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Docket Nos.: 50-413 and 50-414

> Mr. H. B. Tucker, Vice President Nuclear Production Department Duke Power Company 422 South Church Street Charlotte, North Carolina 28242

Dear Mr. Tucker:

Subject: Amendment Correction

My December 14, 1987, letter forwarded Amendments 37 and 29 to Facility Operating Licenses NPF-35 and NPF-52 for the Catawba Nuclear Station, Units 1 and 2. The amendments included changes to the Technical Specifications. Please replace pages 3/4 4-22 and B 3/4 6-3 included with those amendments with the enclosed revised pages.

Sincerely,

Kahtan N. Jabbour, Project Manager Project Directorate II-3 Division of Reactor Projects, I/II

Enclosure: As stated

cc w/enclosure: See next page

DISTRIBUTION: Docket File NRC PDR Local PDR PRC System PDII-3 Reading M. Rood K. Jabbour OGC-Bethesda D. Hagan J. Stone E. Jordan L. Harmon W. Jones T. Barnhart (4) ACRS (10) GPA/PA 8802090295 880125 ADOCK 05000413 ARM/LFMB PDR PDR L. Reyes S. Varga/G. Lainas E. Butcher PD#II-3/DRP-I/II PD#II-3/DRP-I/II PD#私了3/DRP-1/II MRood/mac KJabbour Acting PD 01/19/88 01/ (//88 01/19/88

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BASES

3/4.6.1.8 ANNULUS VENTILATION SYSTEM

The OPERABILITY of the Annulus Ventilation System ensures that during LOCA conditions, containment vessel leakage into the annulus will be filtered through the HEPA filters and activated Carbon adsorber trains prior to discharge to the atmosphere. Operation of the system with the heaters operating to maintain low humidity using automatic control for at least 10 continuous hours in a 31-day period is sufficient to reduce the buildup of moisture on the adsorbers and HEPA filters. This requirement is necessary to meet the assumptions used in the safety analyses and limit the SITE BOUNDARY radiation doses to within the dose guide-line values of 10 CFR Part 100 during LOCA conditions. ANSI N510-1980 will be used as a procedural guide for surveillance testing.

3/4.6.1.9 CONTAINMENT PURGE SYSTEMS

The containment purge supply and exhaust isolation valves for the lower compartment and the upper compartment (24-inch), and instrument room (12-inch), and the Hydrogen Purge System (4-inch) are required to be sealed closed during plant operation since these valves have not been demonstrated capable of closing during a LOCA. Maintaining these valves sealed closed during plant operations ensures that excessive quantities of radioactive materials will not be released via the Containment Purge System. To provide assurance that these containment valves cannot be inadvertently opened, the valves are sealed closed in accordance with Standard Review Plan 6.2.4 which includes mechanical devices to seal or lock the valve closed, or prevents power from being supplied to the valve operator.

The use of the containment purge lines is restricted to the 4-inch Containment Air Release and Addition System valves since, unlike the lower compartment and the upper compartment, instrument room, and the Hydrogen Purge System valves, these 4-inch valves are capable of closing during a LOCA. Therefore, the SITE BOUNDARY dose guideline values of 10 CFR Part 100 would not be exceeded in the event of an accident during containment purging operation. Operation with the line open will be limited to 3000 hours during a calendar year for the 4-inch valves. The total time the containment purge (vent) system isolation valves may be open during MODES 1, 2, 3, and 4 in a calendar year is a function of anticipated need and operating experience. Only safety-related reasons; e.g., containment pressure control or the reduction of airborne radioactivity to facilitate personnel access for surveillance and maintenance activities, may be used to justify the opening of these isolation valves.

Leakage integrity tests with a maximum allowable leakage rate for containment purge supply and exhaust values will provide early indication of resilient material seal degradation and will allow opportunity for repair before gross leakage failures could develop. The 0.60 L_a leakage limit of Specification

3.6.1.2b. shall not be exceeded when the leakage rates determined by the leakage integrity tests of these values are added to the previously determined total for all values and penetrations subject to Type B and C tests.

CATAWB4 - UNITS 1 & 2

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Amenament No. 37 (Unit 1) Amenament No. 29 (Unit 2)

TABLE 3.4-1

REACTOR COOLANT SYSTEM PRESSURE ISOLATION VALVES

VALVE NUMBER	FUNCTION
NI59	Accumulator Discharge
NI60	Accumulator Discharge
NI70	Accumulator Discharge
NI71	Accumulator Discharge
NI81	Accumulator Discharge
NI82	Accumulator Discharge
NI93	Accumulator Discharge
NI94	Accumulator Discharge
NI124	Safety Injection (Hot Leg)
NI125	Residual Heat Removal (Hot Leg)
NI126	Safety Injection (Hot Leg)
NI128	Safety Injection (Hot Leg)
NI129	Residual Heat Removal (Hot Leg)
NI134	Safety Injection (Hot Leg)
NI156	Safety Injection (Hot Leg)
NI157	Safety Injection (Hot Leg)
NI159	Safety Injection (Hot Leg)
NI160	Safety Injection (Hot Leg)
NI165 NI167 NI169 NI171 NI175 NI176 NI180 NI181	Safety Injection/Residual Heat Removal (Cold Leg) Safety Injection/Residual Heat Removal (Cold Leg)
NI248	Upper Head Injection
NI249	Upper Head Injection
NI250	Upper Head Injection
NI251	Upper Head Injection
NI252	Upper Head Injection
NI253	Upper Head Injection

Note 1: Upon the disconnection of the UHI System from the Reactor Coolant System, this specification is no longer applicable.

Amendment No.32 (Unit 1) Amendment No.23 (Unit 2)

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