1980, does not include any recommendations for PORV block valve position indication. There are no other requirements for two channels of indication per PORV block valve. The Catawba Final Safety Evaluation Report does not include a second channel for indication.

Based on its review, the staff agrees with the licensee's assessment and finds that the reduction in the total number of channels for Catawba Unit 2 PORV block valve position indicator from 2/valve to 1/valve is acceptable.

III. FINAL NO SIGNIFICANT HAZARDS CONSIDERATION DETERMINATION

The State of South Carolina was informed by telephone on May 27, 1988, of the staff's no significant hazards consideration determination. The State contact had no comments on the determination.

The staff has reviewed the licensee's request for the above amendments and determined that should this request be implemented, it would not (1) involve a significant increase in the probability or consequences of an accident previously evaluated because the accident analysis takes credit only for the safety related PORV block valve position indication. The second channel is not safety related and is not relied upon during any accident scenario. Therefore, changing the requirement from two indicators per valve to one indicator per valve will have no effect on the probability or consequences of the accidents previously analyzed. Also, the licensee's proposed amendments would not (2) create the possibility of a new or different kind of accident from any accident previously evaluated because the design and modes of operation of the station will not be affected by changing the total number of PORV block valve position indicators required by the TSs. The second channel of position indication is not safety related and is not relied upon in the accident analyses. As such, no new or different kind of accident would be possible. Finally, the proposed amendments would not (3) involve a significant reduction in a margin of safety because of the reasons stated above in items (1) and (2).

Accordingly, the Commission finds that this request does not involve a significant hazards consideration.

IV. FINDINGS OF EMERGENCY WARRANTING AN AMENDMENT WITHOUT NOTICE

The licensee's application for the TS change has been timely. The licensee states that Catawba Units 1 and 2 were placed in a seven day Action Statement (Action a. of TS 3.3.3.6) on Friday May 20, 1988, when the existence of a second independent channel of position indication was called into question. Subsequently, a second position indication per valve was identified and surveillances were performed. Two of the three non-safety position indicators in Catawba Unit 2 did not pass the surveillance test. These indicators cannot be repaired or replaced with the unit on-line. The licensee's request for emergency amendments resulted from these surveillance tests performed in accordance with TS 3.3.3.6. The non-safety channels in Catawba Unit 1 passed the tests and the licensee could not reasonably predict the outcome of the test for Catawba Unit 2.

The licensee's letter dated May 26, 1988, outlines the sequence of events (from September 25, 1987 till May 26, 1988) that led to the emergency TS amendment request.

On May 27, 1988, at 1355, a trip occurred at Catawba Unit 2. Subsequently, the licensee supplemented its May 26, 1988 letter to inform the staff that absent the approval of this emergency TS change, the restart of Unit 2 would be delayed approximately 1 to 2 days.

The staff finds that failure to grant the proposed changes in a timely manner would increase the outage time of the Catawba Unit 2. We also find that the licensee could not reasonably have avoided this situation, that the licensee has responded in a timely manner, and has not delayed its application to take advantage of the Emergency License Amendments provisions of 10 CFR 50.91. Accordingly, the staff concludes that the licensee has satisfied the requirements of 10 CFR 50.91(a)(5), and that a valid emergency exists.

V. ENVIRONMENTAL CONSIDERATION

These amendments involve changes to the installation or use of facility components located within the restricted area as defined in 10 CFR Part 20. The staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite and that there is no significant increase in individual or cumulative occupational exposure. The Commission has made a final no significant hazards consideration finding with respect to this amendment. Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of these amendments.

VI. CONCLUSION

We have concluded, based on the considerations discussed above, that: (1) the amendment does not (a) significantly increase the probability or consequences of an accident previously evaluated, (b) create the possibility of a new or different kind of accident from any previously evaluated or (c) significantly reduce a safety margin and, therefore, the amendment does not involve significant hazards consideration; (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations, and the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: K. Jabbour, PD#II-3/DRP-I/II
Barry S. Marcus, SICB/DEST

Dated: June 1, 1988